REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8410230067 DOC.DATE: 84/10/19 NOTARIZED: NO DOCKET #
FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
AUTH.NAME AUTHOR AFFILIATION
ALEXICH,M.P. Indiana & Michigan Electric Co.
RECIP.NAME RECIPIENT AFFILIATION
DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Application for amends to Licenses DPR-58 & DPR-74,revising Tech Specs to reflect administrative changes.Fee paid.

DISTRIBUTION CODE: A001D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 5+18
TITLE: OR Submittal: General Distribution

NOTES: 05000315
OL:10/25/74
w/o Check 05000316
OL:12/23/72

	RECIPIENT ID CODE/NAME		COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME		COPIES LTTR	COPIES ENCL
	NRR ORB1 BC	01	7	7				
INTERNAL:	ADM/LFMB		1	0	ELD/HDS3		1	0
	NRR/DE/MTEB		1	1	NRR/DL DIR		1	1
	NRR/DL/ORAB		1	0	NRR/DSI/METB		1	1
	NRR/DSI/RAB		1	1	REG FILE	04	1	1
	RGN3		1	1				
EXTERNAL:	ACRS	09	6	6	LPDR	03	2	2
	NRC PDR	02	1	1	NSIC	05	1	1
	NTIS		1	1				

TOTAL NUMBER OF COPIES REQUIRED: LTTR 27 ENCL 24

INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

October 19, 1984
AEP:NRC:0433D

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
TECHNICAL SPECIFICATION CHANGE REQUEST

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Submittal No. AEP:NRC:0433 dated May 10, 1983.

Dear Mr. Denton:

This letter and its Attachments constitute an application for amendment to the Appendix 'A' Technical Specifications for the Donald C. Cook Nuclear Plant Unit Nos. 1 and 2. Attachment No. 1 is the description of the changes. Attachment No. 2 contains the revised Technical Specification pages. All changes have been indicated by a vertical line on the page. The purpose of these changes is to supplement our previous submittal AEP:NRC:0433 referenced above.

Our evaluation of the content of this Technical Specification change requests proposed by this letter is provided in Attachment No. 1. Many of the changes are administrative. Of those that are not, we believe all are of the type where the change may result in some increase to the probability or consequences of a previously analyzed accident, but the results of the change are clearly within criteria that are currently acceptable to the NRC. Based on our evaluation we have concluded that these changes will not involve a significant hazards consideration as defined in 10 CFR 50.92. In addition we believe that the proposed changes will not adversely impact the environment.

In compliance with the requirements of 10 CFR 50.91(b)(1), a copy of this letter and its attachments have been transmitted to Mr. R. C. Callen of the Michigan Public Service Commission.

These proposed Technical Specifications have been reviewed and approved by both the Plant Nuclear Safety Review Committee and the AEPSC Nuclear Safety and Design Review Committee.

A001 1/1 Rec'd w/out Check

Pursuant to 10 CFR 170.22, a check in the amount of $150.00 is attached for the NRC processing of these aforementioned requests. We request that the subject Technical Specification change requests be reviewed in conjunction with the above referenced submittal.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

M. P. Alexich
Vice President

RBK 10/17/84

MPA/cm

Attachments

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Charnoff
E. R. Swanson, NRC Resident Inspector - Bridgman

ATTACHMENT 1 TO AEP:NRC:0433D

TECHNICAL SPECIFICATION CHANGE DESCRIPTIONS

Change No. 1

Unit No. 1; Page 3/4 6-1

This change would add the proposed footnote to page 3/4 6-1 to make Technical Specifications between Unit 1 and Unit 2 consistent.

The change may result in either some increase to the probability or consequences of a previously analyzed accident or may reduce in some way a safety margin, but the results of the change are clearly within all criteria which are acceptable to the NRC as indicated in Revision 4 to the Standard Technical Specifications. In addition, acceptable operation with the proposed change has been demonstrated, as indicated by compliance with the Unit 2 Technical Specifications.

