Central Files

INDIANA & MICHIGAN ELECTRIC COMPANY

P. O. BOX 18
BOWLING GREEN STATION
NEW YORK, N. Y. 10004

RECEIVED DISTRIBUTION SERVICES UNIT
1980 NOV 10 PM 1 03
US NRC DISTRIBUTION SERVICES BRANCH

October 31, 1980
AEP:NRC:00356C

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
IE BULLETIN 79-01B/ENVIRONMENTAL QUALIFICATION OF CLASS 1E EQUIPMENT

Mr. James G. Keppler, Regional Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

This letter has two purposes. The first is to revise our second set of responses to IE Bulletin 79-01B entitled, "Environmental Qualification of Class 1E Equipment" which was submitted on May 7, 1980 (AEP:NRC:00356A) and amended by our letter of June 5, 1980 (AEP:NRC:00356B). In this regard, the revised pages contained in Attachments 1, 2, 3 and 4 to this letter should be incorporated into Attachments 2, 3, 5 and 6, respectively to AEP:NRC:00356A. Please note that this letter and AEP:NRC:00356B contain only revisions to the original, complete submittal. These revisions were generated by our ongoing re-review of the previously submitted material. We have discussed some of these changes with a member of your staff.

The second purpose of this letter is to respond to the Order for Modification of License attached to S. A. Varga's letter to J. E. Dolan dated August 29, 1980 and revised in D. G. Eisenhut's letter to J. E. Dolan dated September 19, 1980; and to comment on Supplements No. 2 and No. 3 to IE Bulletin 79-01B attached respectively to your letters to J. E. Dolan dated September 30, 1980, and October 24, 1980.

A0 2

NOV. 6 1980

8011200005

D

ARCHIVES
INSTITUTE

The basis for the Order for Modification of License was the NRC position that our submittal was incomplete. If the Bulletin guidance prior to the issuance of Supplement No. 2 is used as the criteria, we hold that our submittal was complete as of May 7, 1980. Therefore, our Operating License should not have been modified, and we now respectfully request that the license condition be removed. We believe that Supplements No. 2 and No. 3, having been issued after the date on which the Order for Modification of License became effective, are not part of the Staff's request as described in the previously referenced NRC letter of September 19, 1980.

The issue of the environmental qualification of equipment utilized to bring the reactor to a cold shutdown has yet to be properly resolved in regards to IE Bulletin 79-01B. Supplement 1 to the Bulletin stated (Q.3): 'For equipment that is not environmentally qualified the licensee should either provide plans to qualify this equipment or provide justification that qualification is not needed to achieve safe shutdown to meet licensing requirements applicable to your facility' (emphasis added). Appendix H to the Cook Plant FSAR and our accepted response to Appendix A to Branch Technical Position No. APCSB9.5-1 state in effect that the licensing basis of the Cook Plant is that hot shutdown is safe shutdown. Therefore, our submittals to date have been fully responsive to the Bulletin requirement. Supplement No. 2 differs from Supplement No. 1 in that it directly requests (Q.1 and Q.5) that cold shutdown equipment be environmentally qualified and that such qualification information be supplied by November 1, 1980. The recent Supplement No. 3 states that qualification information regarding equipment needed to achieve a "Hot Safe Shutdown" must be provided by November 1, 1980. This requirement is consistent with Supplement No. 1 and our submittal of May 7, 1980 fully addresses this equipment. Supplement No. 3 states in addition that at least one "path" to cold shutdown must be environmentally qualified and the related qualification information must be supplied to the NRC by February 1, 1981. We take exception to this requirement since we consider that the inclusion of equipment necessary to bring the reactor to cold shutdown represents a change to the licensing basis of the Cook Plant. We do not believe that changing the Plant's licensing basis through Supplements to IE Bulletins is the proper regulatory process.

Supplement No. 2 included TMI Action Plan equipment within the 79-01B review scope; Supplement No. 3 reiterates this position while presenting a more clearly defined schedule for the submittal of the pertinent qualification information. Supplement No. 3 also states that, 'This position represents no change in the staff position regarding the scope of the review.' However, Supplement No. 2 itself states: '...in Bulletin Supplement No. 1, issued on February 29, 1980, the answer to question No. 5 specified that TMI lessons learned equipment was not included in the review.'

Our various submittals in response to this Bulletin and the TMI-related requirements have demonstrated our commitment to the proper environmental qualification of safety-related equipment. In accordance with your guidance, we will supply all available qualification information for installed TMI Action Plan equipment by February 1, 1981. Our response in this area will be in accordance with our previously submitted plans regarding the environmental qualification requirements for TMI-related equipment.

In summary, first we are revising our IE Bulletin 79-01B response for the reasons given at the beginning of this letter. Second, we are requesting that the Order for Modification of License be withdrawn since we find its basis for issuance not applicable to our Plant. Third, we are taking exception to the inclusion of cold shutdown equipment as part of the 79-01B scope and request that you withdraw that requirement from it. Last, we will address the TMI lessons learned items in the manner described above.

Very truly yours,

G. P. Maloney
Vice President

cc: (w/attachment)
N. C. Moseley - NRC
D. V. Shaller - Bridgman
NRC Resident Inspector at Cook Plant - Bridgman
F. Jablonski - NRC Region III

cc: (w/o attachment)
R. C. Callen
G. Charnoff
John E. Dolan
R. W. Jurgensen
R. S. Hunter

STATE OF NEW YORK)
)
COUNTY OF NEW YORK)

G. P. Maloney, being duly sworn, deposes and says he is the Vice President of licensee Indiana and Michigan Electric Company, that he has read the foregoing fourth response to NRC IE Bulletin 79-01B and knows the content thereof, and the said contents are true to the best of his knowledge and belief.

G. P. Maloney

Subscribed and sworn to before me this 31st day of October, 1980.

Notary Public

GREGORY M. GURICAN
Notary Public, State of New York
No. 31-4643431
Qualified in New York County
Commission Expires March 30, 1981

ATTACHMENT 1

TO

AEP:NRC:00356C

Revisions to Attachment 2 to AEP:NRC:00356A

The following revisions should be made to Attachment 2 to AEP:NRC:00356A. Revised pages are attached.

Page	Remark
113 - 114	Qualification Reference CP11 added
214	Qualification Reference for control cable termination at penetration deleted.
215 - 223, 226 - 228	Qualification Reference for control cable terminations at solenoid valve deleted.

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58
SYSTEM Control Air Purifier Sys FAN 1-HV-CEQ1

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
9532 CR-1	Power Cable	X		CP11	
HV-CEQ1	Fan Motor	X		F1	
	Power Cable Term. at Fan Motor	X		TP3	
	Power Cable Term. at Floodup Term Box	X		TP2	
	Power Cable Term. inside floodup tube at penetration.	X		TP1	

Rev 1

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58
SYSTEM Control Air Recirc. System FAN 1-HV-CEQ1

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
9604 CG-1	Power Cable	X		CP11	Rev 1
HV-CEQ2	FAN Motor	X		F1	
	Power Cable Term. at FAN Motor	X		TP3	
	Power Cable Team. at Floodup Team Box	X		TP2	
	Power Cable Term. Inside Floodup Tube at Penetration	✓		TP1	

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM VALVE QCM-250

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
8774 CG-1	POWER CABLE	X		CP7 CP8 CP9	
8775 CG-1	CONTROL CABLE	X		CC7 CC8	
8776 G-1	CONTROL CABLE	X		CC7 CC8	
QCM-250	VALVE MOTOR OPERATOR	X		V5	
	PWR. CABLE TERMINATION AT VALVE MTR. OPERATOR	X		TP3	
	PWR. CABLE TERMINATION AT FLOOD UP TERM. BOX	X		TP2	
	PWR. CABLE TERMINATION inside flood up tube AT PENETRATION	X		TP1	
	control cable term at Valve Lim. SW's	X		TC1	
	control cable term at term. box	X		TC8	
	control cable term at penetration box	X			Rev 1

