

INDIANA & MICHIGAN
ELECTRIC COMPANY
DONALD C. COOK NUCLEAR PLANT

PROCEDURE COVER SHEET

Procedure No. **PMP 2081 EPP.023**

Revision No. **CANCELLED**

TITLE ACTIVATION AND OPERATIONS OF THE EMERGENCY CONTROL
CENTER (ECC) (AN EMERGENCY OPERATIONS FACILITY)

SCOPE OF REVISION

REASON FOR CANCELLATION: SUPERSEDED BY PMP 2081 EPP.022.

DCR
JUN 07 1982

SIGNATURES

	ORIGINAL	CANCELLED	REV. 2	Rev. 3
PREPARED BY	<i>David Webster</i>	<i>Richard Began</i>		
QUALITY ASSURANCE REVIEW	<i>[Signature]</i>	N.A.		
INTERFACING DEPARTMENT HEAD CONCURRENCE	N.A.	N.A.		
DEPARTMENT HEAD APPROVAL	N.A.	N.A.		
PLANT NUCLEAR SAFETY COMMITTEE	<i>[Signature]</i>	N.A.		
PLANT MANAGER APPROVAL	<i>[Signature]</i>	<i>[Signature]</i>		
DATE OF ISSUE	3-31-81	6/4/82		

INDIANA & MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT

PLANT MANAGER PROCEDURE

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Identification Number	Title	Revision No. And Date	Comments
PMP 2081 EPP.014	Off-Site Dose Assessments	Revision 0 4-1-81	TP-1,2-25-82 Exp N/A
EPP.015	Sampling and Analysis of Waterborne Releases.	Revision 0 4-1-81	
EPP.016	Collection and Analysis of Liquid and Gaseous Samples	Revision 0 4-1-81	
EPP.017	Interpretation of Liquid and Gaseous Samples	Revision 0 4-1-81	
EPP.018	Transportation Accidents Involving Radioactive Material	Revision 0 4-1-81	
EPP.019	AEP Emergency Response Organization Activation and Management	Revision 0 4-1-82	
EPP.020	Activation and Operation of the Technical Support Center (TSC)	Revision 0 4-1-81	TP-1,11-16-81 Exp N/A TP-2,3-12-82 Exp N/A TP-3,3-12-82 Exp N/A
EPP.021	Activation and Operation of the Operations Staging Area (OSA) and Personnel Accountability	Revision 1 5-25-82	
EPP.022	Activation and Operation of the Emergency Operations Facility	Revision 1 6-4-82	
EPP.023	Activation and Operation of the Emergency Control Center (ECC) (An Emergency Operations Facility)	CANCELLED 6-4-82.	
EPP.024	Activation and Operation of the Joint Public Information Center (JPIC) (An Emergency Operations Facility)	Revision 0. 4-1-81	

INDIANA & MICHIGAN

ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT

PROCEDURE COVER SHEET

Procedure No. PMP 2081 EPP.022

Revision No. 1

TITLE

ACTIVATION AND OPERATION OF THE EMERGENCY OPERATIONS FACILITY

SCOPE OF REVISION

Rev. 1: Revised the operation of the EOF to a single location as per NRC requirements.

DCR

JUN 07 1982

SIGNATURES

	ORIGINAL	Rev. 1	REV. 2	Rev. 3
PREPARED BY	<i>Dave Webster</i>	<i>Richard Beane</i>		
QUALITY ASSURANCE REVIEW	<i>John King</i>	<i>John King</i>		
INTERFACING DEPARTMENT HEAD CONCURRENCE	N.A.	N.A.		
DEPARTMENT HEAD APPROVAL	N.A.	N.A.		
PLANT NUCLEAR SAFETY COMMITTEE	<i>Richard Beane</i>	<i>Richard Beane</i>		
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INDIANA & MICHIGAN ELECTRIC COMPANY
DONALD C. COOK NUCLEAR PLANT

ACTIVATION AND OPERATION OF THE
EMERGENCY OPERATIONS FACILITY

1.0 OBJECTIVE

- 1.1 This procedure defines the responsibilities of individuals in the Emergency Operations Facility (EOF); the interfaces of the EOF with other centers and organizations; the operation of the EOF during short-term emergency response; and the staffing necessary to carry out these functions.