Change No. 2

Unit No. 1; Page 3/4 6-14 and 6-15

(a) In Section 4.6.3.1.1, items (a) and (b) have been replaced by a new paragraph and reference to 92 days testing of the operable valves has been deleted. The test frequency will be as per the ASME Code Section XI requirements when tested pursuant to specification 4.0.5. The testing requirements for each valve under specification 4.0.5 has been incorporated in a new section 4.6.3.1.3.

(b) In Section 4.6.3.1.2, items (d) and (e), which referred to cycling the valves one complete cycle, have been deleted and replaced by a new section No. 4.6.3.1.3. The new section notes that each valve in table 3.6-1 will be tested pursuant to specification 4.0.5.

This change would add consistency between the Technical Specification surveillance requirements in Units 1 and 2. This change would be administrative change since the ASME Section XI code requirement are similar to current Technical Specification requirements. A small number of valves, involved with item (a) above, will be tested every cold shutdown instead of once every 92 days, when tested pursuant to ASME code, Section XI requirements. The change may result in either some increase to the probability or consequences of a previously analyzed accident or may reduce in some way a safety margin, but the results of the change are clearly within all criteria which are acceptable to the NRC as indicated in Revision 4 to the Standard Technical Specifications. In addition acceptable operation, with the proposed changes has been demonstrated, as indicated by compliance with the Unit No. 2 Technical Specifications.

Change No. 3

Unit No. 1; Pages 3/4-6-16 through 6-22; Table 3.6-1

We propose the column entitled "Testable During Plant Operation" be deleted to be consistent with Unit No. 2 Technical Specifications. This would allow the valves in Table 3.6-1 to be tested pursuant to Technical Specification 4.0.5. Specification 4.0.5 requires that the in-service testing of ASME Code Class 1, 2 and 3 pumps and valves be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. This proposed change is considered an administrative change which would achieve consistency in the valve testing requirements and make the Unit No. 1 Technical Specification identical to that of Unit No. 2.

Change No. 4

Unit No. 2; Page 3/4 6-30; Table 3.6-1

We propose asterisks be added to valves SM-8 and SM-10 to denote that these valves may be opened on an intermittent basis under administrative controls. Administrative control of these valves would allow upper containment samples to be taken in MODES 1, 2, 3, and 4 without having personnel to access upper containment. Thus, our ALARA program would be further enhanced. This change would make Unit No. 1 Technical Specification consistent with that of Unit No. 2. Since this proposed change would allow the identical operating practices to that of Unit 1, which have been demonstrated as acceptable, we believe this change would not involve a Significant Hazards Consideration.

Change No. 5

Unit No. 2; Pages 3/4 6-28 and 6-31; Table 3.6-1

This change would delete valves listed in Table 3.6-1 as E6 (NSW-415-1), E7 (NSW-415-2), E8 (NSW-415-3), E9 (NSW-415-4), E10 (NSW-419-1), E11 (NSW-419-2), E12 (NSW-419-3), E13 (NSW-419-4), E14 (NSW-224-1), E15 (NSW-224-2), E16 (NSW-224-3), E17 (NSW-224-4), E32 (NSW-417-3), and E33 (NSW-417-4). We propose that the valves be renumbered sequentially (i.e., E6, E7, etc.) In Amendment No. 47 these valves were replaced by air operated isolation valves listed in Table 3.6-1 as B48 (WCR-900), B50 (WCR-904), B52 (WCR-908), B54 (WCR-912), B56 (WCR-920), B58 (WCR-924), B60 (WCR-928), B62 (WCR-932), B28 (WCR-941), B29 (WCR-942), B30 (WCR-943), B31 (WCR-944), B64 (WCR-960), and B66 (WCR-964). When the previous Technical Specification change request was submitted, the original designation (NSW) valves cited above was inadvertantly left in the Technical Specifications. This is considered to be an administrative change to correct this oversight.

Change No. 6

Unit No. 1; Table 3.6-1; Pages 3/4 6-17, 6-18, and 6-19
Unit No. 2; Table 3.6-1
Pages 3/4 6-17, 6-18, 6-19, 6-22, 6-23, 6-24, and 6-30.

We propose that the footnote in all of the above pages be deleted. The footnote references an effective time period prior to 1982 refueling outage. This proposed change is an administrative change, in that, a sentence in the Technical Specifications that is no longer applicable, would be deleted.