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No.1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-11

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-012	SOLENOID	X		S17	
9054R-1	CONTROL CABLE	X		CC1	
8791CR-1	CONTROL CABLE	X		CC7 CC8	
					Rev 1
	control cable term at term box	X		TC8	
	control cable term at Pen. term box	X		TC8	

Rev: 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-21

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-021	SOLENOID	x		S17	
9181R-1	CONTROL CABLE	x		CI1 CI2	
9171CR-1	CONTROL CABLE	x		CC1 CC8	
	Control cable term at Pen. Term box	x		TC8	
	control cable term at term box	x		TC8	

1 Rev.

9/2
Rev. 1 1980

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-101

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-121	SOLENOID	X		S17	
8480CG-1	CONTROL CABLE	X		CC7 CC8	
					Rev. 1
	control cable: term at Pen. term box	X		TC8	
	control cable term at term box	X		TC8	

Rev. 1 - 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-102

PLANT ID No.	GENERIC NAME.	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-122	SOLENOID	X		S17	
91956-1	CONTROL CABLE	X		CC1	
91916-1	CONTROL CABLE	X		CC7 CC8	
	c a				Rev. 1
	a.				
	control cable term at Pen. term box	X		TC8	
	control cable term at term box	X		TC8	

Rev. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCP-103

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-123	SOLENOID	X		S17	
8478G-1	CONTROL CABLE	X		CC1	
9196CG-1	CONTROL CABLE	X		CC7 CC8	
					Rw.1
	control cable term at Pen. term box	X		TC8	
	Control cable term at term box	X		TC8	

Rev. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No.1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-104

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO – 124	SOLENOID	X		S17	
8479G-1	CONTROL CABLE	X		CC1	
				-	
				-	
9197CG-1	CONTROL CABLE	X		CC7 CC8	
					Rw. 1
	control cable term at Pen term box	X		TC8	
	control cable term at term box	X		TC8	

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-105

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-125	SOLENOID	×		S17	
9056G-1	CONTROL CABLE	×		CC1	
9052CG-1	CONTROL CABLE	×		CC7 CC8	
	control cable term at Pen term box	×		TC8	
	control cable term at term box	×		TC8	

Rw. 1

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-106

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-126	SOLENOID	X		S17	
9061G-1	CONTROL CABLE	X		CC1	
9057CG-1	CONTROL CABLE	X		CC7 CC8	
					Rw.1
	control/cable term at Rm. term box	X		TC8	
	control cable term at term box	X		TC8	

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM Valve VCR-107

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-127	SOLENOID	X		S17	
9070G-1	CONTROL CABLE	X		CC1	
9066CG-1	CONTROL CABLE	X		CC7 CC8	
	control cable term at Pen. term box	X		TCB	
	control cable term at term box	X		TCB	

Rev.

Rev. 2 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No.1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM _____ VALVE NRV-153

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
NRV-153	SOLENOID	X		S11	
	LIMIT SWITCH	X		LS1	
876G1	CONTROL CABLE	X		cc7 cc8	
877G1	CONTROL CABLE	X		cc5 cc6	
877ZG1	CONTROL CABLE	X		cc5 cc6	
877 3G1	CONTROL CABLE	X		cc5 cc6	
858 8G1	CONTROL CABLE	X		cc1	
					Rw:1
	CONTROL CABLE TERM. A TERM. BOX	X		TC8	
	CONTROL CABLE TERM. A FLOOD UP TERM BOX	X		TC7	
	CONTROL CABLE TERM. A FLOOD UP TUBE	X		TC6	
	control cable term at Lim Sw's	X		LS1	

PAGE 226

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No.1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM _____ VALVE NRV-152

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
NRV-152	SOLENOID	X		S11	
	LIMIT SWITCH	X		LS1	
875921	CONTROL CABLE	X		CC7 CC8	
876021	CONTROL CABLE	X		CC5 CC6	
876121	CONTROL CABLE	X		CC5 CC6	
876221	CONTROL CABLE	X		CC5 CC6	
862621	CONTROL CABLE	X		CC1	
					Rw.1
	CONTROL CABLE TERM A TERM. BOX	X		TC8	
	CONTROL CABLE TERM. A FLOOD UP TERM BOX	X		TC7	
	CONTROL CABLE TERM A FLOOD UP TUBE	X		TC6	
	control cable term at Lim. Sw's	X		LS1	

Rw.1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No.1
DOCKET No. 50-315, LICENSE No. DPR-58

SYSTEM _____ VALVE NRV-151

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
NRV-151	SOLENOID	×		S11	
	LIMIT SWITCH	×		LS1	
875421	CONTROL CABLE	×		CC5 CC6	
875521	CONTROL CABLE	×		CC5 CC6	
875621	CONTROL CABLE	×		CC5 CC6	
875721	CONTROL CABLE	×		CC5 CC6	
857721	CONTROL CABLE	×		CC1	
	CON.				Rw.1
	CONTROL CABLE TERM. AT TERM. BOX	×		TC8	
	CONTROL CABLE TER. AT FLOOD UP TERM. BOX	×		TC7	
	CONTROL CABLE TERM. AT FLOOD UP TUBE	×		TC6	
	control cable term at Lim. SW's	×		LS1	

Rw. 1. 9/2/80

ATTACHMENT 2

TO

AEP:NRC:00356C

Revisions to Attachment 3 to AEP:NRC:00356A

The following revisions should be made to Attachment 3 to AEP:NRC:00356A. Revised pages are also attached.

Page	Remark
218 - 226, 232 - 234	Qualification Reference for control cable terminations at solenoid valve deleted.

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM VCR-11 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
X50-12	SOLENOID	X		S17	
8491CR-2	CONTROL CABLE	X		CC7 CC8	
9054R-2	CONTROL CABLE	X		CC1	
					Rev 1
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rev. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM VCR-21 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-21	SOLENOID	X		S17	
917/CR-2	CONTROL CABLE	X		CC7 CC8	
9181R-2	CONTROL CABLE	X		CC1	
				b	Rev. 1
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rev. 1 9/1/82

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM VCR-101 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-121	SOLENOID	X		S17	
8480CG-2	CONTROL CABLE	X		CC7 CC8	
8484G-2	CONTROL CABLE	X		CC1	
					Rev. 1
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rev. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM _____ VCR-102 VALVE ___

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-122	SOLENOID	X		S17	
9191CG-2	CONTROL CABLE	X		CC7 CC8	
9195G-2	CONTROL CABLE	X		CC7 CC8	
	AT				Rw.
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rw. 1 9/2/FD

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM VCR-103 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-124	SOLENOID	X		S17	
9196CG-2	CONTROL CABLE	X		CC7 CC8	
8478G-2	CONTROL CABLE	X		CC1	
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rw.

