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AUTH.NAME AUTHOR AFFILIATION
ALEXICH,M.P. Indiana & Michigan Electric Co.
RECIP.NAME RECIPIENT AFFILIATION
DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards revised Tech Specs, changing Tables 2.2-1, 3.2-1 & 3.3-3 re RdF resistance temp detector (RTD) & two Westinghouse proprietary. Safety evaluations withheld (ref 10CFR2.790).

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INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

August 13, 1985
AEP:NRC:0942D

Donald C. Cook Nuclear Plant Unit No. 1
Docket No. 50-315
License No. DPR-58
ADDITIONAL INFORMATION FOR RdF RTD TECHNICAL SPECIFICATIONS

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

This letter and its attachments provide the numerical values and the safety evaluation for the Technical Specification changes identified in our letter dated July 30, 1985, identifier AEP:NRC:0942A. Attachment 1 is the proprietary version of the safety evaluation for these values as determined by our NSSS vendor, Westinghouse Electric Corporation. The revised Technical Specification pages are included as Attachment 2. Attachment 3 is the Westinghouse Application for Withholding Proprietary Information from Public Disclosure, CAW-85-051, accompanying affidavit, and Proprietary Information Notice for the safety evaluations included in Attachment 1.

As previously indicated, we request approval of this amendment in time to support Unit 1 startup. These Technical Specification changes will have to be in place two days before initial criticality.

Attachment 4 is a Westinghouse safety evaluation for operation during the interval between our obtaining the RTD cross-calibration data and the incorporation of the resulting corrections into the instrumentation. Attachment 5 is the Westinghouse Application for Withholding Proprietary Information from Public Disclosure, CAW-85-053, accompanying affidavit, and Proprietary Information Notice for the safety evaluations included in Attachment 4.

The non-proprietary versions of the two safety evaluations, identified as Attachments 1 and 4 above, will be sent to the U.S. Nuclear Regulatory Commission under a separate letter.

It is to be noted that in our July 30 letter, we agreed to maintain the Tavg safety signals in the tripped condition when operating above the P-12 setpoint, if we were granted the Technical Specification changes associated with Table 3.3-3 and its notes. Subsequent to that, we were able to perform a

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safety analysis (Attachment 4) which we believe demonstrates that placing the Tavg safety signals in the tripped condition is not required for the protection of public health and safety. However, discussions with your staff have indicated that they would like the requirement to remain in conjunction with the use of the proposed footnote to Table 3.3-3 during the current startup. We agree to this commitment for the startup of Unit 1 Cycle 9 only, in order to provide your staff sufficient time to review the safety analysis in Attachment 4.

As this submittal contains information proprietary to Westinghouse Electric Corporation, it is supported by two affidavits signed by Westinghouse, the owner of the information. The affidavits set forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b)(4) of Section 2.790 of the Commission's regulations.

Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.790 of the Commission's regulations. Correspondence with respect to the proprietary aspects of the Applications for Withholding or the supporting Westinghouse affidavits should reference CAW-85-051 or CAW-85-053 and should be addressed to R. A. Wiesemann, Manager Regulatory and Legislative Affairs, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230.

Based upon the Westinghouse evaluations and our initial reviews of those evaluations, we believe that the proposed changes will not result in 1) a significant change in the types of effluents or a significant increase in the amounts of any effluent that may be released offsite, or 2) a significant increase in individual or cumulative occupational radiation exposure and thus do not involve a significant hazards consideration as defined by 10 CFR 50.92.

These proposed changes will be reviewed by the Plant Nuclear Safety Review Committee and the Nuclear Safety and Design Review Committee at their next regularly scheduled meetings.

In compliance with the requirements of 10 CFR 50.91(b)(1), a copy of this letter and its non-proprietary attachments will be transmitted to Mr. R.C. Callen of the Michigan Public Service Commission and Mr. G. Bruchmann of the Michigan Department of Public Health.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

M. P. Alexich
Vice President 8/12/85

/bjs

Attachments

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
G. Bruchmann
R. C. Callen
G. Charnoff
NRC Resident Inspector - Bridgman