2.0 RESPONSIBILITIES AND FUNCTIONS

- 2.1 Responsibilities - See Exhibit A for the Emergency Operations Facility Organization.
- 2.1.1 Radiation Control and Waste Handling Manager (RCWHM): Assistant Plant Manager
 - 2.1.2 Communications Director: Production Supervisor, Technical
 - 2.1.3 Communications Personnel: Training Instructors, Performance Engineers, Operations Quality Control Implementation Coordinators
 - 2.1.4 Radiological Assessment Director: R.P. Performance Engineer
 - 2.1.5 Radiation Protection Manager: Plant Radiation Protection Supervisor
- 2.2 Functions - The EOF, upon activation, will assume responsibility for performing the following functions:
- 2.2.1 Management of overall I&M response
 - 2.2.2 Coordination of radiological and environmental assessment
 - 2.2.3 Determination of recommended public protective actions

- 2.2.4 Coordination of emergency response activities with Berrien County, the State of Michigan, and the Federal Government.

2.3 The Emergency Operations Facility is a support facility established for the purpose of performing the above functions. The EOF shall have appropriate technical data information and plant records to assist in the diagnosis of plant conditions to evaluate the potential or actual release of radioactive materials to the environment. An Assistant Plant Manager in this center shall organize and manage I&M support (dose assessment, communication, and radiation protection) resources to support the TSC and control room operators.

3.0 LIMITATIONS AND PRECAUTIONS

- 3.1 The Emergency Operation Facility shall be activated as necessary for any emergency condition and specifically for Site Emergencies and General Emergencies. This procedure covers the operation of the EOF until establishment of the AEP Emergency Response Organization on-site and/or until deactivation of the EOF.

4.0 INSTRUCTIONS

- 4.1. Upon recognition of the need for, or desirability of activating the EOF, the On-Site Emergency Coordinator shall direct the Communications Director to initiate PMP-2080-EPP.008, calling off-duty Plant personnel for the purpose of staffing the EOF.
- 4.2 The initial people to arrive in the EOF should prepare the facility for operation by performing the following:
 - 4.2.1 Unlock the EOF equipment cabinet. The key may be obtained from the Shift Security Supervisor or from the Training Department secretary.
 - 4.2.2 Turn on the radiation monitor. Instructions are posted on the monitor.
 - 4.2.3 Set up the communications/dose assessment room and "command center" for operation, as per Exhibit D.
 - 4.2.3.1 Set up tables and chairs.

- 4.2.3.2 Set up telecopier.
- 4.2.3.3 Set out telephones.
- 4.2.3.4 Set out the headset for the board writer.
- 4.2.4 Turn on Xerox machine.
- 4.3 The Radiation Control and Waste Handling Manager shall manage I&M support activities (dose assessment, communications, and radiation protection) from the EOF, and interface with the On-Site Emergency Coordinator. Specifically, the RCWHM shall:
 - 4.3.1 Oversee arrangement of the training building as the EOF. See Exhibit D.
 - 4.3.2 Establish communications directly as needed, with the OSEC for the purpose of managing I&M response to the event.
 - 4.3.3 Ensure the EOF assumes responsibility for communication with off-site organizations.
 - 4.3.4 Ensure the EOF assumes dose assessment and protective action decision-making responsibilities.
 - 4.3.5 Coordinate I&M and Federal, State, and local response to the event via:
 - 4.3.5.1 Information passed both ways through the communicators.
 - 4.3.5.2 Establishing the "command center", as needed. See Exhibit D.
 - 4.3.5.3 Establishing the NRC room, as needed. See Exhibit D.
 - 4.3.5.4 Establishing the New York Emergency Response Organization room, as needed. See Exhibit D.
 - 4.3.6 Complete the top portion of Exhibit C for use by the Radiological Assessment Director.
 - 4.3.7 Ensure Exhibit C is normally developed every 15 minutes. Approve Exhibit C upon completion and prior to release to off-site organizations.