Rw. 1 9/2/10

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM _____, VCR-104 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-123	SOLENOID	X		S17	
9197CG-2	CONTROL CABLE	X		CC7 CC8	
8479 G-2	CONTROL CABLE	X		CC1	
					Rev. 1
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT BLEED UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT BLEED UP TUBE	X		TC6	

PAGE 223

Rev. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM _____ V.C.R.-105. VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
YSO-125	SOLENOID	X		517	
9052CG-2	CONTROL CABLE	X		CC7 CC8	
9056G-2	CONTROL CABLE	X		CC1	
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rev. 1

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM VCR-106 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-126	SOLENOID	X		S17	
9057CG-2	CONTROL CABLE	X		CC7 CC8	
9061G-2	CONTROL CABLE	X		CC1	
	CONT. CABLE TERM AT TERM. BOX	X		TC8	
	CONT. CABLE TERM AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM. AT FLOOD UP TUBE	X		TC6	

Rw.1

Rw. 1 9/2/5

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM _____ VCR-107 VALVE

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
XSO-127	SOLENOID	X		S17	
9066CG-2	CONTROL CABLE	X		CC7 CC8	
9070G-2	CONTROL CABLE	X		CC1	
					Rw. 1
	CONT CABLE TERM AT TERM. BOX.	X		TC8	
	CONT. CABLE TERM. AT FLOOD UP TERM BOX	X		TC7	
	CONT. CABLE TERM AT FLOOD UP TUBE	X		TC6	

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM VALVE NRV 151

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
NRV-151	VALVE SOL.	X		S11	
8755R-2	CONTROL CABLE	X		CC1	
8543 R-2	CONTROL CABLE	X		CC2 CC3 CC4	
9705 CR-2	CONTROL CABLE	X		CC7 CC8	
	CONTROL CABLE TERM AT VALVE LIM. SW's.	X		LS1	
	CONTROL CABLE TERM. AT TERM. BOX	X		TCB	
	CONTROL CABLE TERM AT FLOOD UP BOX	X		TC7	
	CONTROL CABLE TERM. AT FLOOD UP TUBE	X		TC6	
					Rw.
	Limit Switch	X		LS1	

PAGE 1 of 2

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM _____ VALVE NRV-152

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
NRV-152	VALVE SOL	X		S11	
8544R-2	CONTROL CABLE	X		CC2 CC3 CC4	
8760R-2	CONTROL CABLE	X		CC5 CC6	
9706CR-2	CONTROL CABLE	X		CC7 CC8	
	CONTROL CABLE TERM. AT VALVE LIM. SW's	X		LS1	
	CONTROL CABLE TERM. AT TERM. BOX	X		TC8	
	CONTROL CABLE TERM. AT FLOOD UP BOX	X		TC7	
	CONTROL CABLE TERM. AT FLOODUP TUBE	X		TC6	
					Rw.1
	Limit Switch	X		LS1	

PAGE 235

Rw. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT No. 2
DOCKET No. 50-316, LICENSE No. DPR-74

SYSTEM _____ VALVE NRV153

PLANT ID No.	GENERIC NAME	LOCATION		EQUIP QUAL. CHART REF.	
		INSIDE CONT.	OUTSIDE CONT.		
NRV-153	VALVE SOL.	X		S11	
8599 G-2	CONTROL CABLE	X		cc2 cc3 cc4	
8771 G-2	CONTROL CABLE	X		cc5 cc6	
8757CG2	CONTROL CABLE	X		cc7 cc8	
	CONTROL CABLE TERM. AT VALVE LIM. SW's	X		LS1	
	CONTROL CABLE TERM. AT TERM. BOX	X		TC8	
	CONTROL CABLE TERM. AT FLOOD UP BOX	X		TC7	
	CONTROL CABLE TERM. AT FLOODUP TUBE	X		TC6	
					Ew.
	Limit Switch			LS1	

Rev 1 9/2/80

ATTACHMENT 3

TO

AEP:NRC:00356C

Revisions to Attachment 5 to AEP:NRC:00356A

The following revisions should be made to Attachment 5 to AEP:NRC:00356A. Revised pages are also attached.

Page	Remark
CC7-1	Operating time qualification added
CC12-1,2	Pages removed
CP11-1,2	New pages
F1-1	Operating time qualification added
LS1-1	Operating time qualification added. Note deleted.
M1-1, M2	Operating time qualification added
TC14-1	Radiation qualification method added
TC16-1,2	Pages removed
TI1-1, TI1-2, TI4-1	Submergence qualification reference added
TP2-1	Model number revised
V8-1, V10-1	Radiation qualification revised

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 | DOCKET NO. 50-315 | LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: VARIOUS COMPONENT: CONTROL CABLE MANUFACTURER: CONTINENTAL MODEL NUMBER: ITEM # 3122 FUNCTION: VARIOUS ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: IN & OUT OF CONTAINMENT	Operating Time	1 YEAR	See Note 1	FSAR Table 7.5-2		See Note 1	
	Temperature (°F)	Fig 022.9-1,-2	345	FSAR App Q	8	SEQ.	
	Pressure (PSIA)	Fig 2 Fig 1	121.7	Aero 6504	8	SEQ.	
	Relative Humidity (%)	100	100		8	SEQ.	
	Chemical Spray	2000 ppmB	1500 ppmB	T.S. 3/4.5.5 3/4.5.6	8	SEQ.	
	Radiation (10^6 rads)	Fig 4 150	150	WCAP 7410-L Vol	8	SEQ.	
	Aging (years)		See Note 2				
FLOOD LEVEL ELEV: 614 ABOVE FLOOD LEVEL: No	Submergence		FLOODUP Tubes *				

*Documentation References:

8. CONAX CORP. TEST REPORT IPS-348

Notes:

* EXCEPT for CABLES ON VCR-11, 21, 101, 102, 103, 104, 105, 106 & 107 AND NMO-151, 152 & 153. See Cable No.( ).

1) Containment temp 2.78 hrs after accident = 185°F (Fig 3, App N, FSAR), cable temp rating 194°F (90°C)

2) XLPE/NSA braid 40 yrs @ 60 per Table C-1 3, pg C, Enclosure 4 to NRC IE Bulletin 79-01B,

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: VARIOUS COMPONENT: Power Cable MANUFACTURER: KERITE MODEL NUMBER: Item # 3127 FUNCTION: VARIOUS ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: INSIDE Containment	Operating Time	1 YEAR	Note 2	FSAR TABLE 7.1-2	7	SEQ	
	Temperature (°F)	FIG 0-27	325	FSAR APP O	7	SEQ.	
	Pressure (PSIA)	FIG 0-27	96.7	FSAR APP O	7	SEQ.	
	Relative Humidity (%)	NA	100	NA	7	SEQ.	
	Chemical Spray	NA	2600 ppm B	NA	7	SEQ.	
	Radiation (10^6 rads)	17	200	See Note 3	See note 1		
	Aging (years)		No Details Yes		See note 1		
FLOOD LEVEL ELEV: 641' ABOVE FLOOD LEVEL: No	Submergence		INSIDE FLOODUP				

*Documentation References:

7. Kerite Co. Report on the Effects of Gamma Rad. and Autoclaving on Kerite Power + Control Cable.

Notes: 1. Letter from Robert Henry (Kerite) to A. Casso (AEP) of 9-18-80

2. Cable Temp rating 194°F. 230°F for 10 sec and 11.5 psig for 1 sec does not represent a challenge to the mechanical elect. quality of the cable

3. AEPSC NS+L CALCULATION DC-N-6120-2.

Rev 0 9/2/80

7. Qualified by Kerite Co. Report on the effects of Gamma Radiation and Autoclaving on Kerite Nuclear Control Cables
April 30, 1970.

