

# REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8310200158 DOC. DATE: 83/10/14 NOTARIZED: NO DOCKET #  
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316  
 AUTH. NAME: AUTHOR AFFILIATION  
 ALEXICH, M.P. Indiana & Michigan Electric Co.  
 RECIP. NAME: RECIPIENT AFFILIATION  
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Requests extension until 831215 to submit rept on  
 plant-specific evaluation of relief & safety valves &  
 associated piping sys, per NUREG-0737, Item II.D.1.2.

DISTRIBUTION CODE: A046S COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2  
 TITLE: OR Submittal: TMI Action Plan Rgmt NUREG-0737 & NUREG-0660

## NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	NRR ORB1 BC 01	7		
INTERNAL:	ELD/HDS3	1	IE/DEPER DIR 33	1
	IE/DEPER/EPB	3	IE/DEPER/IRB	1
	NRR PAWLSON, W.	1	NRR/DHFS/DEPY29	1
	NRR/DL DIR 14	1	NRR/DL/ADL 16	1
	NRR/DL/ORAB 18	3	NRR/DSI/ADRS 27	1
	NRR/DSI/AEB	1	NRR/DSI/ASB	1
	NRR/DSI/RAB	1	NRR/DST DIR 30	1
	<u>REG FILE</u> 04	1	RGN3	1
EXTERNAL:	ACRS 34	10	LPDR 03	2
	NRC PDR 02	1	NSIC 05	1
	NTIS 1	1		

TOTAL NUMBER OF COPIES REQUIRED: LTTR 42 ENCL 0



# INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631  
COLUMBUS, OHIO 43216

October 14, 1983  
AEP:NRC:0585G

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2  
Docket Nos. 50-315 and 50-316  
License Nos. DPR-58 and DPR-74  
NUREG-0737, Item II.D.1.2  
PWR SAFETY AND RELIEF VALVE TEST PROGRAM

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

References: (1) Letter No. AEP:NRC:0585C dated July 9, 1982  
(2) Letter No. AEP:NRC:0585F dated May 27, 1983

Dear Mr. Denton:

The purpose of this letter is to request an extension to the submittal date of the Plant Specific Evaluation of the Relief and Safety Valves and associated piping system. This extension would allow us to perform further analyses and investigate the possibility of alternate design modifications that could be implemented to meet the requirements of NUREG-0737.

In our letter of May 27, 1983 (AEP:NRC:0585F), we had requested an extension to the submittal date of the Plant Specific Evaluation from June 1, 1983 to September 15, 1983. In that letter we informed the NRC that Teledyne Engineering Services (TES) had performed a preliminary evaluation, of the as-built piping, based on elastic analysis. That evaluation showed that the discharge piping was overloaded during a safety valve transient and the resulting loop seal water discharge conditions.

The reason for considering alternate design modifications, as was discussed in telephone conversations with your staff, is that we recently learned that many Westinghouse PWR plants were able to retain the water in the loop seals and still meet the concerns of NUREG-0737, Item II.D.1.2 with relatively minor restraint modifications. We have added a drain line and supports, necessary to meet the regulatory concerns of NUREG-0737, on Unit No. 1.

As a first step in evaluating the possibilities of alternate modifications and in ensuring continued safe operation of Unit 2, we have authorized Teledyne to perform an analysis based on the inelastic behavior of the piping system. This analysis assumes water still is in the loop. Teledyne has performed a preliminary evaluation. Based

8310200158 831014  
PDR ADOCK 05000315  
P PDR

AP46  
10

100-100000

100-100000  
100-100000  
100-100000  
100-100000  
100-100000

100-100000  
100-100000  
100-100000  
100-100000  
100-100000

100-100000 (E)  
100-100000 (E)

100-100000

100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000

100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)  
100-100000 (E)

100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000

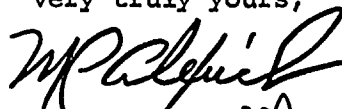
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000  
100-100000

upon that evaluation, Teledyne has indicated that should a safety valve actuate, the pipe between the pressurizer and the valve would not rupture and would not fail in such a way so as to restrict flow from the pressurizer. Further, the preliminary evaluation allowed Teledyne to conclude that the discharge piping downstream of the safety valve would not fail so as to restrict flow from the safety valve.

It is anticipated that Teledyne will complete their analysis by mid-November, 1983. We can then define our schedule for completion of review of the analysis, and define any modifications needed to meet the concerns associated with Item II.D.1.2 of NUREG-0737. Therefore, we are requesting an extension on all safety and relief valve qualification report submittal dates, until the additional analyses can be completed and details of the modifications are finalized. We anticipate that we can complete our review of the Teledyne report and submit the findings to you by December 15, 1983. This should allow us to complete the modifications as necessary during the next Unit 2 refueling outage.

This document has been prepared following corporate Procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,



M. P. Alexich  
Vice President

MPA/cam

cc: John E. Dolan  
W. G. Smith, Jr. - Bridgman  
R. C. Callen  
G. Charnoff  
E. R. Swanson, NRC Resident Inspector - Bridgman  
D. Boyd - NRC Region III