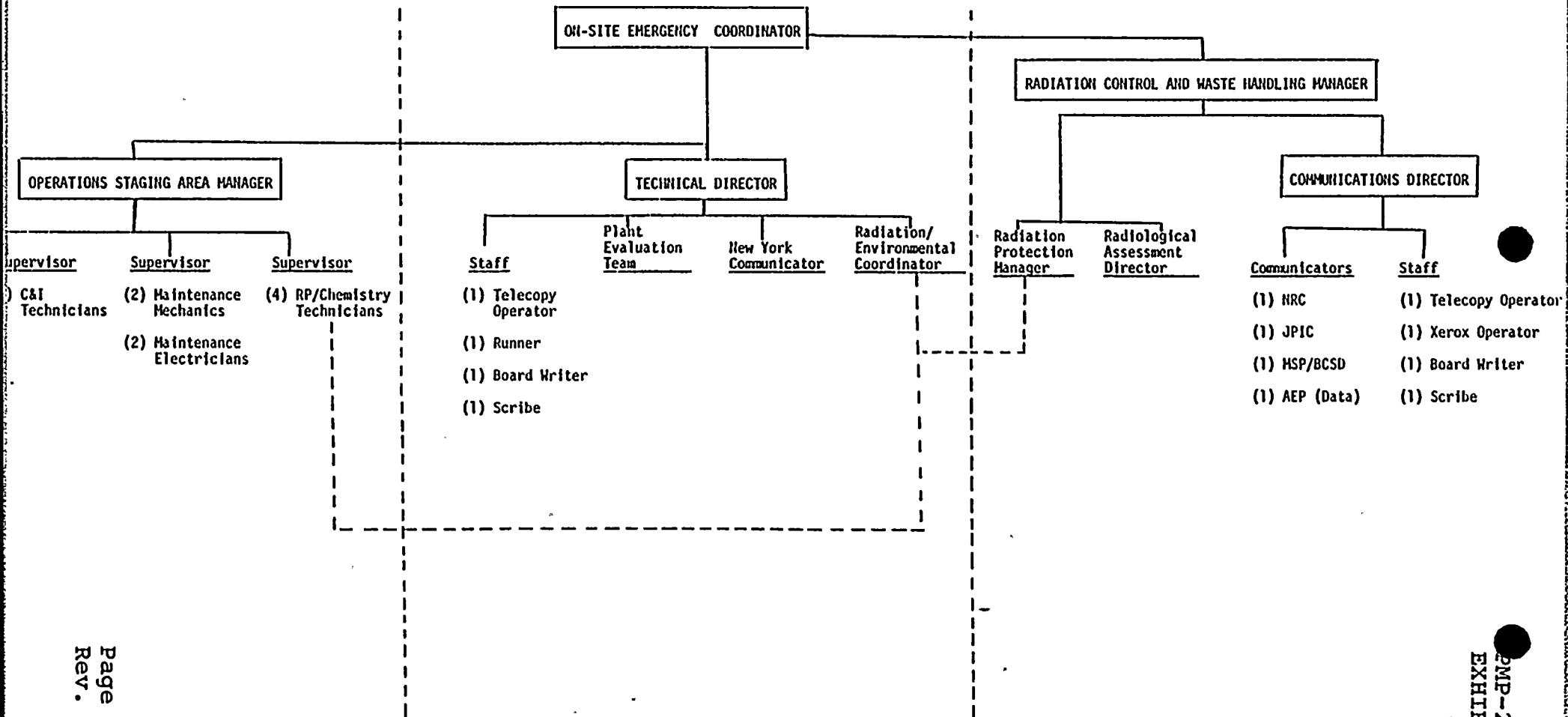
- 4.3.8 Remain cognizant of the activities of the Radiation Protection Manager in response to the emergency.
- 4.3.9 Direct the Shift Security Supervisor to establish security controls around the EOF building. These controls shall ensure that access is restricted to personnel assigned to the EOF.
- 4.4 The Communications Director shall perform the following functions:
 - 4.4.1 Establish a chronological history log of events and make this log available to communications, dose assessment, radiation protection, and "command center" personnel.
 - 4.4.2 Ensure that detailed event data, Exhibit B, is made available regularly to communications, dose assessment and radiation protection personnel. Normally this data will be provided every fifteen minutes by telecopier from the TSC.
 - 4.4.3 Ensure the communicators, dose assessment and radiation protection personnel are brought up-to-date regarding event data and information required to perform their functions.
 - 4.4.4 Establish communication with the Berrien County Sheriff's Department, Michigan State Police, Nuclear Regulatory Commission, American Electric Power Initial Assessment Group and Public Affairs. Assume responsibility for this function from the TSC.
 - 4.4.4.1 Exhibit B is to be provided to NRC and AEP, and to the JPIC and Westinghouse as needed.
 - 4.4.4.2 Exhibit C is to be provided to AEP, the JPIC, the Berrien County Sheriff's Department, and the Michigan State Police. Exhibit C is also to be supplied to the NRC as needed.