Type of Test: Sequential, gamma radiation
steam
chemical spray

Test Profile:

.8 Mrads/hr, 120 Mrads
325°F, 82 psig for 13 hrs
228°F, 5 psig for 7 days

Chemical Spray: Borated water, 1-1/2% solution of boric acid and distilled water buffered at a PH of 9.5

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: HV-CEQ-1 HV-CEQ-2 COMPONENT: FAN Motors MANUFACTURER: WESTINGHOUSE CORP. MODEL NUMBER: TBDP FUNCTION: CIRCULATE AIR ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: INSIDE CONTAINMENT	Operating Time	1/YEAR	Greater than one year	Table 7.5-2	21	Seq	
	Temperature (°F)	F16 13.13-1	320	FSAR App N	21	Seq.	
	Pressure (PSIA)	F.G. 2 Fig 1	89.7	AEW 6504	21	Seq.	
	Relative Humidity (%)	100	100		21	Seq.	
	Chemical Spray	2000 ppm B	2500 ppm B	T.S. 3/4.5 3/4.5.6	21	Seq.	
	Radiation (10^6 rads)	150	200	WCAP 7410-L Vol I	21	Seq.	
	Aging (years)		200°C / 5 hrs Yes		21	Seq.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: YES	Submergence	NA	NA		N.A	NA	

*Documentation References:

21. Westinghouse Corp. Test Report WCAP-7829.

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 | DOCKET NO. 50-315 | LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: Reactor Coolant PLANT ID NO: Same as Valve Served COMPONENT: Limit Switch MANUFACTURER: NAMCO MODEL NUMBER: EA 180 FUNCTION: Valve position Indication ACCURACY: SPEC: NA DEMON: NA SERVICE: Pzr PoRV's NRV-151,-152,-153 LOCATION: IN Containment	Operating Time	Not taken credit for on design basis accident	>1 year		43	SEQ.	
	Temperature (°F)	Fig 022.9-1,-2	340	FSAR App Q	43	SEQ.	
	Pressure (PSIA)	Fig 1 Fig 2	84.7	AEW 6504	43	SEQ.	
	Relative Humidity (%)	100	100		43	SEQ.	
	Chemical Spray	2000 ppm B	3000 ppm B	T.S. 3/4.5 3/4.5.6	43	SEQ.	
	Radiation (10^6 rads)	NA	204	WCAP 7410-L VOL.1	43	SEQ.	
	Aging (years)		200hrs/200°F Yes		43	SEQ.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: yes	Submergence	NA	NA	NA	NA		

*Documentation References:

43. Acme-Cleveland Development Co. Qual. of NAMCO Controls Limit Switch Sept. 5, 1978

Notes:

Rev. 1

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: CVCS, SI, RHR PLANT ID NO: PP-050, 026, 035 COMPONENT: Pump Motor MANUFACTURER: WESTINGHOUSE MODEL NUMBER: 5808Z, 5009H, 5009-T24 FUNCTION: Emergency Core Cooling ACCURACY: SPEC: NA DEMON: NA SERVICE: Centrifugal Charging, Safety Injection & Residual Heat Removal Pumps LOCATION: Outside Containment	Operating Time	1 YEAR	Greater than one year		See Note B.		
	Temperature (°F)	NA	NA	NA	NA	NA	
	Pressure (PSIA)	NA	NA	NA	NA	NA	
	Relative Humidity (%)	NA	NA	NA	NA	NA	
	Chemical Spray	NA	NA	NA	NA	NA	
	Radiation (10^6 rads)	16.6	200	See Note A	See Note B	Similarity to TESTED Equipment	
	Aging (years)		200°C/5oohrs YES		See note B	TEST	
FLOOD LEVEL ELEV: NA ABOVE FLOOD LEVEL: NA	Submergence	NA	NA	NA	NA	NA	

*Documentation References:

Notes:
A) AEPSC NSRL calculation DC-N-6520-2.
B) Westinghouse Test Report WCAP 7829.
B) letter of LF Caso (AEP) to F. Noon (WECo) of 3-20-80.
letter of F. Noon (WECo) to LF Caso (AEP) of 4-21-80.

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: Containment Spray PLANT ID NO: PP-009 COMPONENT: Pump Motor MANUFACTURER: Reliance MODEL NUMBER: Frame # 5810 P FUNCTION: CT Spray ACCURACY: SPEC: NA DEMON: NA SERVICE: Containment Spray Pump LOCATION: Outside CT	Operating Time	1 year	Greater than one year				
	Temperature (°F)	NA	NA	NA	NA		
	Pressure (PSIA)	NA	NA	NA	NA		
	Relative Humidity (%)	NA	NA	NA	NA		
	Chemical Spray	NA	NA	NA	NA		
	Radiation (10^6 rads)	17	100	AEPSC NSLL calc. DC-N-6420-2	see note below	similarity to tested materials	see note below
	Aging (years)						
FLOOD LEVEL ELEV: ABOVE FLOOD LEVEL: NA	Submergence	NA	NA	NA	NA		

*Documentation References:

Notes: Information received by telephone from manufacturer. Letter of confirmation expected.

ncl

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 | DOCKET NO. 50-315 | LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: RHR PLANT ID NO: ICM-303,-306, COMPONENT: Control Cable Termination MANUFACTURER: NA MODEL NUMBER: Cable Term at Valve FUNCTION: Long term post-accident cooling ACCURACY: SPEC: NA DEMON: NA SERVICE: RECIRCULATION LOCATION: In Containment	Operating Time	1 OAY	See Note 1 on Cable Terminat.	Note A below	22	Simul.	
	Temperature (°F)	Fig 0822.9-1,-2	330	FSAR App Q	22	Simul	
	Pressure (PSIA)	Fig 1 Fig 2	104.7	AEW 650Y	22	Simul.	
	Relative Humidity (%)	100	100		22	Simul.	
	Chemical Spray	NA	2600 ppm B	INSIDE CT EXTERNAL	22	Simul.	
	Radiation (10^6 rads)	<4.6	100	ALP NSIL CALCULATION DI N-(44)N-2	1	Seq 1	
	Aging (years)		180°c / 1 mbr / Yes		22	Simul.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: No	Submergence		Floodup Tubes				

*Documentation References:

22. Limitorque Corp Test Report # 600198.

1. Conax Corp. Test Report IPS-234.

Notes: (A) Letter from T. Tilling (AEP) to K. Kniel (NRC) dated 4-14-75 & 9-29-75

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: N/A COMPONENT: INSTRUMENTATION TERMINATION MANUFACTURER: N/A MODEL NUMBER: BARTON INSTRUMENT TERMINATION FUNCTION: ACCURACY: SPEC: N/A DEMON: N/A SERVICE: VARIOUS LOCATION: In Containment	Operating Time	1 YEAR	See Note 1 on Cable Terminat	Table 7.5-2	8,9,13	11,12,14 Separate	
	Temperature (°F)	Fig 022.9-1,-2	340	FSAR App Q	8,9,10,	11,12,14 Separate	
	Pressure (PSIA)	Fig 2 Fig 1	119.7	AEW 6504	8,9,	10,11,12,14 Separate	
	Relative Humidity (%)	100	100		8,9,	10,11,12,14 Separate	
	Chemical Spray	2000 ppm B	2000 ppm B	T.S. 3/4.5 3/4.5.6	8,9,	10,11,12,14 Separate	
	Radiation (10^6 rads)	150	150	WCAP 7410-L Vol 1	8,9,	10,11,12,14 Separate	
	Aging (years)		250°F, 7 days Yes		8,9,	10,11,12,14 Separate	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: No	Submergence		Yes		8,9,	10,11,12,14;17,18,19 Separate	

*Documentation References:

8. Conax Corp. Test Report IPS-348
9. FIRL Test Report F-C4033-1
10. FIRL Test Report F-C3683
11. Isomedix Corp. Test Report of May 1976
12. Cerro Wire + Cable Test Report of May 1976
14. FIRL Test Report F-C4033-3
18. Conax Corp Test Report IPS-327
19. " " " " IPS-329
17. " " " " IPS-326

