

Central file

INDIANA & MICHIGAN ELECTRIC COMPANY

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BOWLING GREEN STATION
NEW YORK, N. Y. 10004

December 11, 1979
AEP:NRC:00296

Donald C. Cook Nuclear Plant Unit 2
Docket No. ~~50-315~~ 50-316
License No. DPR-74
Cycle 2 Refueling Outage

Mr. J. G. Keppler, Regional Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region III
799 Roosevelt Road
Glen Ellyn, IL. 60137

Dear Mr. Keppler:

This letter provides information concerning our plans for the refueling outage for Cycle 2 in the Donald C. Cook Nuclear Plant Unit 2. Cycle 1 was shutdown on October 19, 1979. The startup of Cycle 2 is expected about the middle of December, 1979.

The new fuel batch will consist of eighty fuel assemblies which were manufactured by the Westinghouse Electric Corporation, as was the Cycle 1 fuel. The new fuel is designated as Region 4 and is enriched to 3.40 w/o U-235. The new fuel will replace sixty-five Region 1 and fifteen Region 2 fuel assemblies. Forty-eight Region 4 fuel assemblies will contain fresh burnable poisons. Two symmetrically located Region 3 fuel assemblies will contain secondary source rods and their associated depleted burnable poisons. Located 90° from these assemblies are two Region 3 assemblies with matching depleted burnable poisons inserted to preserve core symmetry. There will also be two additional secondary source assemblies added in Cycle 2 for irradiation.

SENT TO PDR 1-3-80
per JER & RTH

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Mr. J. G. Keppler, Regional Director

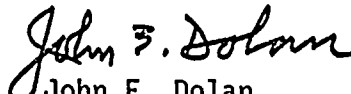
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The reload fuel has the same mechanical and thermal hydraulic design as the Cycle 1 fuel. The reload safety evaluation was made in accordance with the methodology presented in Reference 1. The FSAR analyses which could potentially be affected by the fuel reload were reviewed for the Cycle 2 core design. It has been concluded that Cycle 2 operation will not cause the FSAR safety limits to be exceeded.

The reload core design and safety analysis were reviewed by the Nuclear Safety Design Review Committee (NSDRC) and the Plant Nuclear Safety Review Committee (PNSRC). Both committees have concluded that no unreviewed safety question is involved and no Technical Specification change is required.

Very truly yours,


John E. Dolan
Vice President

JED:em

Reference 1: WCAP-9273, "Westinghouse Reload Safety Evaluation Methodology,"
March 1978.

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Mr. J. G. Keppler, Regional Director

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DC-N-6695.7