- 4.4.5 Assume responsibility for the dose assessment and radiation protection functions from the TSC.
- 4.4.6 Communicate requests from off-site groups to the Technical Director.
- 4.5 The Radiological Assessment Director is responsible for:
 - 4.5.1. Performing dose calculations for on-site and off-site areas and reporting these results to the Communications Director and Radiation Control and Waste Handling Manager.
 - 4.5.2 Recommending protective actions on-site and off-site when appropriate.
- 4.6 The Radiation Protection Manager is responsible for:
 - 4.6.1 Implementing the emergency radiation protection program including personnel monitoring and decontamination.
 - 4.6.2 Directing activities of Radiation Monitoring teams and reporting results to the Radiation Control and Waste Handling Manager.
 - 4.6.3 Directing the activities of Re-entry and Rescue teams, as appropriate.
 - 4.6.4 Coordinating first-aid activities.

(OPERATIONS STAGING AREA)

(TECHNICAL SUPPORT CENTER)

(EMERGENCY OPERATIONS FACILITY)



PMP 2081.EPP.022
EXHIBIT B

D. C. Cook Unit No: _____ Data Taken By _____ Date _____ Time _____

RCS PARAMETER

- | | | | | | | | | | |
|-----|------------------------|----------|-------|-----|----------------------|----------|-----------------|-------|--------|
| 1. | RWST Level | _____ | % | 2. | Containment Pressure | _____ | PSIG | | |
| 3. | Containment Sump Level | _____ | % | 4. | Containment Level | _____ | % | | |
| 5. | BIT Flow | LP1 | _____ | GPM | LP2 | _____ | GPM | | |
| 6. | SI Flow | North | _____ | GPM | South | _____ | GPM | | |
| 8. | RHR To CTS Flow | East | _____ | GPM | West | _____ | GPM | | |
| 9. | CTS | East (ON | _____ | OFF | _____) | West (ON | _____ | OFF | _____) |
| 10. | RCP Status | LP1 (ON | _____ | OFF | _____) | LP2 (ON | _____ | OFF | _____) |
| | | LP3 (ON | _____ | OFF | _____) | LP4 (ON | _____ | OFF | _____) |
| 11. | RCS Press. | _____ | PSIG | → | ↑ | ↓ | | | |
| 13. | PZR Liquid Temp. | _____ | °F | 12. | PZR Level | _____ | % | | |
| 15. | PZR Backup Htrs. | (ON | _____ | OFF | _____) | 14. | PZR Steam Temp. | _____ | °F |
| 17. | Charging Flow | _____ | GPM | 16. | PZR Cycling Htrs. | (ON | _____ | OFF | _____) |
| | | | | 18. | Letdown Flow | _____ | GPM | | |

NSSS LOOP PARAMETERS

NSSS LOOP PARAMETERS		Loop 1	Loop 2	Loop 3	Loop 4
19.	Wide Range T _H	_____ °F	_____ °F	_____ °F	_____ °F
20.	Wide Range T _C	_____ °F	_____ °F	_____ °F	_____ °F
21.	S/G Pressure	_____ PSIG	_____ PSIG	_____ PSIG	_____ PSIG
22.	S/G N.R. Level	_____ %	_____ %	_____ %	_____ %
23.	S/G W.R. Level	_____ %	_____ %	_____ %	_____ %
24.	Steam Flow (pph X 10 ⁶)	_____	_____	_____	_____
25.	Feed Flow (pph X 10 ⁶)	_____	_____	_____	_____
26.	Aux. Feed Flow(pph X 10 ³)	_____	_____	_____	_____
27.	MSIV Status	(OP___CL___)	(OP___CL___)	(OP___CL___)	(OP___CL___)
28.	CST Level	_____ %	_____ Ft.		

DOSE PARAMETERS

- | | | | |
|-----|-----------------------|-------|----------|
| 29. | R-2 Contain. Area | _____ | R/Hr. |
| 30. | R-11 Contain. Part. | _____ | CPM |
| 31. | R-12 Contain. Gas | _____ | CPM |
| 32. | R-33 Gland Exh. Vent | _____ | CPM |
| 33. | R-15 Air Ejector Vent | _____ | CPM |
| 34. | R-26 Vent Stack Gas | _____ | CPM |
| | | | |
| 35. | Vent Flow | _____ | CFM |
| 36. | GSLO Flow | _____ | CFM |
| 37. | SJAE Flow | _____ | CFM |
| | | | |
| 38. | Wind Speed | _____ | MPH |
| 39. | Wind Direction | _____ | ° (From) |
| 40. | Air Temp. ΔT | _____ | °C |

