

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

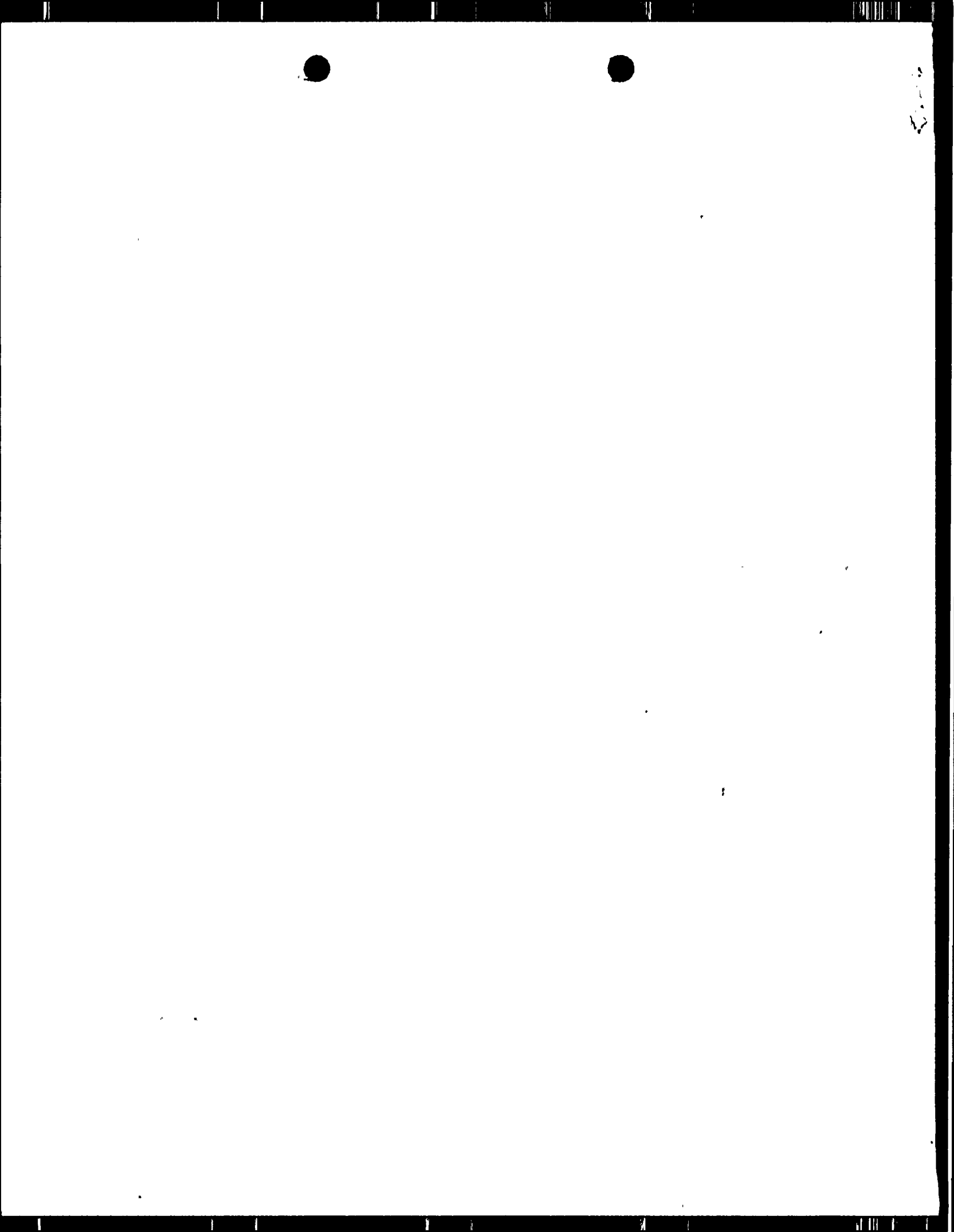
ACCESSION NBR: 8709230055 DOC. DATE: 87/09/16 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 AUTH. NAME AUTHOR AFFILIATION
 BEILMAN, T. P. Indiana & Michigan Power Co.
 SMITH, W. G. Indiana & Michigan Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-017-00: on 870815, discovered incorrect calibr of
 values for transmitter span for pressurizer level. Caused by
 failure to incorporate correct changes into procedures.
 Procedures revised. W/870916 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD3-3 LA	1 1	PD3-3 PD	1 1
	WIGGINGTON, D	1 1		
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	AEOD/DOA	1 1	AEOD/DSP/NAS	1 1
	AEOD/DSP/ROAB	2 2	AEOD/DSP/TPAB	1 1
	DEDRO	1 1	NRR/DEST/ADS	1 0
	NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
	NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
	NRR/DEST/RSB	1 1	NRR/DEST/SGB	1 1
	NRR/DLPQ/HFB	1 1	NRR/DLPQ/QAB	1 1
	NRR/DOEA/EAB	1 1	NRR/DREP/RAB	1 1
	NRR/DREP/RPB	2 2	NRR/DRIS/SIB	1 1
	NRR/PMAS/ILRB	1 1	REG-FILE 02	1 1
	RES DEPY GI	1 1	RES TELFORD, J	1 1
	RES/DE/EIB	1 1	RGN3 FILE 01	1 1
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1



LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT - UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 3 1 5 1 OF 0 3				PAGE (3) 1 OF 0 3		
TITLE (4) FAILURE TO INCORPORATE CHANGES TO PRESSURIZER LEVEL PROTECTION SET VALUES INTO PROCEDURES																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 8	1 5	8 7	8 7	0 1 7	0 0 0	9 1	6 8	7					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
6		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
0 0 0		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.406(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(vii)(A)						
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)						
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME T. P. BEILMAN INSTRUMENTATION AND CONTROL SUPERINTENDENT										TELEPHONE NUMBER						
										AREA CODE		6 1 6 4 6 5 - 5 9 0 1				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

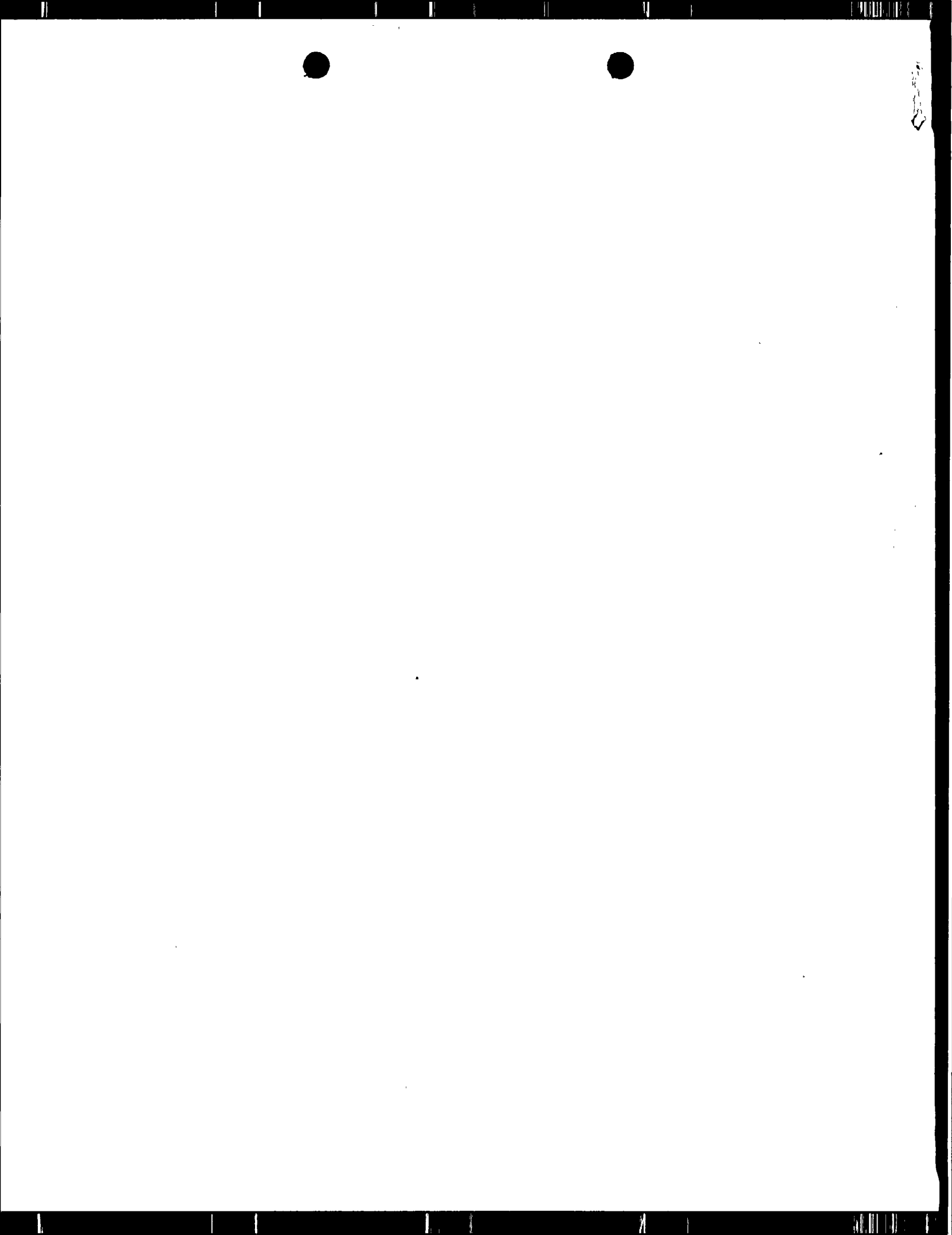
On August 7, 1987 it was discovered that the values for the transmitter span for the pressurizer level were incorrect in calibration procedures 1 THP 6030 IMP.108, 109 and 110 "Pressurizer Level Protection Set for Channels I, II and III," respectively. The correct values were contained in Engineering Control Procedure (ECP) 12-NI-01 approved and issued on January 28, 1977. These values had never been incorporated into Unit One procedures. It was determined on August 15, 1987 that the Unit One High Pressurizer Level Reactor Trip setpoint of 91 percent was the equivalent of 93.0027 percent of indicated span as determined by the ECP values. This is outside the Technical Specification allowable value of less than or equal to 93 percent.

