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ACCESSION NBR: 8711110149 DOC. DATE: 87/11/05 NOTARIZED: NO DOCKET #
FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
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BEILMAN, T. P. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
SMITH, W. G. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-012-00: on 871008, reactor trip beakers opened. Caused by personnel error due to lack of attention. Individual involved counselled. W/871105 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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	PD3-3 LA	1	1	PD3-3 PD	1	1
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	ARM/DCTS/DAB	1	1	DEDRO	1	1
	NRR/DEST/ADS	1	0	NRR/DEST/CEB	1	1
	NRR/DEST/ELB	1	1	NRR/DEST/ICSB	1	1
	NRR/DEST/MEB	1	1	NRR/DEST/MTB	1	1
	NRR/DEST/PSB	1	1	NRR/DEST/RSB	1	1
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	NRR/DREP/RAB	1	1	NRR/DREP/RPB	2	2
	NRR/DRIS/SIB	1	1	NRR/PMAS/ILRB	1	1
	REG FILE 02	1	1	RES DEPY GI	1	1
	RES TELFORD, J	1	1	RES/DE/EIB	1	1
	RGN3 FILE 01	1	1			
EXTERNAL:	EG&G GROH, M	5	5	H ST LOBBY WARD	1	1
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	NSIC HARRIS, J	1	1	NSIC MAYS, G	1	1

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NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
D. C. COOK NUCLEAR PLANT - UNIT 2	0 5 0 0 0 3 1 6	1 OF 0 3

TITLE (4) INADVERTENT OPENING OF REACTOR TRIP BREAKERS CAUSED BY PERSONNEL ERROR - WRONG POWER RANGE NUCLEAR INSTRUMENTATION CHANNEL TESTED

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 0	0 8	8 7	8 7	— 0 1 2	— 0 0	1 1	0 5	8 7		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	3
POWER LEVEL (10)	0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

	20.402(b)		20.405(c)	X	50.73(a)(2)(iv)
	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)
	20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)

	73.71(b)
	73.71(c)
	OTHER (Specify in Abstract below and in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
T. P. BEILMAN INSTRUMENTATION AND CONTROL SUPERINTENDENT	AREA CODE 6 1 6 — 4 6 5 - 5 9 0 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 8, 1987 at 2328 hours, an Engineered Safety Features Actuation (Reactor Trip breakers opening) occurred due to personnel error. The unit was in Hot Standby and surveillance testing was being performed prior to startup. Instrument and Control (I and C) technicians placed Power Range Nuclear Instrumentation channel IV (N-44) into trip and mistakenly began testing on Channel III (N-43) which satisfied the two of four logic required for a reactor trip signal. Feedwater isolation followed due to coincident Low Tavg. Main Turbine and Main Feedpump Turbine trips were also received due to reactor trip. No abnormal reactor trip sequence responses were noted.

Immediate corrective actions included returning N-43 to normal and evaluating plant responses. N-44 surveillance testing was then completed. The event was discussed with the individual involved who was counselled to be more careful when working on safety related equipment, especially during times such as unit startup when numerous I and C activities are taking place. Additionally, a meeting was held with I and C personnel to discuss the event and to remind them of the responsibility each person in a work group has, to follow each step of the job to ensure its correct evolution.

IE 22
1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
D. C. COOK NUCLEAR PLANT - UNIT 2	0 5 0 0 0 3 1 6	8 7	— 0 1 2	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Conditions Prior to Occurrence

Unit Two - Mode 3 (hot standby).

Description of Event

On October 8, 1987 at 2328 hours, an Engineered Safety Features Actuation (Reactor Trip Breakers opening) occurred due to personnel error. The error was caused by inattention to the work detail. Unit Two was in Mode 3 with preparations being made for power operation. Two (I and C) technicians were performing surveillance testing on Power Range Nuclear Instrumentation (EIIS/IG) using approved procedures. Channel IV (N-44) was removed from service by placing test switches located behind the control panel to the trip position. This action satisfies one of the two channels necessary for two of four logic reactor trip. When returning to the front of the panel to perform testing on N-44, the lead technician mistakenly began the test on Channel III (N-43). When the trip setpoint was reached on N-43, the second logic channel was satisfied and an unexpected reactor trip signal occurred. The lead technician then recognized his error and returned N-43 to normal. Plant response consisted of the opening of reactor trip breakers (EIIS/JE-BKR), feedwater isolation (EIIS/JB) (from coincident low Tavg), main turbine (EIIS/TA-TRB) and feedpump turbine trip (EIIS/SI-P). All control and shutdown rods (EIIS/AA-ROD) were at bottom and motor driven auxiliary feedpumps (EIIS/BA-P) on prior to the event. All systems functioned as required. The NRC was notified of the event via the ENS at 0148 hours on October 9, 1987. There were no inoperative structures, components or systems that contributed to this event.

Cause of Event

Personnel error due to lack of attention caused this event. The procedure being used indicated N-44 to be tested. Per discussion with the technician involved, the channel identification was adequate.

Analysis of Event

This event is being reported per the requirements of 10CFR50.73(a)(2)(iv), as an event that resulted in an unplanned automatic actuation of an Engineered Safety Feature (reactor trip breakers). Assessment of required plant responses were performed and no abnormalities were noted.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
D. C. COOK NUCLEAR PLANT - UNIT 2	0 5 0 0 0 3 1 6	8 7	— 0 1 2	— 0 0	0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Action

N-43 was returned to normal. Operators assessed plant response to required automatic actions. The surveillance for N-44 was then successfully completed. The event was discussed with the individual involved who was counselled to be more careful when working on safety related equipment, especially during times such as unit startup when numerous I and C activities are taking place. Additionally, a meeting was held with I and C personnel to discuss the event and to remind them of the responsibility each person in a work group has, to follow each step of the job to ensure its correct evolution.

Failed Component Identification

None.

Previous Similar Events

None.

Indiana Michigan
Power Company
Cook Nuclear Plant
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616 465 5901

AEP
INDIANA
MICHIGAN
POWER

November 5, 1987

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Reporting System, the following report is being submitted:

87-012-0

Sincerely,

W. G. Smith, Jr.
Plant Manager

WGS:afh

Attachment

cc: John E. Dolan
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