SPECIAL NOTES OR DATA

Data Verified By _____

PMP 2081 EPP.022
EXHIBIT C

① Description of Event _____

Classification: Unusual Event____ Alert____ Site Emergency____ General Emergency____
Injured Personnel: Yes, Number____, No____ Radioactivity Contaminated: Yes____ No____
Estimated Duration of Release

Source Term for Calculation	Flow for Calculation
___ Monitor Reading	___ Measurements
___ Upper Limit of Monitor Range	___ Upper Limit on Containment
___ Dose Rate 6" from Effluent Pipe	___ Leak Rate
___ Sample	Time Flow Obtained _____
___ Time Source Term Obtained	

Time of Measurement _____ Measured Dose Rate _____
 Location of Measurement _____ Dose Rate _____
 _____ Iodine Conc. _____

Meteorological Conditions: Wind Speed (mph) _____
 Wind Direction: From _____ To _____
 Stability Class: ΔT _____ Pasquill Cat. A B C D E
 Precipitation: Yes _____ No _____

Release Rate: Total Gas Ci/sec Release Height: Ground Level

Projected Dose At:	Site Boundary	2 Miles	5 Miles	10 Miles
Dose Rate, Whole Body, R/hr				
*Integrated Dose, Whole Body, R				
Dose Rate, Thyroid, R/hr				
*Integrated Dose, Thyroid, R				
Sectors Affected				

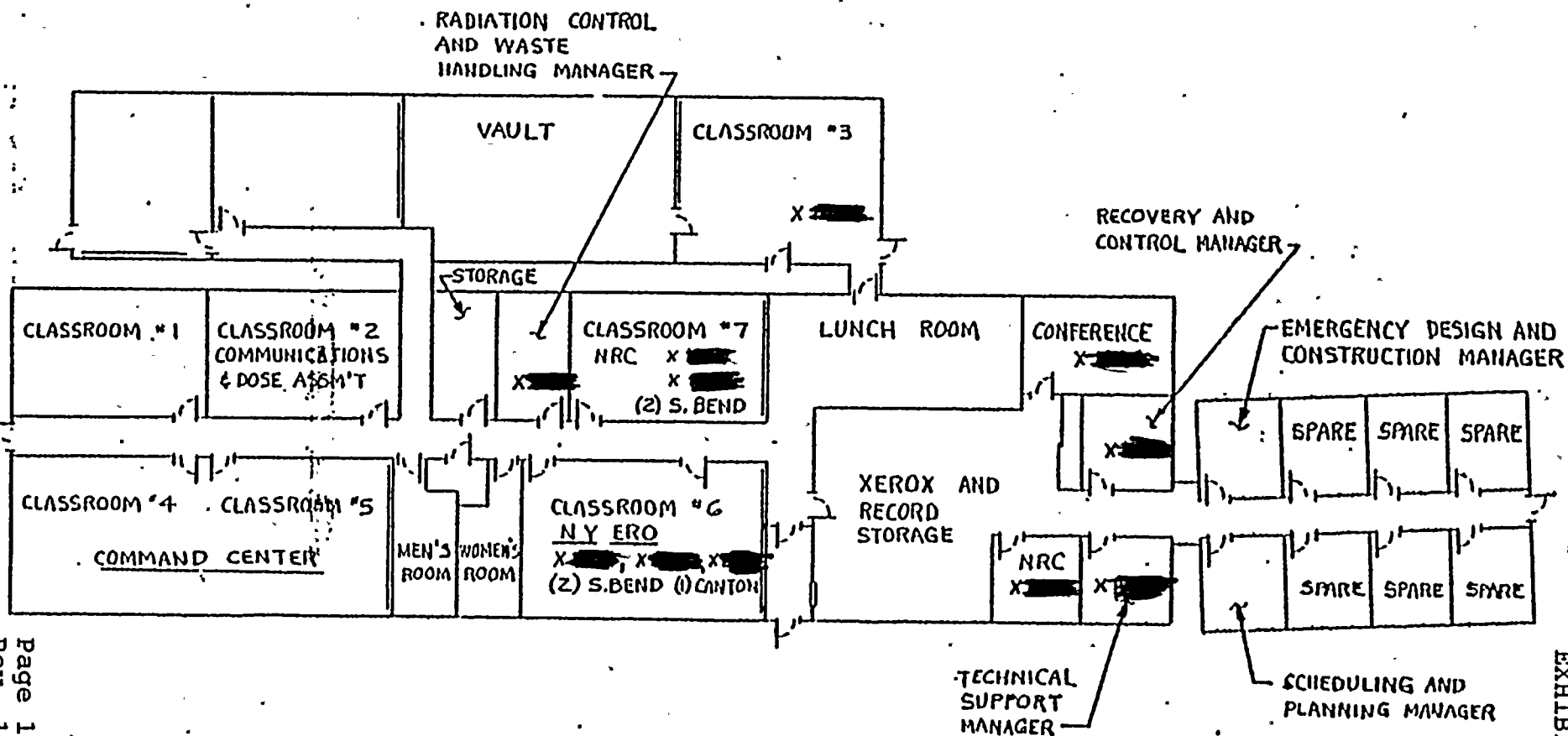
*Integrated from the time of initial calculation to end of projected release.