The reason for the error was defective procedures although the actual cause for the condition is unknown due to the time frame involved.

The Unit One procedures were changed to reflect the ECP values and the pressurizer level transmitters were recalibrated with IMP.109 and 110 being completed on August 10, 1987 and IMP.108 being completed on August 20, 1987.

8709230055 870916
PDR ADOCK 05000315
S PDR

IE22 1/1



LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
D. C. COOK NUCLEAR PLANT - UNIT 1	0 5 0 0 0 3 1 5	8 7	— 0 1 7	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 306A's) (17)

Conditions Prior to Occurrence

Unit One in Mode 6 (Refueling).

Description of Event

On August 7, 1987 it was discovered that the values for the transmitter (EIIS/JC-D) span for the pressurizer level (PZR) were incorrect in calibration procedures **1 THP 6030 IMP.108, 109 and 110 "Pressurizer Level Protection Set for Channels I, II and III", respectively. The correct values were contained in Engineering Control Procedure (ECP) 12-NI-01 "Pressurizer Level Transmitters," which had been issued on January 28, 1977. These values had never been incorporated into the Unit One procedures.

The result was that the reactor trip from pressurizer high level, which is normally set at 91 percent of span was set at the equivalent of 93.0027 percent of the span under the values required by the ECP. This setpoint is outside of Technical Specification 2.2.1 allowable value of less than or equal to 93 percent of instrument span by 0.0027 percent high. Further, instrument calibration within the acceptable +0.05 percent of the normal calibration setpoint tolerance and instrument drift between calibrations could have resulted in a trip setpoint as high as 94 percent.

Cause of the Event

The cause of the event was the failure to incorporate the correct values into the procedures in 1977, although formal document control measures did not exist for ECP's until recently.

