

INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

September 19, 1986
AEP:NRC:1007

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
NRC INSPECTION REPORT NOS. 50-315/86027 (DRS)
AND 50-316/86027 (DRS)


Mr. James G. Keppler, Regional Director
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Mr. Keppler:

This letter is in response to Mr. C. W. Hehl's letter dated August 21, 1986, which forwarded the report on the routine safety inspection conducted by members of your staff. This inspection was conducted from July 7 through July 11, 1986 on activities at the D. C. Cook Nuclear Plant Units 1 and 2. The Notice of Violation attached to Mr. Hehl's letter identified one violation, which is addressed in Attachment 1 to this letter.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,


M. P. Alexich
Vice President
9/19/86

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Attachment

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Bruchmann
G. Charnoff
NRC Resident Inspector - Bridgman

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bc: J. G. Feinstein/M. W. Evarts
S. H. Horowitz/T. O. Argenta/R. C. Carruth
J. J. Markowsky/S. H. Steinhart/J. A. Kobyra
R. W. Jurgensen
R. F. Kroeger
M. L. Horvath - Bridgman
E. A. Morse - Bridgman
C. A. Erikson
J. B. Shinnock
D. L. Wigginton, NRC - Washington, D.C.
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ATTACHMENT 1 TO AEP:NRC:1007

NRC INSPECTION REPORT 50-315/86027; 50-316/86027

NRG Violation

"Unit 1 Technical Specification 3.3.2.1, specifies that the Engineered Safety Feature Actuation System instrumentation channels shall be response-timed as shown in Table 3.3-5. TS Table 3.3-5 requires that the turbine trip reactor trip ESF activation on steam generator high-high water level have an overall response time of less than or equal to 2.5 seconds. TS definition 1.23, 'Engineered Safety Feature Response Time,' states in part that the response time 'shall be that time interval from when the monitored parameter exceeds its ESF actuation setpoint at the channel sensor until the ESF equipment is capable of performing its safety function.'

Contrary to the above, prior to July 10, 1986, the licensee did not adequately response time test the Unit 1 steam generator water level high-high level turbine trip reactor trip, in that, response time was not measured through the turbine trip logic (i.e., the licensee did not fully test the time interval from the channel sensor to the ESF equipment).

The licensee demonstrated that the overall response time, was less than TS Table 3.3-5 requirement of 2.5 seconds on July 10, 1986."

Response to NRG Violation

Corrective Action Taken and Results Achieved

On July 10, 1986, following the sequence of events described in LER 315/86-007 (see Attachment 2), it was determined that the time-response test procedure did not fully test the complete circuitry for the Unit 1 steam generator high-high water level turbine trip-reactor trip. Therefore, a special procedure was written and a time response test was conducted on that portion of the circuitry not included in the above surveillance procedure. The test included that portion of the circuit from the output of the turbine trip control master relay to the closure of the steam stop valves. In addition, the time response from the steam stop valve closure to reactor trip was determined from an actual trip. It was found that the total response time for the turbine trip-reactor trip from a steam generator high-high water level was within technical specification limits.

Corrective Action To Be Taken To Avoid Further Violation

We will revise the appropriate time response testing procedures prior to the end of the Unit 1 1987 refueling outage to accomplish the testing of this circuitry as required by the Technical Specifications.

Date When Full Compliance Will Be Achieved

Full compliance was achieved on July 10, 1986, when the response time testing was completed.

ATTACHMENT 2 TO AEP:NRC:1007

LER 315/86-007 REV. 1