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# INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631  
COLUMBUS, OHIO 43216

November 7, 1986  
AEP:NRC:1013

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2  
Docket Nos. 50-315 and 50-316  
License Nos. DPR-58 and DPR-74  
NRC INSPECTION REPORT NOS. 50-315/86030 (DRP)  
AND 50-316/86030 (DRP)

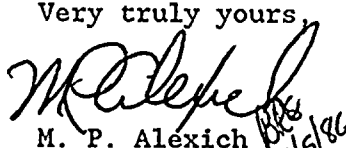
Mr. James G. Keppler, Regional Director  
U.S. Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Dear Mr. Keppler:

This letter is in response to Mr. W. G. Guldemon's letter dated October 8, 1986, which forwarded the report on the routine safety inspection conducted by members of your staff. This inspection was conducted from August 4 through September 8, 1986 on activities at the D. C. Cook Nuclear Plant Units 1 and 2. The Notice of Violation attached to Mr. Guldemon's letter identified two violations. Violation one is addressed in the attachment to this letter. The inspection showed that action had been taken to correct violation two and therefore, no reply is required.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

  
M. P. Alexich  
Vice President

MPA/cm  
Attachment

cc: John E. Dolan  
W. G. Smith, Jr. - Bridgman  
R. C. Callen  
G. Bruchmann  
G. Charnoff  
NRC Resident Inspector - Bridgman

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## Inspection Report 315/86030; 316/86030

NRC Violation

"10 CFR 50.73 (a)(2)(ii) requires that the licensee submit a Licensee Event Report (LER) for any event or condition that resulted in the condition of the power plant, including its principal safety barriers, being seriously degraded, or that resulted in the power plant being: (a) in an unanalyzed condition that significantly compromised plant safety; (b) in a condition that was outside the design basis of the plant; or (c) in a condition not covered by the plant's Operating and Emergency Procedures. Unit 1 and 2 Technical Specifications require all safety relief valves operable in Modes 1 through 3.

Contrary to the above, the licensee did not submit a Licensee Event Report when a number of the Unit 1 and Unit 2 main steam safety relief valves, a principle safety barrier, were found degraded during a Technical Specification surveillance test with the units in Mode 3."

Response to NRC ViolationCorrective Action Taken and Results Achieved

Upon discovery that several of the main steam safety valve setpoints were out of technical specification tolerance the immediate corrective action taken, as required by procedure 12 MHP 4030.STP.002 "Main Steam Setpoint Verification-Secondary System Safety Valve Setting", was to reset the safety valves setpoints to within their specified ranges.

Licensee Event Reports were not submitted at the time the valves were found to be out of tolerance, for reasons explained in LER No. 86-020-00. This event and the corrective actions have since been reported in detail in LER No. 86-020-00 submitted on September 24, 1986.

Corrective Action To Be Taken To Avoid Further Violations

Procedure 12 MHP 4030.STP.002 will be modified prior to the next scheduled surveillance to include instructions on reportability. In the future condition reports will be initiated as documentation of any safety valves failing to be in compliance.

Date When Full Compliance Achieved

Full compliance was achieved on September 24, 1986 upon issuance of LER No. 86-020-00.

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