

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8704160300 DOC. DATE: 87/04/09 NOTARIZED: NO DOCKET #  
 FACIL: 50--315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
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 KRIESEL, T. A. Indiana & Michigan Electric Co.  
 SMITH, W. G. Indiana & Michigan Electric Co.  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-002-00: on 870324, noted that sample of steam generator blowdown not taken on 870316. Caused by procedural deficiencies. Sampling procedure modified to require sampler to check for flow through line. W/870409 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4  
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

## NOTES:

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|           | PD3-3 LA                  | 1 1                 |  | PD3-3 PD                  | 1 1                 |
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| INTERNAL: | ACRS MICHELSON            | 1 1                 |  | ACRS MOELLER              | 1 1                 |
|           | ACRS WYLIE                | 1 1                 |  | AEOD/DOA                  | 1 1                 |
|           | AEOD/DSP/ROAB             | 2 2                 |  | AEOD/DSP/TAPK             | 1 1                 |
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|           | NRR/PMAS/ILRH             | 1 1                 |  | NRR/PMAS/PTSK             | 1 1                 |
|           | <u>REG FILE</u> 02        | 1 1                 |  | RES SPEIS, T              | 1 1                 |
|           | RGN3 FILE 01              | 1 1                 |  |                           |                     |
| EXTERNAL: | EG&G GROH, M              | 5 5                 |  | H ST LOBBY WARD           | 1 1                 |
|           | LPDR                      | 1 1                 |  | NRC PDR                   | 1 1                 |
|           | NSIC HARRIS, J            | 1 1                 |  | NSIC MAYS, G              | 1 1                 |

**LICENSEE EVENT REPORT (LER)**

|                                      |                               |                |
|--------------------------------------|-------------------------------|----------------|
| FACILITY NAME (1)                    | DOCKET NUMBER (2)             | PAGE (3)       |
| DONALD C. COOK NUCLEAR PLANT, UNIT 1 | 0   5   0   0   0   3   1   5 | 1   OF   0   3 |

|           |  |
|-----------|--|
| TITLE (4) | NON-REPRESENTATIVE STEAM GENERATOR BLOWDOWN SAMPLE<br>TAKEN DUE TO PROCEDURE DEFICIENCY. |
|-----------|--|

| EVENT DATE (5) |     |      | LER NUMBER (6) |   |                   |   | REPORT DATE (7) |       |     | OTHER FACILITIES INVOLVED (8) |                |   |   |   |   |                  |   |   |   |   |   |  |  |  |  |   |  |  |  |  |   |  |  |  |  |   |  |  |  |  |   |  |  |  |  |
|----------------|-----|------|----------------|---|-------------------|---|-----------------|-------|-----|-------------------------------|----------------|---|---|---|---|------------------|---|---|---|---|---|--|--|--|--|---|--|--|--|--|---|--|--|--|--|---|--|--|--|--|---|--|--|--|--|
| MONTH          | DAY | YEAR | YEAR           |   | SEQUENTIAL NUMBER |   | REVISION NUMBER | MONTH | DAY | YEAR                          | FACILITY NAMES |   |   |   |   | DOCKET NUMBER(S) |   |   |   |   |   |  |  |  |  |   |  |  |  |  |   |  |  |  |  |   |  |  |  |  |   |  |  |  |  |
| 0              | 3   | 1    | 5              | 8 | 7                 | 8 | 7               | —     | 0   | 0                             | 2              | — | 0 | 0 | 0 | 4                | 0 | 9 | 8 | 7 | 0 |  |  |  |  | 5 |  |  |  |  | 0 |  |  |  |  | 0 |  |  |  |  | 0 |  |  |  |  |
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| OPERATING MODE (9) |  | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) |   |                  |  |                      |  |
|--------------------|--|--|---|------------------|--|----------------------|--|
| POWER LEVEL (10)   |  | 20.402(b)  |   | 20.405(c)        |  | 50.73(a)(2)(iv)      | 73.71(b)   |
|                    |  | 20.405(a)(1)(i)  |   | 50.36(c)(1)      |  | 50.73(a)(2)(v)       | 73.71(c)   |
|                    |  | 20.405(a)(1)(ii)   |   | 50.36(c)(2)      |  | 50.73(a)(2)(vii)     | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
|                    |  | 20.405(a)(1)(iii)  | X | 50.73(a)(2)(i)   |  | 50.73(a)(2)(viii)(A) |  |
|                    |  | 20.405(a)(1)(iv)   |   | 50.73(a)(2)(ii)  |  | 50.73(a)(2)(viii)(B) |  |
|                    |  | 20.405(a)(1)(v)  |   | 50.73(a)(2)(iii) |  | 50.73(a)(2)(ix)      |  |

| LICENSEE CONTACT FOR THIS LER (12)                                       |  |  |  |  |  |  |                  |   |   |   |   |   |   |   |   |   |   |
|--|--|--|--|--|--|--|------------------|---|---|---|---|---|---|---|---|---|---|
| NAME   |  |  |  |  |  |  | TELEPHONE NUMBER |   |   |   |   |   |   |   |   |   |   |
| T. A. KRIESEL - TECHNICAL PHYSICAL SCIENCES<br>DEPARTMENT SUPERINTENDENT |  |  |  |  |  |  | AREA CODE        |   |   |   |   |   |   |   |   |   |   |
|  |  |  |  |  |  |  | 6                | 1 | 6 | 4 | 6 | 5 | - | 5 | 9 | 0 | 1 |

