

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8712080005 DOC. DATE: 87/12/01 NOTARIZED: NO DOCKET #  
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
 AUTH. NAME AUTHOR AFFILIATION  
 BEILMAN, T.P. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 SMITH, W.G. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-017-01: on 870815, failure to incorporate changes to  
 pressurizer level protection set values into procedures.  
 W/8 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4  
 TITLE: 50.73 Licensee Event Report. (LER), Incident Rpt, etc.

### NOTES:

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	NRR/DEST/ELB	1 1	NRR/DEST/ICSB	1 1
	NRR/DEST/MEB	1 1	NRR/DEST/MTB	1 1
	NRR/DEST/PSB	1 1	NRR/DEST/RSB	1 1
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	NRR/DREP/RAB	1 1	NRR/DREP/RPB	2 2
	NRR/DRIS/SIB	1 1	NRR/PMAS/ILRB	1 1
	REG FILE 02	1 1	RES DEPY GI	1 1
	RES TELFORD, J	1 1	RES/DE/EIB	1 1
	RGN3 FILE 01	1 1		
EXTERNAL:	EG&G GROH, M	5 5	FORD BLDG HOY, A	1 1
	H ST LOBBY WARD	1 1	LPDR	1 1
	NRC PDR	1 1	NSIC HARRIS, J	1 1
	NSIC MAYS, G	1 1		

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**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) <b>D. C. COOK NUCLEAR PLANT - UNIT 1</b>										DOCKET NUMBER (2) <b>0 5 0 0 0 3 1 5</b>										PAGE (3) <b>1 OF 0 3</b>												
TITLE (4) <b>FAILURE TO INCORPORATE CHANGES TO PRESSURIZER LEVEL PROTECTION SET VALUES INTO PROCEDURES</b>																																
EVENT DATE (5)			LER NUMBER (6)						REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES												DOCKET NUMBER(S)											
0	8	1	5	8	7	0	1	7	0	1														0 5 0 0 0 0								
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																													
6			20.402(b)						20.406(a)						50.73(a)(2)(iv)						73.71(b)											
POWER LEVEL (10)			20.406(a)(1)(i)						50.38(a)(1)						50.73(a)(2)(vi)						73.71(c)											
0 1 0 1 0			20.406(a)(1)(ii)						50.38(a)(2)						50.73(a)(2)(vii)						OTHER (Specify in Abstract below and in Text, NRC Form 364A)											
			20.406(a)(1)(iii)						X 50.73(a)(2)(ii)						50.73(a)(2)(viii)(A)																	
			20.406(a)(1)(iv)						50.73(a)(2)(iii)						50.73(a)(2)(viii)(B)																	
			20.406(a)(1)(v)						50.73(a)(2)(iv)						50.73(a)(2)(ix)																	
LICENSEE CONTACT FOR THIS LER (12)																																
NAME <b>T. P. BEILMAN</b> <b>INSTRUMENTATION AND CONTROL SUPERINTENDENT</b>																				TELEPHONE NUMBER												
																				AREA CODE												
6 1 6										4 6 5 1 5 9 0 1																						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC				
SUPPLEMENTAL REPORT EXPECTED (14)																				EXPECTED SUBMISSION DATE (15)												
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO										MONTH DAY YEAR												
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																

This revision to LER 50-315/87-017-00 is being submitted to correct typographical errors and note method of discovery (see marginal markings).

On August 7, 1987 it was discovered during routine periodic procedure reviews, that the values for the transmitter span for the pressurizer level were incorrect in calibration procedures 1 THP 6030 IMP.108, 109 and 100 "Pressurizer Level Protection Set for channels I, II and III," respectively. The correct values were contained in Engineering Control Procedure (ECP) 12-NI-01 approved and issued on January 28, 1977. These values had never been incorporated into Unit One procedures. It was determined on August 15, 1987 that the Unit One High Pressurizer Level Reactor Trip setpoint of 91 percent was the equivalent of 93.27 percent of indicated span as determined by the ECP values. This is outside the Technical Specification allowable value of less or equivalent to 93 percent.