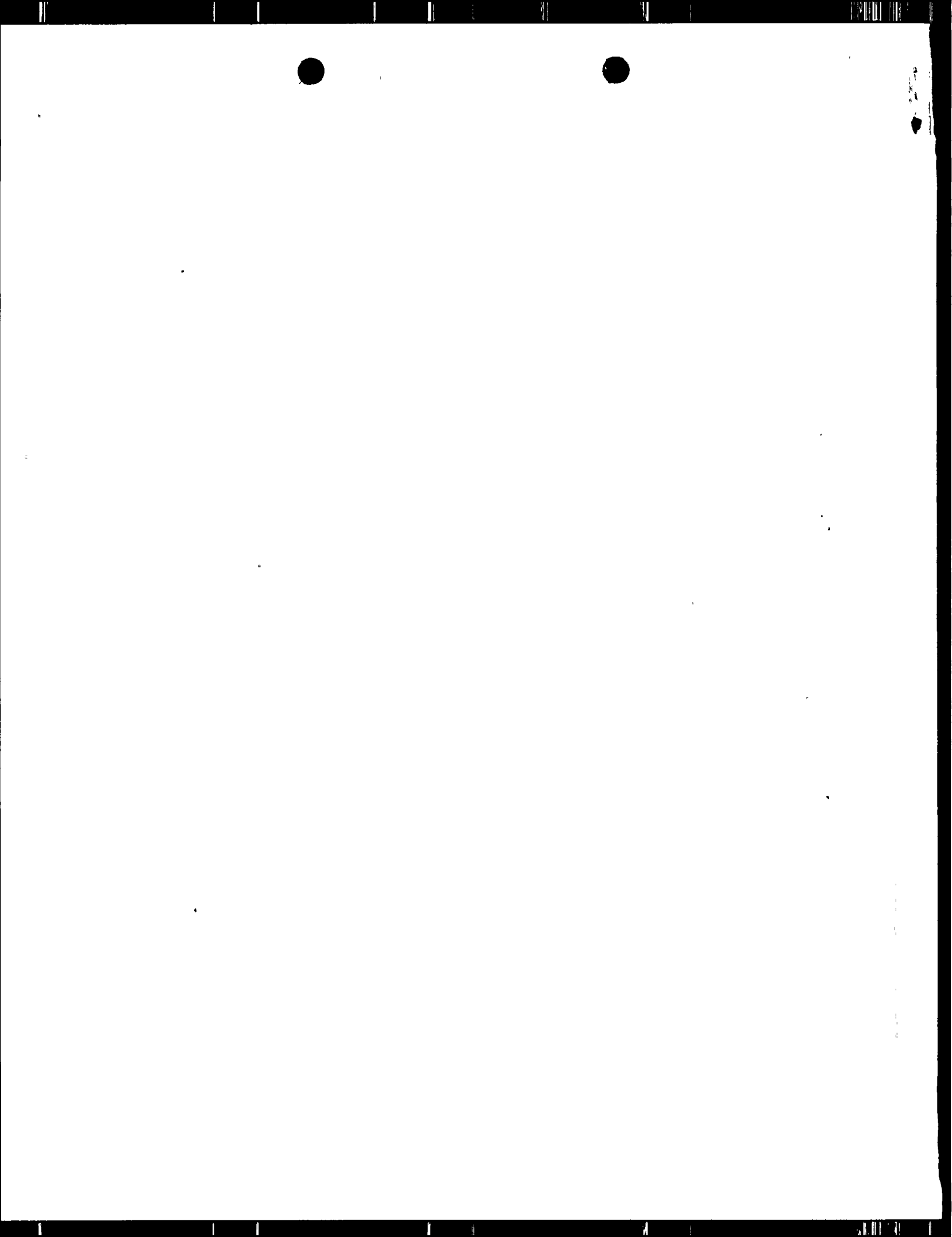
Analysis of the Event

This event is being reported per 10 CFR 50.73 (a)(2)(i) as the procedures did not literally comply with the Technical Specifications. The safety consequences of the setpoint being .0027 percent higher than required is not significant, based on the magnitude of the error and that no credit was taken for this trip in the accident analysis.

Corrective Actions

The ECP values were incorporated into the procedures following confirmation that they were valid. The transmitters were recalibrated using the correct values on August 20, 1987 for IMP.108 and August 10, 1987 for IMP's 109 and 110.

The plant has established a formal control and review process for corporate engineering initiated technical requirements. This will ensure the appropriate procedures are revised when the technical requirements change.



LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
D. C. COOK NUCLEAR PLANT - UNIT 1	0500031587	0	17	00	03	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

