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 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
 AUTH. NAME AUTHOR AFFILIATION
 ALEXICH, M.P. Indiana & Michigan Electric Co.
 RECIP. NAME RECIPIENT AFFILIATION
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Updates status of emergency operating procedures submitted
 830802. Latest target & commitment dates provided on Table
 1. Westinghouse will complete procedure write up by Jul 1984.

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Office of the Director of National Security
Washington, D.C. 20505
January 1, 1964
Mr. J. Edgar Hoover
Federal Bureau of Investigation
Washington, D.C. 20535
Dear Mr. Hoover:

Enclosed for you are two copies of a letterhead memorandum (LHM) dated and captioned as above. The LHM is being prepared by the Office of the Director of National Security (ODNS) in accordance with the provisions of the National Security Act of 1949, as amended, and the National Security Agency (NSA) is the lead agency in the preparation of this LHM.

Very truly yours,
W. Mark Felt
Director, ODNS

Enclosure

1. The Commission has received information from the Department of Health and Human Services, Office of the Assistant Secretary for Health, that the Department is currently conducting a study of the health care needs of the elderly. The study is being conducted in cooperation with the National Academy of Medicine and the National Academy of Sciences. The Commission is interested in the results of this study and would like to receive a copy of the report when it is available.

1111: ON 24th April 1964
1111: ON 24th April 1964

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[illegible][illegible]

INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

January 30, 1984

AEP:NRC:0773D

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket No. 50-315 and 50--316
License No. DPR-58 and DPR-74
Status Report on Emergency Operating Procedures Upgrade

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. NRC
Washington, D.C. 20555

Dear Mr. Denton:

This letter is to update the status of the Emergency Operating Procedures submitted in our letter AEP:NRC:773A dated August 2, 1983. The attached Table 1 updates the latest target and commitment dates, and thus supercedes similar information we transmitted to you in AEP:NRC:0773A.


We have currently contracted with Westinghouse Electric Corporation to write our Emergency Operating Procedures. They have advised us that they will be unable to complete this writeup until July 1984. The specific schedule for writing procedures is included in Figure 1. The corresponding list of the Emergency Operating Procedure is included in Table 2.

Following completion of the procedures, it will be necessary to perform plant specific validation and verification. This will be done during the period from July 1984 to September 1984. Operator training in these procedures is scheduled to be completed in October 1985.

This document has been prepared following Corporate Procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

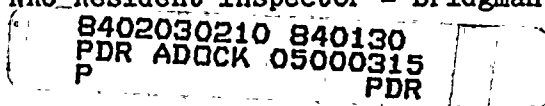
RBK
1/30/84


M. P. Alexich
Vice President

MPA/th

Attachment

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Charnoff
E. R. Swanson, NRC Resident Inspector - Bridgman



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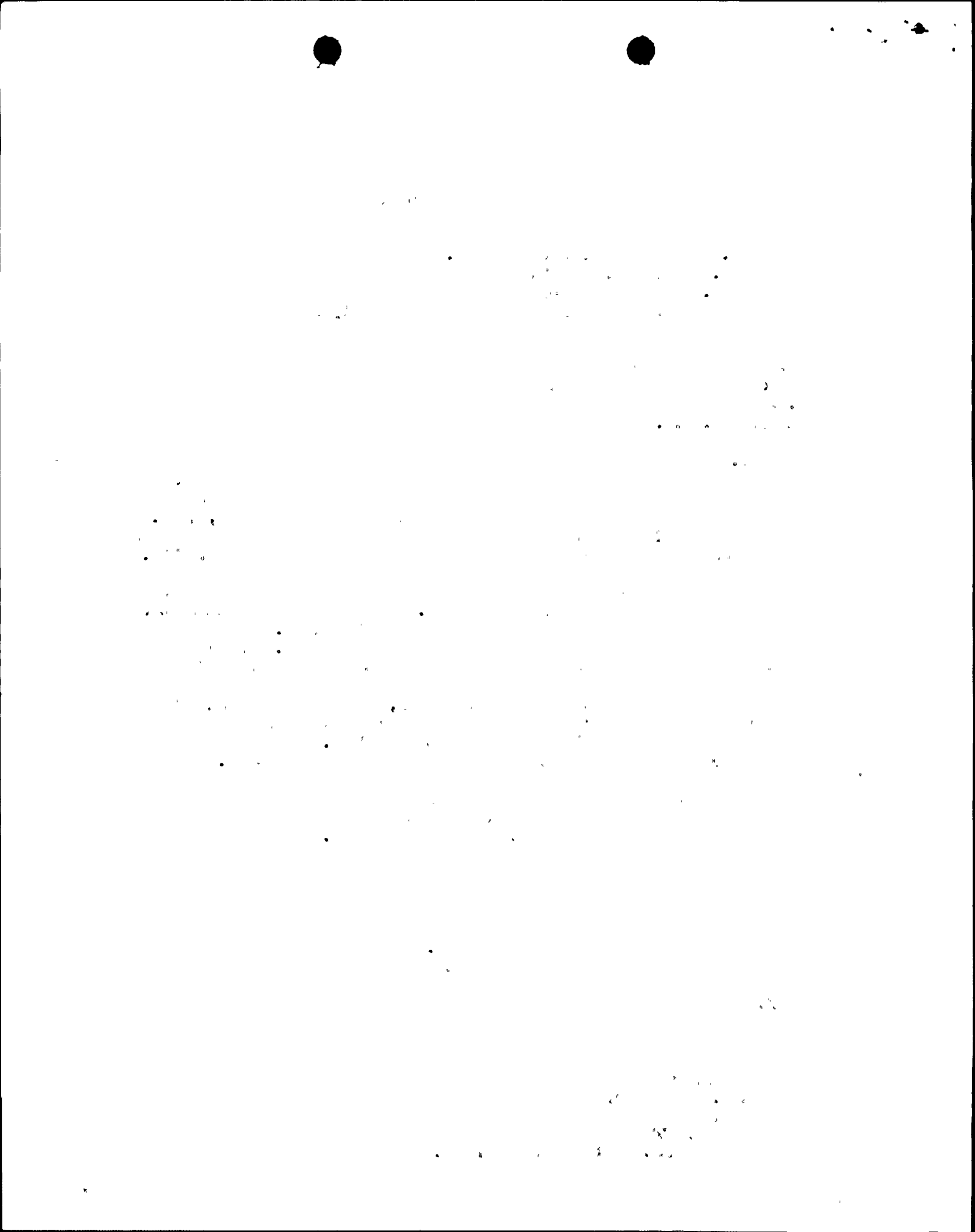