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) |        |           |              |                     |  |       |        |           |              |                     |  |
|--|--------|-----------|--------------|---------------------|--|-------|--------|-----------|--------------|---------------------|--|
| CAUSE  | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |  | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |  |
|  |        |           |              |                     |  |       |        |           |              |                     |  |
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|  |  |                                     |       |     |      |
|--|--|-------------------------------------|-------|-----|------|
| SUPPLEMENTAL REPORT EXPECTED (14)  |  | EXPECTED<br>SUBMISSION<br>DATE (15) | MONTH | DAY | YEAR |
| <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) | <input checked="" type="checkbox"/> NO |                                     |       |     |      |

**ABSTRACT** (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 24, 1987, with the reactor at 90 percent power, it was determined that a representative sample of steam generator (S/G) blowdown was not obtained on March 16, 1987. This was the result of a blowdown valve lineup which allowed the normal blowdown sample point to be isolated from the effluent path. This resulted in a sample being taken on a stagnant section of piping.

Samples are required daily for compositing to meet the requirements of 4.11.1.1.1, Table 4.11-1B.

**Procedural Deficiencies are the cause of the event.**

To prevent recurrence the sampling procedure was changed to require the sampler to verify that there is flow in the piping prior to sampling.

8704160300 870409  
PDR ADOCK 05000315  
S PDR

LEDA 4/2

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/88

|  |  |                |                      |                    |          |    |     |
|--|--|----------------|----------------------|--------------------|----------|----|-----|
| FACILITY NAME (1)<br><br>D. C. COOK NUCLEAR PLANT,<br>UNIT 1 | DOCKET NUMBER (2)<br><br>0 5 0 0 0 3 1 5 | LER NUMBER (6) |                      |                    | PAGE (3) |    |     |
|  |  | YEAR           | SEQUENTIAL<br>NUMBER | REVISION<br>NUMBER |          |    |     |
|  |  | 8 7            | — 0 0 2              | — 0 0              | 0 2      | OF | 0 3 |

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Conditions Prior to Occurrence

Unit 1 at 90 percent reactor thermal power.

Description of Event

On March 24, 1987, it was determined that a representative sample of steam generator blowdown was not obtained on March 16, 1987. This deficiency was discovered following the investigation of suspicious suspended solids results of the March 16, 1987 sample analyses. This event was the result of the normal sample point, DSX-300, (EIIS/SMV) being bypassed while returning blowdown to service using the normal flash tank (EIIS/TK) on March 15, 1987 at 1649 hours. The line-up procedure (OHP 4021.025.001) allowed two flow paths while on the normal flash tank. One flow path isolated DSX-300 by closing BD-110. Samples taken while using this configuration would be off a stagnant pipe section and would not be representative of the blowdown being discharged. This was the configuration that was present when the blowdown sample was taken on March 16, 1987.

Technical Specification 4.11.1.1.1 requires that steam generator (EIIS/SG) blowdown (EIIS/WI) be sampled and composited daily for the radionuclide analyses described in Table 4.11-1 Part B.

There were no inoperable structures, components, or systems that contributed to this event.

Cause of Event

Because the blowdown line-up and sampling were done in accordance with approved procedures, it is concluded that procedural deficiencies are the cause of the problem.

Analysis of the Event

The event is reportable under the criteria of 10 CFR 50.73(a)(2)(i).

The blowdown procedure which allowed the lineup had been in effect since February 27, 1987, so conditions could have been present for inaccurate samples since that time. However, only the sample taken after the March 15, 1987 blowdown lineup had irregular results, therefore, it is believed that the only missed sample was on March 16, 1987.

Because the sample taken March 16 was not valid and could not be replaced, the daily composites could not be properly prepared, however, routine daily analyses for gross beta-gamma, tritium, and gamma spectroscopy (including Iodine-131) on individual steam generator blowdown samples indicated no abnormal levels. Furthermore, prior to being discharged, the effluent

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

|  |  |                |                      |                    |          |    |     |
|--|--|----------------|----------------------|--------------------|----------|----|-----|
| FACILITY NAME (1)<br><br>D.C. COOK NUCLEAR PLANT, UNIT 1 | DOCKET NUMBER (2)<br><br>0 5 0 0 0 3 1 5 | LER NUMBER (6) |                      |                    | PAGE (3) |    |     |
|  |  | YEAR           | SEQUENTIAL<br>NUMBER | REVISION<br>NUMBER |          |    |     |
|  |  | 8 7            | 0 0 2                | 0 0                | 0 3      | OF | 0 3 |

TEXT (If more space is required, use additional NRC Form 365A's) (17)

is monitored by radiation monitor R-19 (EIIS/RI). If more severe conditions, i.e., increased radiological contamination of blowdown water occurred that day, radiation monitor R-19 would have isolated the effluent pathway. Based on this information, it is concluded that the event did not pose a threat to the health and safety of the public.

Corrective Actions

At the time of discovery, it was too late to obtain the required sample for compositing. No corrective action was possible.

To prevent recurrence the sampling procedure was modified to require the sampler to check for flow through the line on flow indicator DFI-352 (EIIS/FI) prior to sampling.

Since March 17, all samples of blowdown have been verified to be representative of the effluent.

Failed Component Identification

No component failures were identified during the event.

Previous Similar Events

None.



**INDIANA & MICHIGAN ELECTRIC COMPANY**

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Telephone (616) 465-5901

April 9, 1987

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Operating License DPR-58  
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73  
entitled Licensee Event Reporting System, the following  
report is being submitted:

87-002-0

Sincerely,

W. G. Smith, Jr.  
Plant Manager

/afh

Attachment

cc: John E. Dolan  
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