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 | DOCKET NO. 50-315 | LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: N/A COMPONENT: RTD TERMINATIONS MANUFACTURER: N/A MODEL NUMBER: RTD TERMINATIONS FUNCTION: ACCURACY: SPEC: N/A DEMON: N/A SERVICE: VARIOUS LOCATION: Inside Containment	Operating Time	1 year	See Note 1 on Cable Terminations	Table 7.5-2	8, 9,	10, 11, 12, 14 Separate	
	Temperature (°F)	FIG 022.9-1,-2	340	FSAR App Q	8, 9,	10, 11, 12, 14 Separate	
	Pressure (PSIA)	FIG 1 FIG 2	119.7	AEW 6504	8, 9,	10, 11, 12, 14 Separate	
	Relative Humidity (%)	100	100		8, 9	10, 11, 12, 14 Separate	
	Chemical Spray	2000 ppm B	2000 ppm B	T.S. 3/4.5 3/4.5.6	8, 9,	10, 11, 12, 14 Separate	
	Radiation (10^6 rads)	FIG 4	150	WCAP 7410-L Vol 1	8, 9,	10, 11, 12, 14 Separate	
	Aging (years)		250°F, 7 days Yes		8, 9,	10, 11, 12, 14 Separate	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: NO	Submergence		Yes		8, 9,	10, 11, 12, 14, 17, 18, 19 Separate	19

*Documentation References:

8. Conax Corp Test Report IPS-348
9. FIRL Test Report F-C4033-1
10. FIRL Test Report F-C3683
11. Isomedix Corp Test Report of MAY 1976
12. Cerro Wire + Cable Test Report of MAY 1976
14. FIRL Test Report F-C4033-3
17. Conax Corp Test Report IPS-326
18. " " " " IPS-327
19. " " " " IPS-329

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: N/A COMPONENT: INSTRUMENT CABLE TERMINATION MANUFACTURER: N/A MODEL NUMBER: INSTRUMENT CABLE SPLICE . AT PENETRATION FUNCTION: VARIOUS ACCURACY: SPEC: N/A DEMON: N/A SERVICE: VARIOUS LOCATION: In Containment	Operating Time	1 YEAR	See Note 1 on Cable Terminal	Table 7.5-2	8,9,19	11,12,14,18,19 Separate	
	Temperature (°F)	FIG 022.9-1,-2	340	FSAR APP Q	8,9,10,	11,12,14,18,19 Separate	
	Pressure (PSIA)	FIG 1 FIG 2	119.7	AEW 6504	8,9,10,	11,12,14,18,19 Separate	
	Relative Humidity (%)	100	100		8,9,10,	11,12,14,18,19 Separate	
	Chemical Spray	2000 ppmB	2000 ppmB	T.S. 3/4.5 3/4.5.6	8,9,10,	11,12,14,18,19 Separate	
	Radiation (10^6 rads)	150	150	WCAP 7410-L Vol 1	8,9,10,	11,12,14,18,19 Separate	
	Aging (years)	-	250F 7 days Yes		8,9,10,	11,12,14,18,19 Separate	
FLOOD LEVEL ELEV: 614 ABOVE FLOOD LEVEL: No	Submergence		Yes		8,9,10,	11,12,14,18,19 Separate	

*Documentation References:

8. Conax Corp Test Report IPS-348
9. FIRL Test Report F-C 4033-1
10. FIRL Test Report F-C 3683
11. Isomedix Corp Test Report of May 1976
12. Cerro wire + Cable Test Report of May 1976
14. FIRL Test Report F-C 4033-3
18. Conax Corp. Test Report IPS-327
19. Conax Corp. Test Report IPS-329
17. " " " " IPS-326

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: NA COMPONENT: TERMINATION MANUFACTURER: NA MODEL NUMBER: STRANDED KAPTON SPLICED TO STRANDED HYPALON FUNCTION: CABLE CONNECTION ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: IN CONTAINMENT	Operating Time	1 year	See Note 1 on Cable Term.	Table 7.5-2	13	Seq.	
	Temperature (°F)	FIG 022.9-1,-2	840	FSAR App 9	13	Seq.	
	Pressure (PSIA)	FIG 2 FIG 1	118	AEW 6504	13	Seq.	
	Relative Humidity (%)	100	100		13	Seq.	
	Chemical Spray	Not Req'd	2500 PPM B	T.S. 3/4.5 3/4.5.6	13	Seq.	
	Radiation (10^6 rads)	150	150	WCAP 7410-L Vol 1	13	Seq.	
	Aging (years)						
FLOOD LEVEL ELEV: 614 ABOVE FLOOD LEVEL: yes	Submergence	NA	NA	NA	NA	NA	

Rev. 1

*Documentation References:

13. Westinghouse - Canada Test Report CWAPD-382

Notes:

Rev. 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: [illegible] PLANT ID NO: VARIOUS COMPONENT: VALVE MOTOR OPERATOR MANUFACTURER: LIMITORQUE MODEL NUMBER: VARIOUS FUNCTION: VARIOUS ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: OUTSIDE CONTAIN.	Operating Time	1 day	28 hr	NSIL A BELOW	44	SEQ	
	Temperature (°F)	Fig. 0.17	212.	CSAR APP. O	44	SEQ.	
	Pressure (PSIA)	Fig 0.17	14.7	CSAR APP O	44	SEQ	
	Relative Humidity (%)	NA	100		44	SEQ.	
	Chemical Spray	NA	NA	NA	NA		
	Radiation (10^6 rads)	4.1	See VALVE MOTOR OPERATOR Note 1.	NEP NSL CALC DC-M 6125-2		See VALVE MOTOR OPERATOR NOTE 1	Rev. 1
	Aging (years)						
FLOOD LEVEL ELEV: NA ABOVE FLOOD LEVEL: NA	Submergence	NA	NA	NA	NA	NA	

*Documentation References:

44. FIRL TEST REPORT F-C5271

Notes:

(1) Letters from J. Tillinghast (NEP) to K. Kniel (NRC) dated 4/11/75 & 9/29/75.

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1 DOCKET NO. 50-315 LICENSE NO. DPR-58

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: Residual Heat Removal PLANT ID NO: ICM-305, 306 COMPONENT: Valve Motor Operator MANUFACTURER: Limitorque MODEL NUMBER: SMB-2 FUNCTION: Long term post accident cooling ACCURACY: SPEC: NA DEMON: NA SERVICE: RHR suction from containment sump LOCATION: Inside Containment Extension	Operating Time	1 day	6 days	Note A Below	22	Simul.	
	Temperature (°F)	Fig 022.9-1,2	330	FSAR NRP Q	22	Simul.	
	Pressure (PSIA)	Fig. 1 Fig. 2.	104.7	NEW 6501+	22	Simul.	
	Relative Humidity (%)	100	100		22	Simul.	
	Chemical Spray	NA	2600 ppm B	Inside of Cont.	22	Simul.	
	Radiation (10^6 rads)	<4.6	See Note B	Accident Conditions	See Note B	SEE NOTE B	
	Aging (years)		130°C/10 hr. Yes		22	Simul.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: YES	Submergence	NA	NA	NA	NA	NA	

*Documentation References:

22. Limitorque Corp. Test Report #600198

Notes:

(A) Letters of J. Tillinghast to K. Kniel (NRC) dated 4/1/75 & 7/23/75

(B) These are Westinghouse supplied valves, manufactured class 1, specified for nuclear service inside containment. Limit switch material for these valves is white melamine (radiation resistant material) On the basis of engineering judgement these valve motoroperators is qualified for the expected exposure. We are seeking further confirmatory data to strengthen this judgement.

Rev. 1 9/2/80

ATTACHMENT 4

TO

AEP:NRC:00356C

Revisions to Attachment 6 to AEP:NRC:00356A

The following revisions should be made to Attachment 6 to AEP:NRC:00356A. Revised pages are also attached.