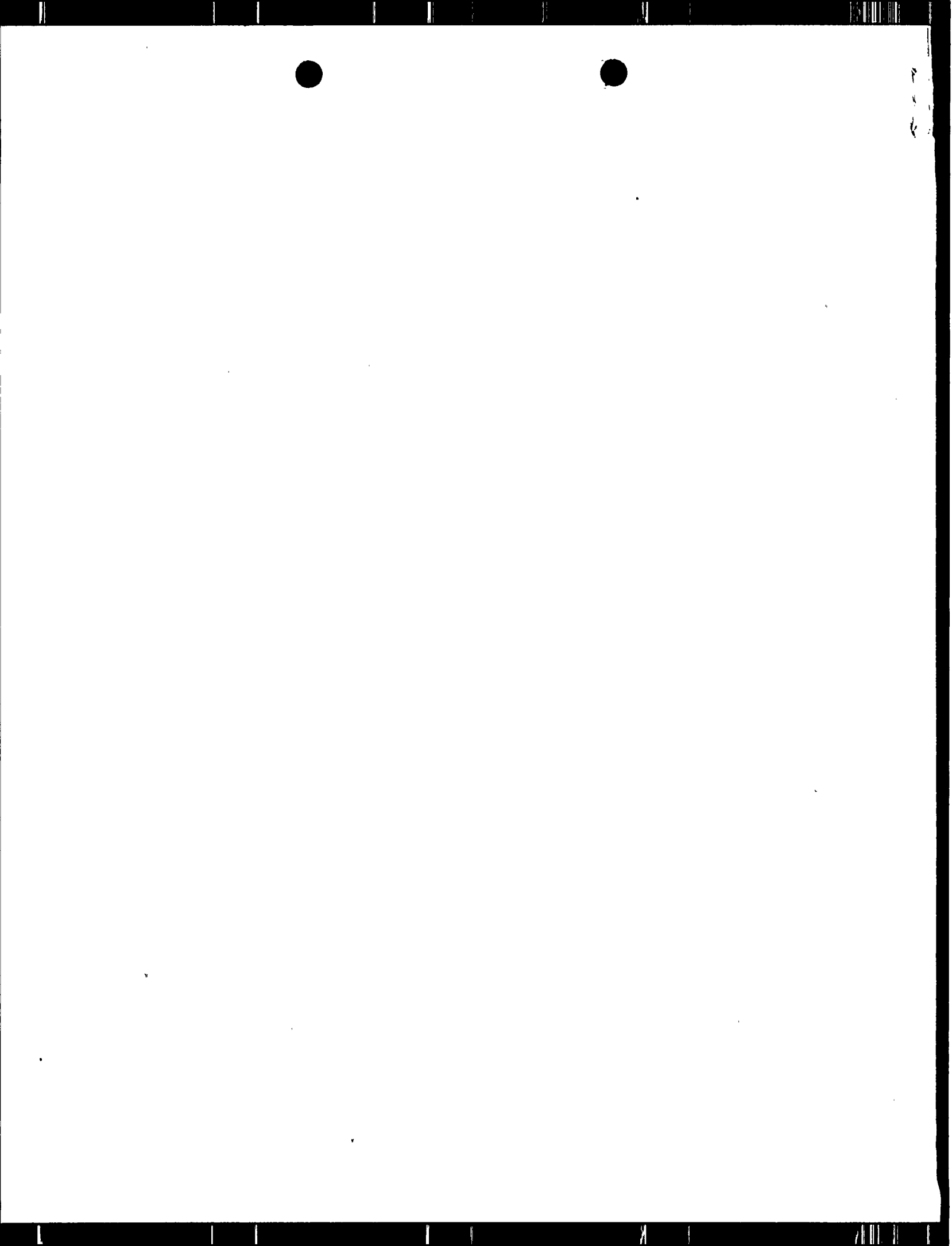
Although this is believed to be an isolated incident, a representative sample of safety related ECP's are being reviewed to verify their content and consistency with Technical Specifications and to identify possible discrepancies in the plant's procedures. Should any additional problems be discovered, a supplemental LER will be submitted.

Failed Component Identification.

There were no component failures related to this event.

Previous Similar Events

None.



Indiana Michigan
Power Company
Cook Nuclear Plant
P.O. Box 458
Bridgman, MI 49106
616 465 5901



September 16, 1987

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-58
Docket No. 50-315

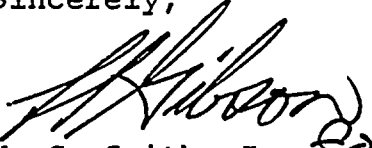
Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73
entitled Licensee Event Reporting System, the following
report is being submitted:

87-017-00

This report is being submitted one day after the 30-day
requirement due to the delays necessary to complete the
engineering evaluation.

Sincerely,


W. G. Smith, Jr. *son*
Plant Manager

/mmp
Attachment

cc: John E. Dolan
A. B. Davis, Region III
M. P. Alexich
R. F. Kroeger
H. B. Brugger
R. W. Jurgensen
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