The reason for the error was defective procedures although the actual cause for the condition is unknown due to the time frame involved.

The Unit One procedures were changed to reflect the ECP values and the pressurizer level transmitters were recalibrated with IMP.109 and 110 being completed on August 10, 1987 and IMP.108 being completed on August 20, 1987.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
D. C. COOK NUCLEAR PLANT - UNIT 1	0500031587	—	017	—01	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This revision to LER 50-315/87-017-00 is being submitted to correct typographical errors and note method of discovery (see marginal markings).

Conditions Prior to Occurrence

Unit One in Mode 6 (Refueling).

Description of Event

On August 7, 1987 it was discovered during routine periodic procedure reviews, that the values for the transmitter (EIIS/JC-D) span for the pressurizer level (PZR) were incorrect in calibration procedure \*\*1 THP 6030 IMP.108, 109 and 110 "Pressurizer Level Protection Set for Channels I, II and III", respectively. The correct values were contained in Engineering Control Procedure (ECP) 12-NI-01 "Pressurizer Level Transmitters," which had been issued on January 28, 1977. These values had never been incorporated into the Unit One procedures.

The result was that the reactor trip from pressurizer high level, which is normally set at 91 percent of span was set at the equivalent of 93.27 percent of the span under the values required by the ECP. This setpoint is outside of Technical Specification 2.2.1 allowable value of less than or equal to 93 percent of instrument span by 0.27 percent high. Further, instrument calibration within the acceptable +0.5 percent of the normal calibration setpoint tolerance and instrument drift between calibrations could have resulted in a trip setpoint as high as 94 percent.

There were no inoperative structures, components or systems that contributed to this event.

Cause of the Event

The cause of the event was the failure to incorporate the correct values into the procedures in 1977, although formal document control measures did not exist for ECP's until recently.

Analysis of the Event

This event is being reported per 10 CFR 50.73 (a)(2)(i) as the procedures did not literally comply with the Technical Specifications. The safety consequences of the setpoint being 0.27 percent higher than required is not significant, based on the magnitude of the error and that no credit was taken for this trip in the accident analysis.

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PDR ADOCK 05000315  
S DCD

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  D. C. COOK NUCLEAR PLANT - UNIT 1	DOCKET NUMBER (2)  0   5   0   0   0   3   1   5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8   7	—   0   1   7	—   0   1	0   3	OF	0   3

TEXT (If more space is required, use additional NRC Form 306A's) (17)

Corrective Actions

The ECP values were incorporated into the procedures following confirmation that they were valid. The transmitters were recalibrated using the correct values on August 20, 1987 for IMP.108 and August 10, 1987 for IMP's 109 and 110.

The plant has established a formal control and review process for corporate engineering initiated technical requirements. This will ensure the appropriate procedures are revised when the technical requirements change.

Although this is believed to be an isolated incident, a representative sample of safety related ECP's are being reviewed to verify their content and consistency with Technical Specifications and to identify possible discrepancies in the plant's procedures. Should any additional problems be discovered, a supplemental LER will be submitted.

Failed Component Identification

There were no component failures related to this event.

Previous Similar Events

None.



Indiana Michigan  
Power Company  
Cook Nuclear Plant  
P.O. Box 458  
Bridgman, MI 49106  
616 465 5901



INDIANA  
MICHIGAN  
POWER

December 1, 1987

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Operating License DPR-58  
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73  
entitled Licensee Event Reporting System, the following  
report is being submitted:

87-017-01

Sincerely,

*W. G. Smith, Jr.*  
W. G. Smith, Jr.  
Plant Manager

WGS:afh

Attachment

cc: J. E. Dolan  
A. B. Davis, Region III  
M. P. Alexich  
R. F. Kroeger  
H. B. Brugger  
R. W. Jurgensen  
NRC Resident Inspector  
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