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 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
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 RECIP. NAME RECIPIENT AFFILIATION
 DENTON, H. R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Application for amend to License DPR-74, changing Tech Specs
 to support continued full power operation of Cycle 5 reload.
 Reasons & 10CR50.92 Justification provided. Fee paid.

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INDIANA & MICHIGAN ELECTRIC COMPANY

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August 28, 1984

AEP:NRC:0860J

Donald C. Cook Nuclear Plant Unit No. 2
Docket No. 50-316
License No. DPR-74
APPLICATION FOR UNIT 2 TECHNICAL SPECIFICATION
CHANGES FOR CYCLE 5

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

This letter and its Attachments constitute an application for amendment to the Donald C. Cook Nuclear Plant Unit No. 2 Appendix A Technical Specifications (T/S). The amendments are requested to support continued full power operation of the Cycle 5 reload of Unit 2. We anticipate that we will need these T/S issued by November 14, 1984 to support full power operation and allow the T/S to be implemented by the Plant in an orderly manner.

Attachment 1 to this letter addresses the requested T/S change. We have provided our analyses pursuant to 10 CFR 50.92 (Significant Hazards Considerations). The Significant Hazards Considerations analyses are being forwarded pursuant to 10 CFR 50.91(a)(1). In addition, we have further determined that the proposed T/S change will not have an adverse environmental impact.

Attachment 2 to this letter contains revised pages for the requested T/S changes. All changes from the current Unit 2 T/S are indicated by a vertical line on the right hand margin of the page. T/S changes contained in Attachment 2 have been reviewed and approved by the Plant Nuclear Safety Review Committee (PNSRC) and will be reviewed by the AEPSC Nuclear Safety and Design Review Committee (NSDRC) at their next scheduled meeting.

As required by 10 CFR 50.91(b)(1), a copy of this entire application for amendment is being transmitted to the appropriate official of the State of Michigan.

To support this application, Exxon Nuclear Company (ENC) has transmitted reports XN-NF-8L-21(P) and XN-NF-84-21-(NP) under separate cover. These reports, entitled "Donald C. Cook Nuclear Plant Unit 2 Cycle 5 5% Steam Generator Tube Plugging Limiting Break LOCA Analysis" were transmitted by Mr. J. C. Chandler of ENC on July 27, 1984 and August 7, 1984. The Exxon letter numbers for the transmittal of these reports are JCC:106:84 and JCC:110:84.

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The reports present LOCA/ECCS calculations performed to determine core peaking limits which permit operation of Donald C. Cook Nuclear Plant Unit 2 within the guidelines of 10 CFR 50.46 Appendix K and support operation of the Donald C. Cook reactor at a total peak limit (F_Q^T) of 2.04 and a corresponding $F_{\Delta H}^N$ limit of 1.49.

Pursuant to 10 CFR 170.12, we have enclosed a check in the amount of \$150 as payment for the application fee for the proposed amendment.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,



M. P. Alexich
Vice President

Ken
8-27-84

MPA/dew

Attachments

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Charnoff
E. R. Swanson, NRC Resident Inspector - Bridgman

ATTACHMENT 1 to AEP:NRC:0860J

Reasons and 10 CFR 50.92 Justification for

Technical Specification Change for

Donald C. Cook Nuclear Plant

Unit No. 2, Cycle 5

Attachment 1 to AEP:NRC:0860J

The proposed amendment is based on a new LOCA/ECCS analysis performed by our fuel vendor, Exxon Nuclear Company (ENC) using the Z-equivalent methodology. The methodology is described in Exxon document XN-NF-82-20(P), Revision 1, Supplement 4, "Exxon Nuclear Company Evaluation Model EXEM/PWR ECCS Model Updates: Adjustments to FLECHT Based Heat Transfer Correlations", July, 1984. The plant-specific analysis is described in Exxon document XN-NF-84-21(P), Revision 2, "Donald C. Cook Unit 2, Cycle 5 - 5% Steam Generator Tube Plugging Limiting Break LOCA/ECCS Analysis", August 1984.

We have reviewed the two Exxon documents and identified no inadequacies. The analytical methods used to demonstrate conformance to regulations reflect some changes from previous models but are utilized in a manner which we believe will be acceptable to the NRC. Therefore, we believe the proposed changes do not involve a significant hazards considerations as defined by 10 CFR 50.92.

ATTACHMENT 2 to AEP:NRC:0860J

Revised Pages With the
Technical Specification Changes for
Donald C. Cook Nuclear Plant
Unit No. 2, Cycle 5

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