

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8601080630 DOC. DATE: 85/10/31 NOTARIZED: NO. DOCKET #
 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
 AUTH. NAME AUTHOR AFFILIATION
 CZAJKA Indiana & Michigan Electric Co.
 SVENSSON, B. A. Indiana & Michigan Electric Co.
 SMITH, W. G. Indiana & Michigan Electric Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: Monthly operating rept for Oct 1985. W/851108 1tr.

DISTRIBUTION CODE: IE24L COPIES RECEIVED: LTR L ENCL L

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TITLE: Operating Rept (50 DKT)-Annual/Semiannual/Monthly

NOTES:

05000316

OL: 12/23/72

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N.R.C. OPERATING DATA REPORT

DOCKET NO. 50-316
 DATE 11/04/85
 COMPLETED BY CZAJKA
 TELEPHONE 616-465-5901

OPERATING STATUS

1. Unit Name D. C. Cook Unit 2
 2. Reporting Period OCT 85 notes
 3. Licensed Thermal Power (MWt) 3411
 4. Name Plate Rating (Gross MWe) 1133
 5. Design Electrical Rating (Net MWe) 1100
 6. Maximum Dependable Capacity (GROSS MWe) 1100
 7. Maximum Dependable Capacity (Net MWe) 1060
 8. If Changes Occur in Capacity Ratings (Items no. 3 through 7) Since Last Report Give Reasons

9. Power Level To Which Restricted. If Any (Net MWe)
 10. Reasons For Restrictions. If Any:

	This Mo.	Yr. to Date	Cumm.
11. Hours in Reporting Period	745.0	7396.0	68664.0
12. No. of Hrs. Reactor Was Critical	158.0	4665.6	47745.6
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator on Line	141.8	4589.1	46587.7
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Therm. Energy Gen. (MWH)	163249	14800561	147878622
17. Gross Elect. Energy Gen. (MWH)	42250	4813770	48599200
18. Net Elect. Energy Gen. (MWH)	37369	4641531	46859265
19. Unit Service Factor	19.0	62.0	70.4
20. Unit Availability Factor	19.0	62.0	70.4
21. Unit Capacity Factor (MDC Net)	4.7	59.2	66.8
22. Unit Capacity Factor (DER Net)	4.6	57.1	65.4
23. Unit Forced Outage Rate	81.0	36.6	15.9
24. Shutdowns Scheduled over Next Six Months (Type, Date, and Duration):			

REFUELING SCHEDULED FOR FEBRUARY 1986

25. If Shut Down At End of Report Period, Estimated Date of Startup:
 UNIT SHUTDOWN FOR TRIP BREAKER ANALYSIS. Unit returned to service 11/8/85.
 26. Units in Test Status (Prior to Commercial Operation):

Forecast Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

8601080630 851031
 PDR ADOCK 05000316
 R PDR

IE24
 1/1

AVERAGE DAILY POWER LEVEL (MWe-Net)

DOCKET NO. 50-316
 UNIT TWO
 DATE 11/04/85
 COMPLETED BY CZAJKA
 TELEPHONE 616-465-5901

MONTH OCT 85

DAY	AVERAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	59
8	0	24	209
9	0	25	211
10	0	26	212
11	0	27	214
12	0	28	316
13	0	29	336
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1985

DOCKET NO. 50-316
 UNIT NAME D.C. Cook, Unit 2
 DATE 11/8/85
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 1 of 2

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
164	850824	F	545.2	A&B	1	N.A.	CC	HTEXCH	The Unit was removed from service on 850824 for steam generator tube leak repairs and to perform required design changes. The Unit was returned to service on 851023 at 1712 hours EDT. During power ascension from the previous outage, a reactor trip occurred from 79% power. The trip was triggered by a spurious indicated low-flow condition in R.C. Loop 2 due to a momentary drop in the output voltage from vital instrument bus, CRID II. The voltage drop is believed to have been caused by a component failure in radiation monitor, ERS-2300. Following the reactor trip it was discovered that reactor trip breaker "A" failed to open. The trip was accomplished through reactor trip
165	851029	F	58.0	A	3	(Later)	ZZ	ZZZZZZ	

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

4
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

5
 Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1985

DOCKET NO. 50-316
 UNIT NAME D.C. Cook, Unit 2
 DATE 11/8/85
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 2 of 2

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
165	(Continued)								breaker "B". The Unit remained shut down at the end of the month (RCS in Mode 3) pending resolution of the reactor trip breaker problem.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

4
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
 Exhibit I - Same Source

(9/77)

Docket No.: 50-316
Unit Name: D.C. Cook Unit 2
Completed By: D. A. Bruck
Telephone: (616) 465-5901
Date: November 11, 1985
Page: 1 of 2

MONTHLY OPERATING ACTIVITIES - OCTOBER, 1985

HIGHLIGHTS:

