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DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Requests changes to Tech Specs,consisting of revised pages to App A.Description of changes encl.

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App 2

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January 22, 1982
AEP:NRC:00591

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
TECHNICAL SPECIFICATIONS CHANGE REQUEST

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Denton:

This letter and its Attachments request several changes to the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 Appendix A Technical Specifications. Attachment No. 1 to this letter contains the description of the changes. Attachment No. 2 contains the revised Technical Specification pages for Unit No. 1 and Attachment No. 3 contains the revised Technical Specification pages for Unit No. 2. All changes are indicated by a vertical line on the right-hand side of the page in Attachment Nos. 2 and 3.

The proposed Technical Specification changes contained herein have been reviewed and approved by the Plant Nuclear Safety Review Committee (PNSRC) and by the AEPSC Nuclear Safety and Design Review Committee (NSDRC).

Four proposed Technical Specification revisions are contained in this submittal. Change Nos. 1, 2, and 3 are considered to be Class II amendments ($1,200 fee each). Class I duplicate fees also apply for these changes. The payment for Change No. 4 will be made as part of the fee to be levied for the issuance of the Category B Technical Specifications of NUREG-0737. Enclosed, therefore, is a check in the amount of $4,800.00 for NRC processing of the aforementioned requests.

A046
s/
1/1
w/check!
$4800.00

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

RFH/os

R. F. Hering
Vice President

cc: John E. Dolan - Columbus
R. W. Jurgensen
W. G. Smith - Bridgman
R. C. Callen
G. Charnoff
Joe Williams, Jr.
NRC Resident Inspector at Cook Plant - Bridgman
R. S. Hunter

ATTACHMENT NO. 1 TO AEP:NRC:0591

DESCRIPTION OF PROPOSED TECHNICAL SPECIFICATION CHANGES FOR
D. C. COOK UNIT NOS. 1 AND 2

Change No. 1

Unit 1; Page 3/4 3-12; Section 3.3.1.1
Unit 2; Page 3/4 3-11; Section 3.3.1.1

For Unit Nos. 1 and 2, it is requested that surveillance requirements for the "Source Range, Neutron Flux" Channels be deleted in Mode 2. The Source Range Channels do not provide any useful information in Mode 2. Furthermore, Westinghouse Technical Bulletin NSD-TB-80-8 recommended not powering Source Range detectors while at power because of possible detector degradation or failure. For Unit 2, it is additionally requested that surveillance requirements for the "Source Range, Neutron Flux" Channels be deleted in Mode 6. While in Mode 6, a weekly surveillance is already required under Specification 4.9.2. For Unit 2, it is also requested to delete the "M" requirement of the Channel Functional Test for the "Source Range, Neutron Flux" Channels and to change the Channel Calibration requirements of the "Intermediate and Source Range, Neutron Flux" Channels (Item Nos. 5 and 6 in Table 4.3-1) to "N.A." (not applicable). The monthly Functional Test ("M") is unnecessary as such testing performs no useful function. The deletion of the monthly requirement will make Table 4.3-1 of Unit 2 consistent with that of Unit 1. Further, under note (6) of Table 4.3-1, neutron detectors may be excluded from Channel Calibration. These changes will not adversely affect the health and safety of the general public.

Change No. 2

Unit 1; Page 3/4 3-26a; Section 3.3.2.1
Unit 2; Page 3/4 3-25a; Section 3.3.2.1
Reference: AEP:NRC:0268

Presently, the Loss-of-Voltage relays in paragraphs 6b and 8a in both Units 1 and 2 are specified as 3196, ± 18 volts and the Degraded Grid Voltage relays in paragraph 8b are specified as 3596, ±18 volts. The Technical Specification values for these voltages and their associated tolerance bands are extremely conservative. Attempts to comply with the allowable voltage values in the Technical Specifications have been unsuccessful due to the inability of the relays to repeat their settings within the desired tolerance.

It is thus proposed to change the allowable values for Loss-of-Voltage relays in paragraphs 6b and 8a of both Units from 3196±18 volts to 3196,+18, -36 volts and change the allowable values of the Degraded Grid Voltage relays in paragraph 8b of both Units from 3596±18 volts to 3596, +36,-18 volts.

Currently, the Loss-of-Voltage relays have Technical Specification allowable values between 79.45% and 80.35% of the rated voltage value (4kV). The proposed change would alter the range from 79.00% to 80.35% (3196, +18, -36 volts). This change is acceptable since motor operation at 79.00% of rated voltage for two-minutes is not significantly different from motor operation at 79.45% of rated voltage for the same two-minute period.

The Degraded Grid Voltage relays now have Technical Specification allowable values between 89.45% and 90.35% of the rated voltage value (4kV). The proposed change would alter the range from 89.45% to 90.80% (3596, +36, -18 volts). This change is acceptable since under postulated worst case Degraded Grid Voltage conditions, the voltage would recover to 93.3% of rated voltage which is still above this proposed new upper tolerance value. These proposed changes are consistent with our analysis for loss of Voltage and Degraded Grid Voltage events and will not affect the health and safety of the general public.

Change No. 3

Unit 1; Page 3/4 2-14; Section 3.2.5
Unit 2; Page 3/4 2-16; Section 3.2.5

This change revises the footnote of Table 3.2-1. A request for this footnote to be revised was previously submitted on December 22, 1978 in AEP:NRC:0109 (Unit 1) and on February 13, 1979 in AEP:NRC: 0111 (Unit 2). The change requested therein was further modified during discussions with members of your Staff. It was agreed that the footnote would be modified to reflect the wording of the Standard Technical Specifications contained in Revision 3 of NUREG-0452. Recent discussions with members of your Staff indicate that this modified footnote of Table 3.2-1 will be issued shortly. The present change further modifies the footnote to make the reporting requirements of Specification 6.9.1.9 not applicable for transients during start-up or shutdown operations. It is requested that this present change be processed independently of our previous request contained in AEP:NRC:0109 and AEP:NRC:0111 as modified through discussions with members of your Staff. The attached Technical Specification changes in Attachment Nos. 2 and 3 reflect both the original change request, as modified through discussions with members of your Staff, and the present change request. These proposed changes will not adversely affect the health and safety of the general public.

Change No. 4

Unit 1; Page 3/4 6-17; Section 3.6.3
Unit 2; Page 3/4 6-18; Section 3.6.3

In order to meet the requirements of NUREG 0737, II.B.3, Post-Accident Sampling Capability, six new containment isolation valves are being installed per Unit. These include sump sample, containment gas sample, and waste return valves.

Table 3.6-1 of both Unit's Technical Specifications has been changed to reflect the addition of these valves. This Technical Specification change will become applicable when the valves are installed. The new valves are numbered 47-52. The valve numbers in Table 3.6-1 have been changed to reflect the addition of these six valves. This change will not affect the health and safety of the general public.