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 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
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SUBJECT: Monthly operating rept for Mar 1989 for DC Cook Nuclear
 Plant. W/890407 ltr.

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U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

April 7, 1989

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 2
Technical Specification 6.9.1.10, the attached Monthly Operating
Report for the month of ~~February~~ ^{March} 1989 is submitted.

Sincerely,

W. G. Smith, Jr.
Plant Manager

WGS/jd

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N.R.C. OPERATING DATA REPORT

DOCKET NO. 50-316
 DATE 03-Apr-89
 COMPLETED BY H. Giles
 TELEPHONE 616-465-5901

OPERATING STATUS

1. Unit Name D. C. Cook Unit 2 -----
2. Reporting Period MAR 89 notes
3. Licensed Thermal Power (MWt) 3411
4. Name Plate Rating (Gross MWe) 1133
5. Design Electrical Rating (Net MWe) 1100
6. Maximum Dependable Capacity (GROSS MWe) 1100
7. Maximum Dependable Capacity (Net MWe) 1060 -----
8. If Changes Occur in Capacity Ratings (Items no. 3 through 7) Since Last Report Give Reasons_____
9. Power Level To Which Restricted. If Any (Net MWe)_____
10. Reasons For Restrictions. If Any:_____

	This Mo.	Yr. to Date	Cumm.
11. Hours in Reporting Period	744.0	2160.0	98592.0
12. No. of Hrs. Reactor Was Critical	395.4	395.4	63990.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator on Line	345.7	345.7	62555.7
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Therm. Energy Gen. (MWH)	775260	775260	190341637
17. Gross Elect. Energy Gen. (MWH)	245880	245880	62142100
18. Net Elect. Energy Gen. (MWH)	233812	233812	59818477
19. Unit Service Factor	46.5	16.0	65.0
20. Unit Availability Factor	46.5	16.0	65.0
21. Unit Capacity Factor (MDC Net)	29.6	10.2	58.6
22. Unit Capacity Factor (DER Net)	28.6	9.8	57.1
23. Unit Forced Outage Rate	0.8	0.8	14.5
24. Shutdowns Scheduled over Next Six Months (Type, Date, and Duration):			

25. If Shut Down At End of Report Period, Estimated Date of Startup:_____
26. Units in Test Status (Prior to Commercial Operation):
- | | | |
|----------------------|----------|----------|
| | Forecast | Achieved |
| INITIAL CRITICALITY | | |
| INITIAL ELECTRICITY | | |
| COMMERCIAL OPERATION | | |

AVERAGE DAILY POWER LEVEL (MWe-Net)

DOCKET NO. 50-316
 UNIT TWO
 DATE 01-Apr-89
 COMPLETED BY H. Giles
 TELEPHONE 616-465-5901

MONTH MAR 89

DAY	AVERAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL
1	0	17	121
2	0	18	298
3	0	19	398
4	0	20	459
5	0	21	461
6	0	22	456
7	0	23	454
8	0	24	160
9	0	25	498
10	0	26	933
11	0	27	1096
12	0	28	1100
13	0	29	1105
14	0	30	1110
15	0	31	1092
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH

DOCKET NO. 50-316
 UNIT NAME D.C. Cook-U2
 DATE April 7, 1989
 COMPLETED BY T. R. Stephens
 TELEPHONE (616) 465-5901

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
189	880423	S	395.5	B&C	1	N/A	ZZ	ZZZZZZ	The unit was removed from service on 880423 for cycle 6-7, refueling and steam generator repair project. Startup testing and low power physics testing were completed on March 17, 1989 with the unit being paralleled to the system on 1129 hours the same day. Reactor power reached 48% RTP on March 19, 1989. The total length of the SG RP outage was 7880.4 hours.
190	89-324	S	2.8	B	1	N/A	ZZ	ZZZZZZ	The unit was removed from service at 0955 hours to repair letdown isolation valve 2-QRV-112 with the reactor remaining critical. Following repairs and main turbine overspeed testing the unit was paralleled at 1240 hours the same day. The reactor reached 100% RTP on March 28, 1989.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

4
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
 Exhibit I - Same Source

(9/77)

DOCKET NO: 50-316
UNIT NAME: D.C. Cook Unit 2
COMPLETED BY: T. R. Stephens
TELEPHONE: (616) 465-5901
DATE: April 7, 1989
PAGE: 1 of 3

MONTHLY OPERATING ACTIVITIES - MARCH 1989

HIGHLIGHTS

The unit entered the reporting period in Mode 4 with the reactor coolant system (RCS) being cooled down to repair leak on control rod drive mechanism housing for control rod located at P-6. Following repairs, the RCS was returned to Mode 3 and no load rated temperature and pressure. Cool down and depressurization was again initiated on March 10, 1989 to permit repairs of No. 1 steam generator stop valve three-way test valve, MM0-210. The RCS was again returned to Mode 3 and NO load rated temperature and pressure on March 14, 1989. Initial criticality for Cycle-11 was achieved at 1238 hours on March 15, 1989. Following completion of low power physics testing the unit was paralleled to the system at 1129 hours on March 17, 1989.

