

INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

July 10, 1986
AEP:NRC:0981

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
NRC INSPECTION REPORT NOS. 50-315/86017 (DRS)
AND 50-316/86017 (DRS)

Mr. James G. Keppler, Regional Director
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Mr. Keppler:

This letter is in response to Mr. W. G. Guldemon's letter dated June 6, 1986, which forwarded the report on the routine safety inspection conducted by members of your staff. This inspection was conducted from April 1 through May 12, 1986 on activities at the D. C. Cook Nuclear Plant Units 1 and 2. The Notice of Violation attached to Mr. Guldemon's letter identified one violation, which is addressed in the attachment to this letter.

As a result of the July 7, 1986 discussion between Mr. P. Barrett (AEPSC) and Mr. B. Burgess (NRC-Region III), the date for submitting this response was extended to July 10, 1986.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

M. P. Alexich

M. P. Alexich
Vice President

RBK
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Attachment

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Bruchmann
G. Charnoff
NRC Resident Inspector - Bridgman

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NRC Violation

"10 CFR Part 50, Appendix B, Criterion V, 'Instructions, Procedures, and Drawings,' as implemented by the Donald C. Cook Updated Quality Assurance Plan, Paragraph 1.7.5, 'Instructions, Procedures and Drawings' requires activities affecting quality to be prescribed by instructions, procedures or drawings appropriate to the circumstances, including appropriate quantitative or qualitative acceptance criteria for determining important activities have been satisfactorily accomplished.

Contrary to the above, the licensee's procedures for verifying correct Essential Service Water (ESW) valve response on receipt of a safety injection signal did not contain appropriate acceptance criteria to show that the minimum flows to the safety-related Component Cooling Water (CCW) heat exchangers, as stated in the Final Safety Analysis Report, were satisfied. The associated ESW valves were merely verified to actuate 'partially open'."

Response to NRC Violation

Corrective Action Taken and Results Achieved

In response to this violation, special test procedures 1-THP-SP.128 (Unit 1) and 2-THP-SP.125 (Unit 2) were written to evaluate and properly position the flow-balancing valves on the Essential Service Water system (ESW) and Component Cooling Water system (CCW).

The flow balance testing was completed in accordance with the above procedures by June 15, 1986 on Unit 1 and June 14, 1986 on Unit 2.

The requirements of Technical Specifications 4.7.4.1.b.2 for Unit 1 and 4.7.4.1.b for Unit 2 have been verified quantitatively by the above actions in conjunction with the current procedures **12-THP-4030-STP.205A and 12-THP-4030.STP.205B.

In addition, an investigation was conducted to identify other safety-related or safety-interface valves which must go to an intermediate position following an actuation. This investigation was completed on May 5, 1986. As a result, additional flow verification testing has been completed with the exception of the Unit 2 auxiliary feedwater flow retention valves. This testing is performed in Mode 1, and thus scheduled to be completed following the Unit 2 startup.

Corrective Action To Be Taken To Avoid Further Violations

The flow balance testing of systems described in the previous paragraph will be repeated on a refueling cycle interval.

Date When Full Compliance Will Be Achieved

The flow balance testing surveillance program will be implemented for the next refueling outages for Units 1 and 2.

The FSAR will be revised in 1987 to reflect the new acceptance criteria provided by Westinghouse.

