NRC Form 366 (6-1998) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

APPROVED BY OMB NO. 3150-0104 EXPIRES 06/30/2001

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Cook Nuclear Plant Unit 1	05000-315	1 of 1

TITLE (4)

Calculations Show that the Divider Barrier Between Upper and Lower Containment Volumes May Be Overstressed

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER			REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	20	1998	1999	--	007	--	00	04	01	1999	Cook Unit 2	05000-316
											FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)	00	20.2201 (b)		20.2203(a)(2)(v)		50.73(a)(2)(i)		50.73(a)(2)(viii)	
		20.2203(a)(1)		20.2203(a)(3)(i)		X 50.73(a)(2)(ii)		50.73(a)(2)(x)	
		20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71	
		20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER	
		20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A	
		20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)			

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Mr: Donald C. Kosloff, Compliance Engineer	616/465-5901, X2129

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X	YES (If Yes, complete EXPECTED SUBMISSION DATE).		NO		08	20	1999

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

While working on a preliminary calculation for the Unit 1 steam generator replacement project, it was concluded that some of the structural load cases indicated overstressed conditions in the steam generator enclosures. Based on these preliminary analyses, on October 20, 1998, with both units in Mode 5, engineering personnel determined that a postulated steam line break could result in calculated stresses exceeding the code allowable stresses for the Unit 1 steam generator enclosures. These reinforced concrete enclosures are part of the divider barrier between the upper and lower containment volumes. Overstressed conditions in these enclosures could allow increased steam bypass flow around the ice condensers which could result in higher than expected containment pressure.

When the potential overstress conditions were identified, it was concluded that the conditions could be due to significant conservatisms in the preliminary calculation and in its interpretation. Engineering continued their evaluation of this issue. However, as evaluations continued, a lack of information confirming an acceptable condition led to the conclusion that there was no longer a reasonable expectation that the structures would not be overstressed. On March 2, 1999, with both units still in Mode 5, the apparent overstress conditions were determined to be reportable under 10CFR50.72(b)(2)(i) and 50.73(a)(2)(ii). Based on this determination, an ENS notification was made on March 2, 1999, at 1314 hours in accordance with 10CFR50.72(b)(2)(i).

Structural engineers are currently developing appropriate analyses of the loads and stresses on the enclosures. The results of these analyses will determine if an overstressed condition could have occurred and, if so, the safety significance of the overstressed condition. Appropriate corrective action will be developed based on an evaluation of the analytical results.