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AUTH.NAME AUTHOR AFFILIATION
ALEXICH,M.P. Indiana & Michigan Electric Co.
RECIP.NAME RECIPIENT AFFILIATION
DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Proposes schedule for meeting 10CFR50.62 requirements re ATWS rule & provides brief description of design. Util expects to implement ATWS mitigating sys actuation circuitry during refueling outage in early 1987.

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INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

October 24, 1985
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Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
IMPLEMENTATION SCHEDULE FOR AMSAC PER 10 CFR 50.62

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

On June 1, 1984 the Commission approved publication of 10 CFR 50.62, regarding the reduction of risk from anticipated transients without scram (ATWS) events for light-water cooled nuclear power plants. 10 CFR 50.62(d) requires each licensee to develop and submit to the Director of the Office of Nuclear Reactor Regulation a proposed schedule for meeting the requirements of the rule. The purpose of this letter is to provide a proposed schedule for meeting the requirements of the ATWS rule and to provide a brief description of the design being considered to meet the requirements of 10 CFR 50.62(c)(1) as it applies to Donald C. Cook Nuclear Plant Unit Nos. 1 and 2.

It is our belief that 10 CFR 50.62(c)(1) requires each pressurized water reactor (PWR) to have equipment from sensor output to, but not including, the final actuation device, that is diverse from the reactor trip system, to automatically initiate the auxiliary feedwater system and to initiate a turbine trip under conditions indicative of an ATWS. In a letter from L. D. Butterfield to C. O. Thomas dated July 25, 1985 (OG-156), the Westinghouse Owners Group (WOG) submitted Topical Report WCAP-10858, "AMSAC Generic Design Package," which describes three conceptual designs that meet this requirement at Westinghouse-designed PWRs. AEPSC is a member of the WOG and participated in the development of the Topical Report.

We expect to be able to implement AMSAC using existing reactor trip system sensors during the Unit 1 refueling outage in early 1987 provided we receive the pertinent favorable Safety Evaluation (SE) by November 30, 1985. Unit 2 implementation will follow at the next planned Unit 2 outage after implementation on Unit 1. It is also noted that this schedule is predicated based on your approval of our plant-specific design by July 30, 1986. We presently anticipate sending you a plant-specific design review package by May 1, 1986.

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This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

M. P. Alexich
Vice President

RBK
10/23/85

cm

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Bruchmann
G. Charnoff
NRC Resident Inspector - Bridgman