

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8404240174 DOC.DATE: 84/04/19 NOTARIZED: NO DOCKET #
 FACIL:50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
 AUTH.NAME AUTHOR AFFILIATION
 ALEXICH,M.P. Indiana & Michigan Electric Co.
 RECIP.NAME RECIPIENT AFFILIATION
 DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards response to second set of questions from BNL re
 Cycle 5 SAR,Exxon Nuclear Repts XN-NF-83-85 & XN-NF-83-85,
 Suppl 1,Rev 1.

DISTRIBUTION CODE: IE26S COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 3
 TITLE: Start-Up Report/ Refueling Report (50 Dkt)

NOTES:

	RECIPIENT ID CODE/NAME		COPIES LTTR ENCL		RECIPIENT ID CODE/NAME		COPIES LTTR ENCL
	NRR ORB1 BC 04		3 3				
INTERNAL:	IE FILE 01		1 1		NRR/DHFS DEPY07		1 1
	NRR/DHFS DIR 08		1 1		NRR/DHFS/PSRB11		1 1
	NRR/DSI/CPB 12		1 1		RM/DDAMI/MIB 09		1 1
EXTERNAL:	ACRS 13		5 5		LPDR 03		1 1
	NRC PDR 02		1 1		NSIC 05		1 1
	NTIS		1 1				

Original To: Reg. Files

TOTAL NUMBER OF COPIES REQUIRED: LTTR 19 ENCL 19

1. The first part of the document is a list of names and addresses of the members of the committee.

2. The second part of the document is a list of names and addresses of the members of the committee.

3. The third part of the document is a list of names and addresses of the members of the committee.

4. The fourth part of the document is a list of names and addresses of the members of the committee.

5. The fifth part of the document is a list of names and addresses of the members of the committee.

6. The sixth part of the document is a list of names and addresses of the members of the committee.

7. The seventh part of the document is a list of names and addresses of the members of the committee.

8. The eighth part of the document is a list of names and addresses of the members of the committee.

INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

April 19, 1984
AEP:NRC:0860D

Donald C. Cook Nuclear Plant Unit No. 2
Docket No. 50-316
License No. DPR-74
Cycle 5 Reload


Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

By this letter we transmit responses to the second set of verbal questions received from Brookhaven National Laboratory (BNL) regarding the Cycle 5 Safety Analysis Report (Exxon Nuclear Co. Report No. XN-NF-83-85 and XN-NF-83-85, Supp. 1, Rev. 1) that was submitted to the NRC in support of the Cycle 5 reload application.

This document has been prepared following corporate procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,


M.P. Alexich *EBK*
Vice President 4/18/84

MPA/bjs

cc: John E. Dolan
W.G. Smith, Jr. - Bridgman
R.C. Callen
G. Charnoff
E.R. Swanson, NRC Resident Inspector - Bridgman

8404240174 840419
PDR ADDCK 05000316
P PDR

IE26
1/1 *Original*
To: R. Swanson

1-1



1-1

1-1

1-1

1-1

1-1

1-1

1-1

Question #1 Is the 2% penalty on DNBR to account for the hydraulically different fuel assemblies in the core taken into account?

Answer #1 The 2% penalty on DNBR has been taken into account in the Cycle 5 transient analyses. Documentation regarding this is a letter from Exxon Nuclear Co. and is attached herewith.

Question #2 What is the value of F_Q used in the transient analyses?

Answer #2 F_Q used in the transient analyses is 2.47 as reported in Table 3.1 of XN-NF-82-32(P).

Question #3 It was reported that the ENC reload fuel will have less available thermal margin but the $F_{\Delta H}$ limit for ENC fuel is higher than the Westinghouse fuel. Why?

Answer #3 The Westinghouse $F_{\Delta H}$ and F_Q limits for Cycle 4 were reduced (from their Cycle 3 values) slightly to allow for an increase in core power. This was done because a new plant specific LOCA analysis was not warranted for the Westinghouse fuel. The Exxon values represent new plant specific analysis for Cycles 4 and 5.

EXXON NUCLEAR COMPANY, Inc.

600 108th Avenue N.E. C 00777, Bellevue, Washington 98009, Telephone (206) 453-4300

April 12, 1984

ENC-AEP/0341

Mr. George John, Sr. Engineer
Indiana & Michigan Electric Company
c/o American Electric Power Service Corp.
One Riverside Plaza
Columbus, OH 43215

Dear George:

- Ref: 1) XN-NF-82-21(P), "Application of Exxon Nuclear Company PWR Thermal Margin Methodology to Mixed Core Configurations," March 1982
- 2) XN-NF-82-32(P), Rev. 1, "Plant Transient Analysis for the Donald C. Cook Unit 2 Reactor at 3425 MWt, "October 1982
- 3) XN-NF-82-32(NP), Rev. 2, "Plant Transient Analysis for the Donald C. Cook Unit 2 Reactor at 3425 MWt: Operation with 5% Steam Generator Tube Plugging," March 1984

The Reference 1 methodology has been employed in the evaluation of MDNHR in D. C. Cook Unit 2 Cycle 4 and Cycle 5 plant transient analyses (References 2 and 3). The NRC Staff Safety Evaluation Report on the reference thermal-hydraulic methodology report mandates the application of a 2% MDNHR penalty for mixed cores analyzed with the reference methodology.

This is to confirm that the penalty was indeed applied in the Reference 2 and 3 analysis in accordance with the Staff SER.

Very truly yours,



H. C. Shaw

Contract Administrator

tlm

cc: M. P. Alexich
J. M. Cleveland
W. L. Zimmermann

RAPIFAX NO.

526

PAGE

1 OF 1