Reactor thermal power reached 48% on March 19, 1989 at 1220 hours. This power level was maintained until 2100 hours on March 23, 1989 when a power reduction was commenced to perform repairs on letdown isolation valve, 2-QRV-112. The unit was removed from parallel at 0955 hours on March 24, 1989. Following repairs to 2-QRV-112 and turbine overspeed testing the unit was paralleled to the system at 1240 hours on the same day. Reactor power was gradually increased with 100% RTP being reached on March 28, 1989. The reporting period ended with a reactor power reduction, for condenser cleaning, being initiated at 2155 hours on March 31, 1989.

DETAILS

- 3-1-89 1934 Started RCS cooldown and depressurization.
- 3-2-89 0402 RCS entered Mode 5.
- 3-4-89 0125 Started RCS heatup and pressure increase in preparation to start-up.
- 0850 RCS entered Mode 4.
- 1930 RCS entered Mode 3.

3-10-89 1630 Began RCS cooldown and depressurization to repair 2-MM0-210.
1738 Spurious RTP trip signal received on low-low level on #21 SG.
2043 RCS entered Mode 4.

3-13-89 1100 Started RCS heatup in preparation for reactor start-up.
1704 RCS entered Mode 3.

3-14-89 0530 RCS at full temp and pressure.

3-15-89 0803 Commenced reactor start-up.
1017 Reactor entered Mode 2.
1238 Rx critical. First time in Cycle 11.

3-17-89 0325 Reactor entered Mode 1.
1129 Unit paralleled to system.

3-19-89 0545 Started power increase to 48%.
1220 RTP at 48%.

3-20-89 0630 QRV-112, letdown isolation valve failed closed.

3-23-89 2100 Started power reduction to repair letdown isolation valve
QVR-112, and perform turbine overspeed test.

3-24-89 0940 RTP at 8%.
0955 Removed generator from parallel by opening A1 and A2 breakers.
1012 Performed 110% and 112% overspeed test.
1240 Unit paralleled to system.
1435 Started power increase at 6%/hr to 48% then 2%/hr to 68% power.

3-25-89 1945 RTP power at 68% for Chemistry hold.

3-26-89 0027 Increasing RTP to 90%.
1045 RTP power at 90%.
1700 Increasing RTP to 99%.

1. The first part of the report is a general introduction to the subject of the study. It discusses the importance of the study and the objectives of the research.

2. The second part of the report is a detailed description of the methodology used in the study. It includes information about the sample size, the data collection methods, and the statistical analysis techniques.

3. The third part of the report is a presentation of the results of the study. It includes tables and figures that illustrate the findings of the research.

4. The fourth part of the report is a discussion of the results and their implications. It discusses the strengths and limitations of the study and provides suggestions for future research.

5. The fifth part of the report is a conclusion that summarizes the main findings of the study and provides a final statement on the importance of the research.

6. The sixth part of the report is a list of references that includes all the sources used in the study. It provides information about the authors, titles, and publication details of the references.

7. The seventh part of the report is an appendix that contains additional information that is not included in the main body of the report. It includes tables, figures, and other data that are relevant to the study.

8. The eighth part of the report is a glossary that defines the key terms and concepts used in the study. It provides a clear and concise explanation of the terminology used throughout the report.

9. The ninth part of the report is a list of figures and tables that are included in the report. It provides a brief description of each figure and table and indicates the page number where it can be found.

3-27-89 0005 RTP power at 99%.

3-28-89 1620 Increasing power to 100% RTP.

1826 RTP at 100%.

3-31-89 2155 Started power reduction to 90%. Reactor power being reduced
for condenser cleaning.

DOCKET NO. 50-316
UNIT NAME D.C.Cook-Unit No. 2
DATE April 7, 1989
COMPLETED BY T. R. Stephens
TELEPHONE (616) 465-5901
PAGE 1 of 1

MAJOR SAFETY-RELATED MAINTENANCE

MARCH 1989

- 2-M-1 Replaced the governor valve stem and the mechanical overspeed tappet assembly on the turbine driven Aux. feedpump.
- 2-M-2 Aligned 2-MMO-210 for proper flow path.

