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50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
AUTH.NAME AUTHOR AFFILIATION
HUNTER,R.S. Indiana & Michigan Electric Co.
RECIP.NAME RECIPIENT AFFILIATION
DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Requests extension of submittal date of plant-specific safety & relief valve qualification & plant-specific piping support evaluation per NUREG-0737 Item II.D.1 until 830301.

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INDIANA & MICHIGAN ELECTRIC COMPANY

P. O. BOX 18
BOWLING GREEN STATION
NEW YORK, N. Y. 10004

July 9, 1982
AEP:NRC:0585C

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
NUREG-0737, ITEM II.D.1
PWR RELIEF & SAFETY VALVE TEST PROGRAM

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

References:
(1) NRC letter from Mr. D. G. Eisenhut dated September 29, 1981

(2) Letter No. AEP:NRC:0585B dated April 7, 1982

(3) WCAP 10105,"Safety Valve Operability Report"

(4) Letter from Mr. R. C. Youngdahl to Mr. H. R. Denton "Status of EPRI PWR Safety and Relief Valve Test Program, NUREG-0737, Item II.D.1." dated July 24, 1981

(5) Letter from Mr. R. C. Youngdahl to Mr. H. R. Denton on "Submittal of PWR Valve Data Package" dated June 1, 1982

Dear Mr. Denton:

This letter and its Attachment address the plant specific responses on PWR Relief and Safety Valve Test Program and respond to the three action items noted in Reference 1. The following are the Indiana & Michigan Electric Company's responses as applicable to the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2.

The Electric Power Research Institute (EPRI) has completed the relief and safety valve test program and the plant specific evaluations

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[illegible]

[illegible]
[illegible]
[illegible]
[illegible]
[illegible]
[illegible]

Mr. [illegible]
Office of Cross Counter Acquisition
[illegible] Industry Commission
[illegible], D. C. 20555

Reference: (1) [illegible] Mr. [illegible] dated [illegible] 1983

(2) Letter [illegible] dated April 7, [illegible]

(3) [illegible] 1012 "Safety Valve Operability [illegible]"

(4) Letter from Mr. [illegible] to Mr. [illegible] "Review of PTI's Safety and Relief Valve Test Program", MAG-[illegible], dated July [illegible], 1981

(5) Letter from Mr. [illegible] to Mr. [illegible] "Submittal of PTI Valve Data Package" dated June [illegible], 1982

Dear Mr. Denton:

This letter and its attach[illegible] provide the plant specific [illegible] information requested in Reference 3. The [illegible] responses to the generic items noted in Reference 3. The [illegible] are the [illegible] [illegible] corporate corporate [illegible] applicable to the [illegible] [illegible] Valve Data [illegible].

The Pacific Gas and Electric Company (PG&E) has completed the [illegible] safety valve [illegible] as part of the plant specific analysis

are in progress. For the reasons noted in the Attachment, we are requesting an extension to the submittal date of the Plant Specific Safety and Relief Valve Qualification and the Plant Specific Piping and Support Evaluation until March 1, 1983.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

R. S. Hunter
Vice President

/os

cc: John E. Dolan - Columbus
R. W. Jurgensen
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Charnoff
Joe Williams, Jr.
NRC Resident Inspector at Cook Plant - Bridgman

[illegible]

[illegible]

[illegible]

cc: [illegible]
[illegible]
[illegible]
[illegible]
[illegible]
[illegible]
[illegible]
[illegible]

ATTACHMENT TO AEP:NRC:0585C

Item 1 Plant Specific Reports for Safety and Relief Valve Qualification

As noted in our earlier response (Reference 2 in the cover letter) the Donald C. Cook Nuclear Plant has Masoneilan Model #38-20721 power operated relief valves (PORV) and Crosby Model #HB-BP-86-6M6 safety valves (SV). These valves are the same size and series as those valves tested in the EPRI valve test program and the EPRI valve test conditions envelop the range of expected operating and accident conditions for the Donald C. Cook Nuclear Plant.

The pressurizer PORVs were tested for various flow transient conditions such as steam, water and steam-to-water transition at two testing facilities, the Marshall Steam Station and Wyle Laboratories. The Masoneilan Valves' performance was acceptable under the various transient conditions simulated during the test.

The pressurizer safety valves were tested at the Combustion Engineering test facility in Windsor, Connecticut. The tests were conducted under steam, water and steam-to-water transition flow conditions. The tests were conducted both with and without a Water Loop Seal. The Donald C. Cook Plant safety valves have Water Loop Seals. The tests confirmed the ability of the safety valves to open and close under their expected operating fluid conditions. However, valve chattering was identified during some of the steam tests and steam-to-water transition tests. This resulted in pressure oscillations in the valve inlet piping.

The Westinghouse Owner's Group, of which Indiana & Michigan Electric Company is a member, authorized the Westinghouse Electric Corporation to perform a study to address: (a) those specific safety valve performance concerns identified during the EPRI valve tests, (b) the overall operability of the pressurizer safety valves and (c) the effect of safety valve performance on system overpressurization. Westinghouse is completing the above study and will shortly submit the final report (WCAP 10105, Reference No. 3 of the cover letter). EPRI is performing a study on the Piping Structural Response to incorporate the effect of the observed safety valve loads on the piping system. Evaluation of the

Plant specific safety valve operability can not be completed until the above mentioned reports have been completed and reviewed by us for their specific applicability to the Cook Plant. The Plant Specific Evaluation of Piping and Supports as noted in Item 2 below is also required to address the qualification of safety and relief valves.

Item 2 Plant Specific Submittals for Piping and Support Evaluations

Teledyne Engineering Services has been retained to assist in the evaluation of the safety and relief valve piping and its supports. The plant specific evaluation of the piping and supports has been delayed due to (a) delay in completing the safety valve testing by EPRI (b) pending completion of final report on piping structural response results from EPRI and (c) completion of the safety valve operability study by Westinghouse Electric Corporation.

We are hereby requesting an extension up to March 1, 1983 to submit a) the Plant Specific Evaluation of Piping and Supports, and b) Plant Specific Qualification of Safety and Relief Valves.

Item 3 Plant Specific Submittals for Block Valve Qualification

Indiana & Michigan Electric Company's position on Block Valve testing is as given in the letter from Mr. R. C. Youngdahl to Mr. H. R. Denton dated July 24, 1981 (Reference No. 4 of the cover letter). The details of block valve test results were submitted to the Commission by Mr. R. C. Youngdahl, Chairman, EPRI Research Advisory Committee on behalf of the Utility Owner's Group on June 1, 1982 (Reference No. 5 of the cover letter). The Donald C. Cook Plant has motor operated Model B-10-3054B-13MS 3" Velan Block Valves. A similar Velan Block Valve was tested at the Marshall Steam Station and the performance of the valve under various test conditions was satisfactory. .