Time Calculation Performed **Input Verified**

[illegible]

Support Requested From State or County Agencies: Yes___ No___
Type

Recommended Emergency Actions For State or County Agencies





1

2

3

4

5

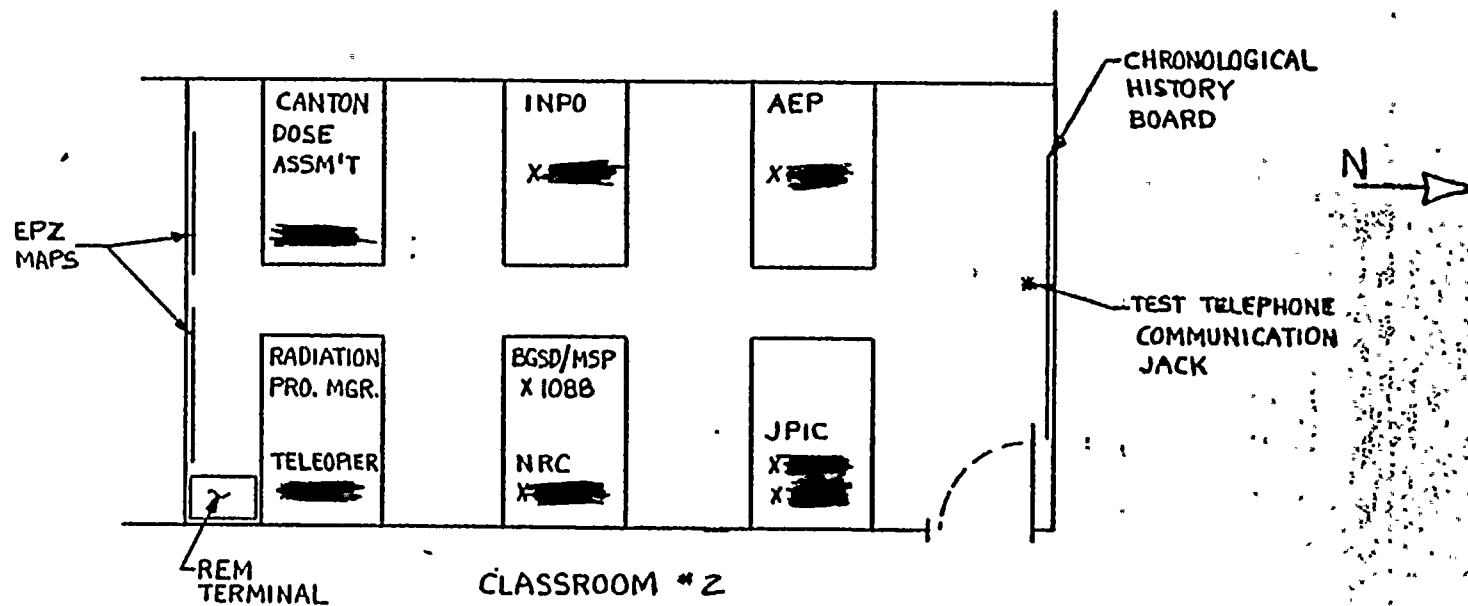
6

7

8

9

10



CLASSROOM #2
DOSE ASSESSMENT AND COMMUNICATIONS

14M E CO O.C. COOK NUCLEAR BRIDGMAN, MI	TRAINING BUILDING EMERGENCY OPERATIONS FACILITY	
	MAY 13 1982	DARRELL STINER