Page	Remark
CP4-1, CP5-1, CP6-1	Submergence qualification added
F1-1	Operating time qualification added
M1-1, M2	Operating time qualification added
TC14-1	Radiation qualification and reference added
TC16-1,2	Pages removed
TI1-1, TI2-1, TI4-1	Submergence qualification reference added
V8-1, V10-1	Radiation qualification revised

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: VARIOUS COMPONENT: Power Cable MANUFACTURER: Cyprus MODEL NUMBER: Item 347 FUNCTION: VARIOUS ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: In Containment	Operating Time	1 YEAR	See Note 1	FSAR Table 7.5-2	35	SEQ	
	Temperature (°F)	Fig 13.13-1	303	FSAR App N	35	SEQ.	
	Pressure (PSIA)	Fig 1 Fig 2	80.7	AEW 6504	35	SEQ.	
	Relative Humidity (%)	100	100		35	SEQ.	
	Chemical Spray	2000 ppm B	2000 ppm B	T.S 3/4.5 3/4.5.6	35	SEQ.	
	Radiation (10^6 rads)	Fig 4 150	300	WCAP 7410-L Vol 1	35	SEQ.	
	Aging (years)		IEEE 383-1974 Para 2.3.3				
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: No	Submergence		Flood up Tubes				

*Documentation References:

35. FIRL TEST REPORT FC 3016

Notes:

1) Containment temp rating 2.7 hrs after accident = 185°F (Fig 3, App N, FSAR). Cable temp rating 199°F

Rev 1 7/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: VARIOUS COMPONENT: Power CABLE MANUFACTURER: ANACONDA WIRE + CABLE Co MODEL NUMBER: Item # 347 FUNCTION: VARIOUS ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: IN AND OUT OF CONTAINMENT	Operating Time	1 YEAR	See Note 1	7.5-2 TABLE FSAR	5	Simul	
	Temperature (°F)	FIG 022.9-1,-2 328.2 PEAK	340	FSAR APP Q.	5	Simul.	
	Pressure (PSIA)	FIG 1 FIG. 2	119.7	AEW C504	6	Simul.	
	Relative Humidity (%)	100	100		5	Simul.	
	Chemical Spray	2000 ppmB	3000 ppmB	T.S. 3/4.5 3/4.5.6	5	Simul.	
	Radiation (10^6 rads)	FIG 4 150	200	WCAP 7410-L VOL 1	5	Simul.	
	Aging (years)		Yes 250°F LONG TERM 7 DAYS		5	Simul.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: No	Submergence		FLOODUP Tubes				

*Documentation References:

5. FIRL TEST REPORT F-C 3341

Notes:

1) containment temp 2.7 hrs after accident = 185°F (Fig 3, App N, FSAR), cable temp rating 194°F

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: VARIOUS COMPONENT: POWER CABLE MANUFACTURER: OKONITE MODEL NUMBER: ITEM # 399 FUNCTION: VARIOUS ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: IN AND OUT of CONTAINMENT	Operating Time	1 year	See Note 1	FSAR Table 7.5-2	6	Simul.	
	Temperature (°F)	FIG 22.9-1,-2 328.2 PEAK	346	FSAR APP Q	6	Simul.	
	Pressure (PSIA)	FIG 1 FIG. 2	127.7	AEC 6504	6	Simul.	
	Relative Humidity (%)	100	100		6	Simul.	
	Chemical Spray	2000 PPM B	2000 PPM B	T.S 3/4.5 3/4.5.6	6	Simul.	
	Radiation (10^6 rads)	FIG 4 150	200	WCAP 7410-L Vol 1	6	Simul.	
	Aging (years)		240°F/7 DAYS Yes				
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: NO	Submergence		FLOOD UP Tubes				

*Documentation References:

6. FIRL Test Report F-C 3694

Notes:

1) Containment temp 2.78 hrs after accident = 185°F (Fig 3, App N, FSAR). Cable temp rating = 194°F

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: HV-CEQ-1 HV-CEQ-2 COMPONENT: FAN MOTORS MANUFACTURER: WESTINGHOUSE CORP. MODEL NUMBER: TBDP FUNCTION: CIRCULATE AIR ACCURACY: SPEC: NA DEMON: NA SERVICE: VARIOUS LOCATION: INSIDE CONTAINMENT	Operating Time	1 YEAR	Greater than one year	Table 7.5-2	21	Seq.	
	Temperature (°F)	Fig 13.13-1	320	FSAR App N	21	SE 8	
	Pressure (PSIA)	Fig 1 Fig 2	89.7	Amd 6604	21	SE 8	
	Relative Humidity (%)	100	100		21	SE 8	
	Chemical Spray	2000 ppm B	2500 ppm B	T.S. 3/4.5 3/4.5.6	21	SE 8	
	Radiation (10^6 rads)	150	200	WCAP 7410-L Vol 1	21	SE 8	
	Aging (years)		200°C/500hrs Yes		21	Seq	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: YES	Submergence	NA	NA		NA	NA	

*Documentation References:

Notes:

21. WESTINGHOUSE CORP. TEST REPORT WCAP-7829

Rev 1 7/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: CVCS, SI, RHR PLANT ID NO: PP-050,026, 035 COMPONENT: Pump Motor MANUFACTURER: WESTINGHOUSE MODEL NUMBER: 5808Z, 5009H, 5009-P24. FUNCTION: Emergency Core Cooling ACCURACY: SPEC: NA DEMON: NA SERVICE: Centrifugal Charging, Safety Injection & Residual Heat Removal Pumps LOCATION: Outside Containment	Operating Time	1 YEAR	greater than one year		See Note B.		
	Temperature (°F)	NA	NA	NA	NA	NA	
	Pressure (PSIA)	NA	NA	NA	NA	NA	
	Relative Humidity (%)	NA	NA	NA	NA	NA	
	Chemical Spray	NA	NA	NA	NA	NA	
	Radiation (10^6 rads)	16.6	200	See Note A	See Note B	Similarity to TESTED Equipment	
	Aging (years)		200°C/500hrs YES		See note B	TEST	
FLOOD LEVEL ELEV: NA ABOVE FLOOD LEVEL: NA	Submergence	NA	NA	NA	NA	NA	

*Documentation References:

Notes:

A) AEPSC NSRL calculation DC-N-6520-2.

B) Westinghouse Test Report WCAP 7829.
letter of LF Caso (AEP) to F. Noon (WECo) of 3-20-80.
letter of F. Noon (WECo) to LF Caso (AEP) of 4-21-80.

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: Containment Spray PLANT ID NO: PP-009 COMPONENT: Pump Motor MANUFACTURER: Reliance MODEL NUMBER: frame # 5810 P FUNCTION: CT Spray ACCURACY: SPEC: NA DEMON: NA SERVICE: Containment Spray Pump LOCATION: Outside CT	Operating Time	1 year	Greater then one year				
	Temperature (°F)	NA	NA	NA	NA		
	Pressure (PSIA)	NA	NA	NA	NA		
	Relative Humidity (%)	NA	NA	NA	NA		
	Chemical Spray	NA	NA	NA	NA		
	Radiation (10^6 rads)	17	100	AEPSC NSEL calc, DC-N-6420-2	see note below	similarity to tested materials	see note below
	Aging (years)						
FLOOD LEVEL ELEV: NA ABOVE FLOOD LEVEL:	Submergence	NA	NA	NA	NA		

*Documentation References:

Notes: Information received by telephone from manufacturer. Letter of confirmation expected.

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: RHR PLANT ID NO: ICM-305, 306 COMPONENT: Control Cable Termination MANUFACTURER: NA MODEL NUMBER: Cable Term. at valves. FUNCTION: long term post accident cooling ACCURACY: SPEC: NA DEMON: NA SERVICE: Recirculation switchover to sump suction LOCATION: In Containment	Operating Time	1 DAY	See Note 1 on Cable Term.	See Note (A)	22	Simul.	
	Temperature (°F)	Fig 022.9-1,-2	330	FSAR Appq	22	Simul.	
	Pressure (PSIA)	Fig 1 Fig 2	104.7	AEW 6504	22	Simul.	
	Relative Humidity (%)	100	100		22	Simul.	
	Chemical Spray	NA	2600 ppmB	inside CT ext.	22	Simul.	
	Radiation (10^6 rads)	4.6	100	AEP NCTL CALCUL. DC-N-6420-2	1	SEQ	
	Aging (years)		180°C/100hrs Yes		22	Simul.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: No	Submergence		Floodup Tubes				

*Documentation References:

22. Limitorque Corp Test Report #600198.

1. Conax Corp. Test Report IPS-234.

Notes: (A) Letters of J. Tillinghast (AEP) to K. Kniel (NRC) dated 4-14-75 and 9-29-75.