TABLE 1

COMMITMENT AND TARGET DATES FOR EMERGENCY RESPONSE ACTIVITIES

<u>I. Safety Parameter Display System, Emergency Operating Facility, and Technical Support Center</u>		
A. Status Report	Commitment:	April 1985
B. SPDS, EOF and TSC Functional (Operational)	Target:	September 1985
<u>II. Detailed Control Room Design Review</u>		
A. Program Plan Report	Commitment:	December 1983
B. Status Report	Commitment:	September 1984
C. Final Summary Report	Target:	December 1985
<u>III. Regulatory Guide 1.97, Post-Accident Instrumentation</u>		
A. Status Report on how we intend to meet Regulatory Guide 1.97 requirements	Commitment:	February 1985
B. Complete implementation phase of Regulatory Guide 1.97 work (MIDAS)	Target:	November 1985
<u>IV. Emergency Operating Procedures (upgrade)</u>		
A. Status Report	Commitment:	January 1984
B. Procedures Generation Package	Target:	July 1984
C. Validation & Verification	Target:	September 1984
D. EOP's Complete and Implemented	Target:	Training started October 1984 Training finished October 1985

TABLE (2)

EOP Procedures List

E-0: Reactor Trip Or Safety Injection

ES-0.0: Rediagnosis

ES-0.1: Reactor Trip Response

ES-0.2: Natural Circulation Cooldown

ES-0.3: Natural Circulation Cooldown With Steam Void In Vessel (With RVLIS)

ES-0.4: Natural Circulation Cooldown With Steam Void In Vessel (Without RVLIS)

E-1: Loss Of Reactor Or Secondary Coolant

ES-1.1: SI Termination

ES-1.2: Post LOCA Cooldown And Depressurization

ES-1.3: Transfer To Cold Leg Recirculation

ES-1.4: Transfer to Hot Leg Recirculation

E-2: Faulted Steam Generator Isolation

E-3: Steam Generator Tube Rupture

ES-3.1: Post-SGTR Cooldown Using Backfill

ES-3.2: Post-SGTR Cooldown Using Blowdown

ES-3.3: Post-SGTR Cooldown Using Steam Dump

ECA-0.0: Loss Of All ac Power

ECA-0.1: Loss Of All ac Power Recovery Without SI Required

ECA-0.2: Loss Of All ac Power Recovery With SI Required

ECA-1.1: Loss Of Emergency Coolant Recirculation

ECA-1.2: LOCA Outside Containment

ECA-2.1: Uncontrolled Depressurization Of All Steam Generators

ECA-3.1: SGTR With Loss Of Reactor Coolant-Subcooled Recovery Desired

ECA-3.2: SGTR With Loss Of Reactor Coolant-Saturated Recovery Desired

ECA-3.3: SGTR Without Pressurizer Pressure Control

TABLE (2) CONTINUED

EOP Procedures List

- FR-S.1: Response to Nuclear Power Generation/ATWS
- FR-S.2: Response to Loss of Core Shutdown

- FR-C.1: Response to Inadequate Core Cooling
- FR-C.2: Response to Degraded Core Cooling
- FR-C.3: Response to Saturated Core Cooling

- FR-H.1: Response to Loss of Secondary Heat Sink
- FR-H.2: Response to Steam Generator Overpressure
- FR-H.3: Response to Steam Generator High Level
- FR-H.4: Response to Loss of Normal Steam Release Capabilities
- FR-H.5: Response to Steam Generator Low Level

- FR-P.1: Response to Imminent Pressurizer Thermal Shock Condition
- FR-P.2: Response to Anticipated Pressurized Thermal Shock Condition

- FR-Z.1: Response to High Containment Pressure
- FR-Z.2: Response to Containment Flooding
- FR-Z.3: Response to High Containment Radiation Level

- FR-I.1: Response to High Pressurizer Level
- FR-I.2: Response to Low Pressurizer Level
- FR-I.3: Response to Voids in Reactor Vessel

FIGURE 1

D. C. COOK EMERGENCY OPERATING PROCEDURE SCHEDULE