The reporting period began with the Reactor Coolant System in Mode 5. On 10-13-85 at 2132, the Reactor Coolant System entered Mode 4. On 10-19-85 at 0616, the Reactor Coolant System entered Mode 3. On 10-23-85 at 0033, the Reactor entered Mode 2 becoming critical at 0102. On 10-23-85 at 1506, the Reactor entered Mode 1 and at 1712, the Unit was paralleled to the system. On 10-23-85 at 2350, with reactor power at 23%, a 3%/hr increase was begun. The power increase was stopped at 2215 with power at 29%. On 10-28-85 at 0945, a power increase of 3%/hr to 50% was begun. The increase was stopped at 1640 with power at 49% due to a chemistry hold. On 10-29-85 at 0345, a power increase to 80% was begun. At 1357 a reactor trip occurred from 79% power. The trip was triggered by a spurious indicated low flow condition in R.C. Loop 2, due to a momentary dip in the output voltage from vital instrument bus, CRID II. The voltage drop is believed to have been caused by a component failure in radiation monitor, ERS-2300. Following the reactor trip it was discovered that reactor trip breather "A" failed to open. The trip was accomplished through reactor trip breaker "B".

The reporting period ended with the Reactor Coolant System in Mode 3.

Total electrical generator for the month of October was 42,250 MWE.

SUMMARY:

10-13-85 At 2132, the Reactor Coolant System entered Mode 4.

10-19-85 At 0616, the Reactor Coolant System entered Mode 3.

At 0831, an Engineered Safety Features actuation resulted from a high alarm on containment particulate radiation monitor, ERS-2301.

At 0850, an Engineered Safety Features actuation resulted from a high alarm on containment particulate radiation monitor, ERS-2401.

10-22-85 At 1441, an Engineered Safety Feature actuation resulted from a high alarm on containment area radiation monitor, VRS-2201.

Docket No.: 50-316
Unit Name: D.C. Cook Unit 2
Completed By: D. A. Bruck
Telephone: (616) 465-5901
Date: November 11, 1985
Page: 2 of 2

10-23-85 At 0102, the Reactor was critical.

At 1506, the Reactor entered Mode 1.

At 1712, the Unit was paralleled to the system.

At 2008, with power at 23%, a 3%/hr increase was started.

At 2215, reactor power increase was stopped at 29%.

At 2350, an Engineered Safety Features actuation resulted from a high alarm on the containment high range noble gas monitor, ERS-2309.

10-24-85 At 2358, an Engineered Safety Features actuation resulted from a high alarm on the containment particulate radiation monitor, ERS-2401.

10-27-85 AT 0910, an Engineered Safety Features actuation resulted from a high alarm on the containment particulate radiation monitor, ERS-2301.

10-28-85 At 0945, a power increase to 50% at 3%/hr was begun.

At 1640, the power increase was stopped at 49% because of secondary chemistry.

10-29-85 At 0345, a power increase to 80% was begun.

At 1357, a reactor trip occurred from 79% power. The trip was triggered by a spurious indicated low flow condition in R.C. Loop 2, due to a momentary dip in the output voltage from vital instrument bus, CRID II. The voltage drop is believed to have been caused by a component failure in radiation monitor, ERS-2300. Following the reactor trip it was discovered that reactor trip breater "A" failed to open. The trip was accomplished through reactor trip breaker "B".

DOCKET NO.	50 - 316
UNIT NAME	D. C. Cook - Unit No. 2
DATE	11-8-85
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901
PAGE	1 of 1

MAJOR SAFETY-RELATED MAINTENANCE

OCTOBER, 1985

- M-1 2-NRV-152, Pressurizer Power Operated Relief Valve leaking by. Lapped seats and replaced gaskets.
- C&I-1 Unit 2 AB diesel engine lube oil level was indicating almost zero level with tank almost full. Level indicator was repaired and calibrated.
- C&I-2 Amphenol triaxial cable bulkhead connectors for the Source Range and Intermediate Range Nuclear Instrumentation Systems were inspected in accordance with Westinghouse Technical Bulletin NSID-TB-84-12. The right connector on Source Range Channel N36 was found faulty and was replaced.
- C&I-3 BLP-122, S/G Level Protection Set III Indicator was indicating erratically. The reference leg to BLP-122 was found to have been nicked with a grinder. A patch was welded at the nick and the indicator was verified operable.
- C&I-4 A calibration check of #2 hydrogen recombiner thermocouples was required. The heater bank was removed and thermocouples were re-inspected. Heater bank was re-installed and thermocouple continuity was verified.
- C&I-5 CFA-454, CCW flow to #24 RCP lower motor bearing was stuck at 3 gpm and would not change when the outlet throttle valve was stroked. The strain gauge was replaced and range resistors from the circuit board were removed. Transmitter was calibrated and system was verified for proper operation.
- C&I-6 CFI-451, CCW flow to #24 RCP upper oil cooler was reading off scale high. The transmitter amplifier was found to be bad. The amplifier board was replaced and the loop was calibrated.



INDIANA & MICHIGAN ELECTRIC COMPANY

Donald C. Cook Nuclear Plant
P.O. Box 458, Bridgman, Michigan 49106

November 8, 1985

Director, Office Of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 2
Technical Specification 6.9.1.6, the attached Monthly Operating Report
for the Month of October, 1985 is submitted.

Sincerely,

W. G. Smith, Jr.
W. G. Smith, Jr.
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan
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IE24
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