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: N/A COMPONENT: INSTRUMENTATION TERMINATION MANUFACTURER: N/A MODEL NUMBER: BARTON INSTRUMENT TERMINATION FUNCTION: ACCURACY: SPEC: N/A DEMON: N/A SERVICE: VARIOUS LOCATION: IN AND OUT Containment	Operating Time	1 year	See Note 1 on: Cable Termination	Table 7.5-2	8, 9, 10,	11, 12, 14 Separate	
	Temperature (°F)	Fig 0L2.9-1,-2 328.2	340	FSAR App Q	8, 9,	10, 11, 12, 14 Separate	
	Pressure (PSIA)	Fig 1 FIG 2	119.7	AEW 6504	8, 9,	10, 11, 12, 14 Separate	
	Relative Humidity (%)	100	100		8, 9,	10, 11, 12, 14 Separate	
	Chemical Spray	2000 ppm B	2000 ppm B	T.S. 3/4.5 3/4.5.6	8, 9,	10, 11, 12, 14 Separate	
	Radiation (10^6 rads)	150	150	WCAP 7410-L Vol 1	8, 9,	10, 11, 12, 14 Separate	
	Aging (years)		250°F, 1 days Yes		8, 9,	10, 11, 12, 14 Separate	
FLOOD LEVEL ELEV: 614 ABOVE FLOOD LEVEL: N/A	Submergence		Yes		8, 9,	10, 11, 12, 14, 17, 18, 19 Separate	

*Documentation References:

8. Conax Corp Test Report IPS-348
9. FIRL Test Report F-C 4033-1
10. FIRL Test Report F-C 3683
11. Isomedix Corp Test Report of MAY 1976
12. Cerro Wire + Cable Test Report of MAY 1976
14. FIRL Test Report F-C 4083-3
17. Conax Corp Test Report IPS-326
18. " " " " IPS-327
19. " " " " IPS-329

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: N/A COMPONENT: RTD TERMINATION MANUFACTURER: N/A MODEL NUMBER: RTD TERMINATION FUNCTION: ACCURACY: SPEC: N/A DEMON: N/A SERVICE: VARIOUS LOCATION: In and Out Containment	Operating Time	1 YEAR	See Note 1 on Cable Terminal	Table 7.5-2	8, 9,	10, 11, 12, 14 Separate	
	Temperature (°F)	Fig 022.9-1,-2 328.2	340	FSAR App Q	8, 9,	10, 11, 12, 14 Separate	
	Pressure (PSIA)	Fig 1 Fig 2	119.7	AEW 6504	8, 9,	10, 11, 12, 14 Separate	
	Relative Humidity (%)	100	100		8, 9,	10, 11, 12, 14 Separate	
	Chemical Spray	2000 ppm B	2000 ppm B	T.S. 3/4.5 3/4.5.6	8, 9,	10, 11, 12, 14 Separate	
	Radiation (10^6 rads)	150	150	WCAP 7410-L Vol 1	8, 9,	10, 11, 12, 14 Separate	
	Aging (years)		250°F 7 days Yes		8, 9,	11, 12, 14 Separate	
FLOOD LEVEL ELEV: 611' ABOVE FLOOD LEVEL: No	Submergence		Yes		8, 9,	10, 11, 12, 14, 17, 18, 19 Separate	19

*Documentation References:

8. Conax Corp Test Report IPS-348
9. FIRL Test Report F-C 4033-1
10. FIRL Test Report F-C 3603
11. Isomedix Corp Test Report of May 1976
12. Cerro Wire & Cable Test Report of May 1976
14. FIRL Test Report F-C 4033-3
17. Conax Corp Test Report IPS-326
18. " " " " IPS-327
19. " " " " IPS-329

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: VARIOUS PLANT ID NO: N/A COMPONENT: INSTRUMENT CABLE TERMINATION MANUFACTURER: N/A MODEL NUMBER: INSTRUMENT CABLE SPLICE AT PENETRATION FUNCTION: ACCURACY: SPEC: N/A DEMON: N/A SERVICE: VARIOUS LOCATION: In Containment	Operating Time	1 YEAR	See Note 1 on Cable Terminal	Table 7.5-2	8, 9, 10,	11, 12, 14, 18, 19 Separate	
	Temperature (°F)	Fig 022.9-1, -2 328.2	340	CSAR App Q	8, 9, 10,	11, 12, 14, 18, 19 Separate	
	Pressure (PSIA)	Fig 6 FIG2	119.7	AECO 6504	8, 9, 10,	11, 12, 14, 18, 19 Separate	
	Relative Humidity (%)	100	100		8, 9, 10,	11, 12, 14, 18, 19 Separate	
	Chemical Spray	2000 ppm B	2000 ppm B	T.S. 3/4.5 3/4.5.6	8, 9, 10,	11, 12, 14, 18, 19 Separate	
	Radiation (10^6 rads)	150	150	WCAP 7410-L Vol 1	8, 9, 10,	11, 12, 14, 18, 19 Separate	
	Aging (years)		250°F 7 days Yes		8, 9,	10, 11, 12, 14, 18, 19 Separate	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: No	Submergence		Yes		8, 9,	10, 11, 12, 14, 18, 19, 17 Separate	1

*Documentation References:

8. Conax Corp. Test Report IPS-348
9. FIRL Test Report F-C 4033-1
10. FIRL Test Report F-C. 3683
11. Isomedix Corp. Test Report of may 1976
12. Cerro Wire & Cable Test Report of MAY 1976
14. FIRL Test Report F-C 4033-3
18. Conax Corp. Test Report IPS-327
19. Conax Corp Test Report IPS-329
17. " " " " IPS-326 1

Notes:

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: Various	Operating Time	1 day	28 hr	Data Analysis	44	Seq	
PLANT ID NO: Various	Temperature (°F)	Fig. Q-27	212	FSAR App 10]	44	Seq.	
COMPONENT: Valve Motor Operator	Pressure (PSIA)	Fig Q-27	14.7	FSAR App Q	44	Seq.	
MANUFACTURER: Limitorque	Relative Humidity (%)	NA	100		44	Seq.	
MODEL NUMBER: Various	Chemical Spray	NA	NA	NA	NA	N A	
FUNCTION: Various	Radiation (10^6 rads)	4.1	See Valve Motor Operator Note 1	AEPSC NSEL calc. DC-N-6010-2		Seq Attach. # 1 valve motor oper Note 1	
ACCURACY: SPEC: NA DEMON: NA	Aging (years)						
SERVICE: VARIOUS	Submergence	NA	NA	NA	NA	NA	
LOCATION: Outside Containment							
FLOOD LEVEL ELEV: NA ABOVE FLOOD LEVEL: NA							

*Documentation References:

44. FIRL TEST REPORT F-C3271

Notes:

A) Letters from J. Tillinghast (AEP) to K. Kniel (NRC) dated 4-14-75 & 9-29-75.

