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 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
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 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-004-00:on 890324,containment Type B & C leakage
 exceeds LCO value.

W/8 ltr.

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D.C. Cook Nuclear Plant, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 1 5					PAGE (3) 1 OF 0 1							
TITLE (4) Containment Type B and C Leakage Exceeds L.C.O. Value Due to Degradation of Isolation Valve Seating Surfaces																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)									
0	3	2	4	8	9	8	9	0	0	4	0	0	0	4	2	1	8	9	0 5 0 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																				
5		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)								
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)								
0 1 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)								
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)												
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)												
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)												
LICENSEE CONTACT FOR THIS LER (12)																						
NAME										TELEPHONE NUMBER												
T.K. Postlewait - Technical Engineering Superintendent										6 1 6 4 6 5 7 5 9 0 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR						
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input type="checkbox"/> NO		0	8	1	5	8	9			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 24, 1989, with the Reactor Coolant System in Mode 5 (Cold Shutdown), the accumulated leakage found while performing the Type B and C Leak Rate Tests on Containment penetrations exceeded the L.C.O. value (0.60 La) of Technical Specification 3.6.1.2.b.

Those Containment isolation valves that exhibit leak rates in excess of acceptance criteria are being repaired and retested to ensure the leak rates are within allowable limits. The B and C Leak Rate Testing is still in progress and will not be completed until the end of the Refueling Outage.

This is an interim report. The final report will be submitted by August 15, 1989, following completion of the B and C Leak Rate Tests.

8904280115 890421
FDR ADOCK 05000315
S PDC



April 21, 1989

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73
entitled Licensee Event Reporting System, the following
report is being submitted:

89-004-00

Sincerely,

A handwritten signature in dark ink, appearing to read 'W. G. Smith, Jr.' with a stylized flourish at the end.

W. G. Smith, Jr.
Plant Manager

WGS:clw

Attachment

cc: D. H. Williams, Jr.
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