

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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SUBJECT: Special rept re fire protection sys. Grinnell Multi-Matic  
 deluge valves protecting Auxiliary Bldg 609 elevation cask  
 handling & 587 elevation drumming room did not satisfy  
 surveillance test. Sys declared inoperable in order to test.

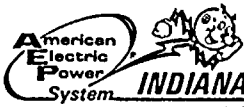
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**INDIANA & MICHIGAN ELECTRIC COMPANY**

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November 28, 1984

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Document Control Manager:

This special report is being submitted to describe two similar events relating to fire protection systems at Donald C. Cook Nuclear Plant. The Grinnell Multi-Matic deluge valves protecting the Auxiliary Building 609' elevation cask handling area and the 587' elevation drumming room did not satisfy surveillance test procedure \*\*120HP4030.STP.120 acceptance criteria which require the deluge valves to open upon actuation of their respective remote actuation valves. The remote actuation valves bleed off detection line air, simulating the actuation of a single detection head in the pre-action air system.

Functional testing of deluge valves was initiated subsequent to an I.E. inspection which took place during the summer of 1981. The surveillance test procedure drafted to perform deluge valve testing was based upon manufacturer's literature which was felt at the time to provide adequate guidance. Review of I.E. Information Notice 84-16 resulted in improvements to the test procedure to include actual opening of the deluge valve utilizing the actual preaction air line remote actuation valve. The valves were tested in March of 1984 and satisfied the requirements of the then current version of \*\*120HP4030.STP.120 (Rev. 7). At approximately this same time research was initiated preparatory to the revision of STP.120. This research resulted in the discovery that the preaction air supply and diaphragm chamber system lines to the actuator had been isolated, prior to the simulated actuation via the remote preaction air valve, in all previous revisions of the surveillance test.

Revision 8 of \*\*120HP4030.STP.120 was first utilized on October 14, 1984 to test the cask handling area deluge system. The procedural acceptance requirements were not satisfied and the system was declared inoperable. Similar results were obtained on October 16 during the test of the drumming room system and it was also declared inoperable. A detailed investigation of the systems involved in the events resulted in the following:

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- It was discovered that flow control orifices were not originally installed in the preaction air supply to the B-1 actuator. Piston check valves were found installed in the preaction air supply line to the B-1 actuators, providing partial flow control.
- The regulator for the preaction air supply had been set at approximately 60 psi. This setting should not have affected operability had the .050" inch flow control orifices been installed.
- Corrosion products were found in the B-1 actuator.

Isolation of the preaction air supply line to the actuator in previous revisions of the surveillance test procedure had masked indications that the flow control orifices were not in place. Flow control orifices and corrosion-resistant actuators are being obtained from the manufacturer. The installed actuators have been cleaned and inspected and the preaction air supply regulator has been set at 30 psi. The 30 psi setting allows adequate bleed-off of preaction air, upon simulated actuation, to cause the actuator to activate the deluge valve. The systems have been tested and have met the surveillance test acceptance criteria. Installation of the flow orifices will provide continued assurance of acceptable operation.

Though the two systems had been declared inoperable for failing to meet the acceptance criteria of the revised test procedure, the two systems would have actuated had the systems been challenged. Indicated drops in detection line pressure during testing indicate that 2 - 3 preaction detection heads would have had to actuate to drop the preaction air supply pressure sufficiently to trip the respective deluge valve but the systems at all times retained the capability to perform their design function.

This report is being submitted pursuant to the requirements of Technical Specification 3.7.9.2 Action Statement (a) because the systems were declared inoperable in excess of 14 days to permit troubleshooting and testing. The systems were returned to operable status on November 7, 1984 (cask handling area) and November 14, 1984 (drumming room).

Sincerely,

  
W. G. Smith, Jr.  
Plant Manager

/mmf

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1. The first step is to identify the problem. This involves understanding the current situation and what needs to be changed.

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November 28, 1984

DOCKET NO(S). 50-3151316

Mr. John Dolan, Vice President  
Indiana and Michigan Electric Company  
c/o American Electric Power  
Service Corporation  
1 Riverside Empatation  
Columbus, Ohio 43215

SUBJECT:

D.C. COOK POWER PLANT UNITS 1 AND 2

The following documents concerning our review of the subject facility are transmitted for your information.

- ☐ Notice of Receipt of Application, dated \_\_\_\_\_.
- ☐ Draft/Final Environmental Statment, dated \_\_\_\_\_.
- ☐ Notice of Availability of Draft/Final Environmental Statement, dated \_\_\_\_\_.
- ☐ Safety Evaluation Report, or Supplement No. \_\_\_\_\_, dated \_\_\_\_\_.
- ☐ Notice of Hearing on Application for Construction Permit, dated \_\_\_\_\_.
- ☐ Notice of Consideration of Issuance of Facility Operating License, dated \_\_\_\_\_.
- ☐ Monthly Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated \_\_\_\_\_.
- ☐ Application and Safety Analysis Report, Volume \_\_\_\_\_.
- ☐ Amendment No. \_\_\_\_\_ to Application/SAR dated \_\_\_\_\_.
- ☐ Construction Permit No. CPPR- \_\_\_\_\_, Amendment No. \_\_\_\_\_ dated \_\_\_\_\_.
- ☐ Facility Operating License No. \_\_\_\_\_, Amendment No. \_\_\_\_\_, dated \_\_\_\_\_.
- ☐ Order Extending Construction Completion Date, dated \_\_\_\_\_.
- ☒ Other (Specify) Monthly Notice covering period November 21, 1984. Expiration date  
for hearing requests and comments December 21, 1984

Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures:  
As stated

cc: w/enclosure

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DATE	11/28/84						

1. The first part of the document discusses the importance of maintaining accurate records of all activities. It emphasizes that this is essential for ensuring the integrity and reliability of the information collected.

2. The second part of the document outlines the procedures for collecting and analyzing data. It details the steps involved in gathering information from various sources and the methods used to process and interpret the results.

3. The third part of the document describes the various types of data that are collected and the methods used to analyze them. It includes a detailed discussion of the different types of information that are gathered, such as intelligence, scientific data, and other relevant information.

4. The fourth part of the document discusses the importance of maintaining the confidentiality of the information collected. It outlines the measures that are taken to ensure that the data is kept secure and that it is only accessible to those who are authorized to view it.

5. The fifth part of the document discusses the importance of maintaining the accuracy of the information collected. It outlines the measures that are taken to ensure that the data is reliable and that it is only used for the purposes for which it was collected.

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