Rev 1 9/2/80

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2 | DOCKET NO. 50-316 | LICENSE NO. DPR-74

EQUIPMENT DESCRIPTION	ENVIRONMENT			DOCUMENTATION REF.*		QUALIFICATION METHOD	OUTSTANDING ITEMS
	PARAMETER	SPEC.	QUAL.	SPEC.	QUAL.		
SYSTEM: RESIDUAL HEAT REMOVAL PLANT ID NO: ICM-305, 306 COMPONENT: VALVE MOTOR OPERATOR MANUFACTURER: LIMITORQUE MODEL NUMBER: SMB-2 FUNCTION: Long term post accident cooling ACCURACY: SPEC: NA DEMON: NA SERVICE: RHR suction from CT. sump. LOCATION: Inside CT. EXTENSION	Operating Time	1 day	6 days	NOTE A Below	22	Simul.	
	Temperature (°F)	Fig 1027.9-1,2	330	FSAR APP Q	22	SIMUL.	
	Pressure (PSIA)	Fig 1 Fig 2	104.7	NEW 6504	22	SIMUL.	
	Relative Humidity (%)	100	100		22	SIMUL	
	Chemical Spray	NA	2600 ppm B	INSIDE CT. EXTENS.	22	SIMUL	
	Radiation (10^6 rads)	24.6	SEE NOTE B	AEPSC CALC. E-N-6420-2	SEE NOTE B	SEE NOTE B	
	Aging (years)		150°C/100hr YES		22	Simul.	
FLOOD LEVEL ELEV: 614' ABOVE FLOOD LEVEL: YES	Submergence	NA	NA	NA	NA	NA	

*Documentation References:

22. LIMITORQUE CORP. TEST REPORT # 600198

Notes:

(A) Letters of J. Tillinghast (AEP) to K. Kniel (NRC) dated 4/14/75 & 9/29/75.

(B) These are Westinghouse supplied valves, insulation class H, specified for nuclear service inside containment. Limit switch material for these valves is white melamine (radiation resistant material). On the basis of engineering judgement this valve motor operator is qualified for its required exposure. We are seeking further supplementary data to strengthen this judgement.

Page VI D-1

Rev. 1 9/2/80

RIII'S CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS (BWR's and PWR's)

Docket No. 50-315
Docket No. 50-316

American Electric Power Service
Corporation
Indiana and Michigan Power Company
ATTN: Mr. John E. Dolan
Vice Chairman
Engineering and Construction
2 Broadway
New York, NY 10004

cc w/encl:
Mr. D. V. Shaller,
Plant Manager
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Ronald Callen, Michigan
Public Service Corporation

RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket No. 50-266
Docket No. 50-301

Wisconsin Electric Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
Power Plants
231 West Michigan
Milwaukee, WI 53201

cc w/encl:
Mr. G. A. Reed, Plant Manager
Central Files
Director, NRR/DPM
Director, NRR/DOR
Timothy Colburn, ORB/NRR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Sandra A. Bast, Lakeshore Citizens for Safe Energy
Mr. John J. Duffy, Chief Boiler Inspector
Mr. Peter Anderson, Wisconsin's Environmental Decade

Docket No. 50-305

Wisconsin Public Service Corporation
ATTN: Mr. E. R. Mathews
Vice President
Power Supply and Engineering
P. O. Box 1200
Green Bay, WI 54305

cc w/encl:
D. C. Hintz, Plant Superintendent
Mr. W. Sayles, Chief Engineer
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Mr. John J. Duffy, Chief Boiler Inspector

RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket Nos. 50-10, 50-237, 50-249;
50-254, 50-265; 50-295, 50-304; 50-373,
50-374; 50-454, 50-455; 50-456 and 50-457

Commonwealth Edison Company
ATTN: Mr. Cordell Reed,
Vice President
Post Office Box 767
Chicago, IL 60690

cc w/encl:
Mr. J. S. Able, Director
of Nuclear Licensing
Mr. D. J. Scott, Station
Superintendent
Mr. N. Kalivianakis,
Station Superintendent
Mr. K. Graesser, Station
Superintendent
Mr. L. J. Burke, Site
Construction Superintendent
Mr. T. E. Quaka, Quality
Assurance Supervisor
Mr. R. H. Holyoak, Station
Superintendent
Mr. B. B. Stephenson,
Project Manager
Mr. V. I. Schlosser,
Project Manager
Mr. R. E. Querio, Station
Superintendent
Mr. Gunner Sorensen, Site
Project Superintendent
Mr. R. Cosaro, Project
Superintendent
Mr. J. F. Gudac, Station
Superintendent
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspectors, RIII
PDR
Local PDR
NSIC
TIC
Mr. Dean Hansell, Office of
Assistant Attorney General
Myron M. Cherry, Chicago

Docket No. 50-358

Cincinnati Gas and Electric
Company
ATTN: Mr. Earl A. Borgmann
Senior Vice President
Engineering Services
and Electric Production
139 East 4th Street
Cincinnati, OH 45201

cc w/encl:
Mr. J. R. Schott, Plant
Superintendent
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Harold W. Kohn, Power
Siting Commission
Citizens Against a Radioactive
Environment
Helen W. Evans, State of Ohio

RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket No. 50-440
Docket No. 50-441

The Cleveland Electric Illuminating
Company
ATTN: Mr. Dalwyn R. Davidson
Vice President - Engineering
P. O. Box 5000
Cleveland, OH 44101

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
PDR
Local PDR
NSIC
TIC
Harold W. Kohn, Power
Siting Commission
Mr. Daniel D. Wilt,
Attorney
Helen Evans,
State of Ohio

Docket No. 50-155
Docket No. 50-255

Consumers Power Company
ATTN: Mr. R. B. DeWitt
Vice President
Nuclear Operations
212 West Michigan Avenue
Jackson, MI 49201

cc w/encl:
Mr. D. P. Hoffman, Nuclear
Licensing Administrator
Mr. C. J. Hartman,
Plant Superintendent
Mr. R. W. Montross, Manager
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspectors, RIII
PDR
Local PDR
NSIC
TIC
Ronald Callen, Michigan
Public Service Commission
Myron M. Cherry, Chicago

RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket No. 50-329
Docket No. 50-330

Consumers Power Company
ATTN: Mr. James W. Cook
Vice President
Midland Project
1945 West Parnall Road
Jackson, MI 49201

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
PDR
Local PDR
NSIC
TIC
Ronald Callen, Michigan
Public Service Commission
Myron M. Cherry, Chicago

Docket No. 50-409

Dairyland Power Cooperative
ATTN: Mr. F. W. Linder
General Manager
2615 East Avenue - South
La Crosse, WI 54601

cc w/encl:
Mr. R. E. Shimshak,
Plant Superintendent
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Mr. John Duffy, Chief
Boiler Inspector

RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket No. 50-341

The Detroit Edison Company
ATTN: Mr. Edward Hines, Assistant
Vice President and Manager
Quality Assurance
2000 Second Avenue
Detroit, MI 48226

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Ronald Callen, Michigan Public
Service Commission
Eugene B. Thomas, Jr.,
Attorney

Docket No. 50-461
Docket No. 50-462

Illinois Power Company
ATTN: Mr. W. C. Gerstner
Executive Vice President
500 South 27th Street
Decatur, IL 62525

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
PDR
Local PDR
NSIC
TIC
Mr. Dean Hansell, Office of
Assistant Attorney General
Mr. Gary N. Wright, Chief
Division of Nuclear Safety

RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket No. 50-331

Iowa Electric Light and Power
Company
ATTN: Mr. Duane Arnold
President
IE Towers
P. O. Box 351
Cedar Rapids, IA 52406

cc w/encl:
D. Mineck, Chief
Engineer
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC

Docket No: 50-367

Northern Indiana Public Service
Company
ATTN: Mr. Eugene M. Shorb
Senior Vice President
5265 Hohman Avenue
Hammond, IN 46325

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
PDR
Local PDR
NSIC
TIC
Mr. Dean Hansell, Office of
Assistant Attorney General