

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 500 HRS. REPORTED LESSONS LEARNED ARE
INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND
RECORDS MANAGEMENT BRANCH (T-4 F33), U.S. NUCLEAR REGULATORY
COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION
PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC
20503

FACILITY NAME (1) Cook Nuclear Plant Unit 2	DOCKET NUMBER (2) 05000-316	PAGE (3) 1 of 1
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TITLE (4) Degraded Component Cooling Water Flow to Containment Main Steam Line Penetrations
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EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
06	10	1996	1999	--	001	--	00	03	29	1999	Cook Plant Unit 1 05000-315
OPERATING MODE (9) 1											
POWER LEVEL (10) 100											
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)											
<input type="checkbox"/> 20.2201 (b)											
<input type="checkbox"/> 20.2203(a)(1)											
<input type="checkbox"/> 20.2203(a)(2)(i)											
<input type="checkbox"/> 20.2203(a)(2)(ii)											
<input type="checkbox"/> 20.2203(a)(2)(iii)											
<input type="checkbox"/> 20.2203(a)(2)(iv)											
<input checked="" type="checkbox"/> 20.2203(a)(2)(v)											
<input type="checkbox"/> 20.2203(a)(3)(i)											
<input type="checkbox"/> 20.2203(a)(3)(ii)											
<input type="checkbox"/> 20.2203(a)(4)											
<input type="checkbox"/> 50.73(a)(2)(i)											
<input checked="" type="checkbox"/> 50.73(a)(2)(ii)											
<input type="checkbox"/> 50.73(a)(2)(iii)											
<input type="checkbox"/> 50.73(a)(2)(iv)											
<input type="checkbox"/> 50.73(a)(2)(v)											
<input type="checkbox"/> 50.73(a)(2)(vi)											
<input type="checkbox"/> 50.73(a)(2)(vii)											
<input type="checkbox"/> 50.73(a)(2)(viii)											
<input type="checkbox"/> 50.73(a)(2)(ix)											
<input type="checkbox"/> 50.73(a)(2)(x)											
<input type="checkbox"/> 73.71											
<input type="checkbox"/> OTHER											
Specify in Abstract below or on NRC Form 366A											

LICENSEE CONTACT FOR THIS LER (12)	
NAME Lyle R. Berry, Regulatory Compliance Engineer	TELEPHONE NUMBER (Include Area Code) (616) 465-5901 x2637

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input checked="" type="checkbox"/> YES	(If Yes, complete EXPECTED SUBMISSION DATE).			<input type="checkbox"/> NO		06	25	1999

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On February 26, 1999, during an engineering review of the Unit 2 Component Cooling Water System (CCW), it was identified that power operation was permitted in June 1996 with degraded CCW flow to the coolers for containment penetrations 2-CPN-3 and 2-CPN-4. The main steam headers for steam generators 22 and 23 pass through these penetrations. Operating with the degraded CCW to these coolers may have resulted in excessive thermal stress on the penetration sleeves/liners.

The Updated Final Safety Analysis Report (UFSAR) credits the penetration coolers for maintaining the temperature of adjacent concrete less than 150 degrees F. At the time of the event, containment exterior surface concrete temperatures were measured at a maximum of 155 degrees F. Operability determinations performed at the time of the event, to justify continued operation with degraded CCW flow to the main steam line penetrations, have been re-evaluated and found to be inadequate. This event was reported via a 4-hour non-emergency Emergency Notification System (ENS) report on February 27, 1999, pursuant to the requirements of 10CFR50.72(b)(2)(i). This LER is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(ii)(B) as an event or condition that resulted in the nuclear power plant being in a condition that was outside the design basis of the plant.

Initial investigations indicate that the cause for this event was an inadequate understanding of the design basis for the containment penetration coolers. Additional investigations to assess potential degradation to the concrete, liners and sleeves at these penetrations are ongoing. The results of those investigations, including identified root cause(s), preventive actions and safety significance will be provided when completed.