

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8403260002 DOC. DATE: 84/03/20 NOTARIZED: NO DOCKET #  
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
 AUTH. NAME: AUTHOR AFFILIATION  
 ALEXICH, M.P. Indiana & Michigan Electric Co.  
 RECIP. NAME: RECIPIENT AFFILIATION  
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Requests approval for use of encl new peaking factor limit  
 rept. for Cycle 8. Rept presumes target band of greater than  
 or equal to 5% target axial flux difference. Rept withheld.

DISTRIBUTION CODE: PA01S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2410  
 TITLE: Proprietary Review Distribution-Operating Reactor

## NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	NRR ORB1 BC	7 7		
INTERNAL:	ELD/HDS3	1 0	REG FILE	1 1
	RGN3	1 1		
EXTERNAL:	ACRS	6 6	LPDR	1 0
	NRC PDR	1 0	NTIS	1 0

1. The first step in the process of the development of the new system is the identification of the requirements. This is done by the user and the system analyst. The user identifies the requirements by providing a list of requirements. The system analyst identifies the requirements by providing a list of requirements.

1. The first step in the process of identifying a problem is to define the problem. This involves identifying the symptoms of the problem and determining the scope of the problem. Once the problem has been defined, the next step is to identify the causes of the problem. This involves identifying the factors that are contributing to the problem and determining the underlying causes. Once the causes have been identified, the next step is to develop a plan of action. This involves identifying the steps that need to be taken to solve the problem and determining the resources that will be needed to implement the plan. Finally, the last step in the process is to implement the plan and monitor the results. This involves putting the plan into action and tracking the progress of the solution. Once the problem has been solved, the final step is to evaluate the results and determine if the solution was effective. This involves comparing the results of the solution to the original problem and determining if the problem has been solved. If the problem has not been solved, the process may need to be repeated.

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# INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631  
COLUMBUS, OHIO 43216

March 20, 1984

AEP:NRC:0745L

Donald C. Cook Nuclear Plant Unit No. 1  
Docket No. 50-315  
License No. DPR-58

UNIT 1 CYCLE 8  
PEAKING FACTOR LIMIT REPORT UPDATE

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Denton:

By this letter and its attachment, we request your approval for use of a new Peaking Factor Limit Report for Donald C. Cook Unit 1 Cycle 8, as described in Technical Specification 4.2.2.2.c. The V(Z) data in this new Peaking Factor Limit Report are not based on a new calculation. Rather, the existing results from the previous calculation are specified in the form of a V(Z) for each of several burnup steps. By comparison, the previous Peaking Factor Limit Report, submitted in letter AEP:NRC:0745H, dated August 31, 1983, showed one V(Z) function over the whole cycle; incorporating the most limiting values from the individual burnup V(Z)'s. The new Peaking Factor Limit Report presumes a target band of  $\pm 5\%$  about the target axial flux difference.

This use of the more detailed Peaking Factor Limit Report is required, as quickly as possible, for Unit 1 to obtain full power operation. The need to use the burnup dependent V(Z) data was discovered only after some time of Cycle 8 power operation, when we found a slight increase in the  $F_0$  peaking factor above that which had been predicted. The use of burnup dependent V(Z) data has been reviewed by our reload fuel supplier, Westinghouse Electric Corporation, and has been shown not to exceed any safety limit, including those limits specified in 10 CFR 50.46 and 10 CFR 50 Appendix K. On this basis we have concluded that its use will not constitute an unreviewed safety question as defined in 10 CFR 50.59.

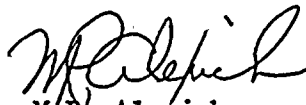
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This document has been prepared following Corporate procedures which incorporated a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,



M.P. Alexich  
Vice President

4/24/81  
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cc: John E. Dolan - Columbus.  
W.G. Smith, Jr. - Bridgman  
R.C. Callen  
G. Charnoff  
E.R. Swanson, NRC Resident Inspector - Bridgman

THE FOLLOWING IS A SUMMARY OF THE RESULTS OF THE  
 ANALYSIS OF THE SAMPLES OF THE  
 SUBSTANCE SUBMITTED FOR ANALYSIS.

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