

INDIANA & MICHIGAN ELECTRIC COMPANY

P. O. BOX 18
BOWLING GREEN STATION
NEW YORK, N. Y. 10004

October 19, 1981
AEP:NRC:0621

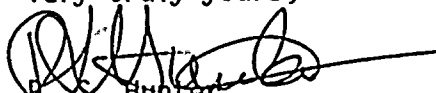
Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
IE Inspection Report Nos. 50-315/81-19; 50-316/81-22

Mr. James G. Keppler, Regional Director
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region III
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

This letter and its attachments respond to Mr. C. E. Norelius' letter of August 31, 1981. Attachment 1 addresses the two violations noted in the above mentioned Inspection Report and the corrective actions taken. Attachment 2 provides you with information on the actions we have taken and plan to take to resolve Mr. I. Yin's concerns as described in Item 7 of the "Details" part of the subject report. An extension up to October 19, 1981 to answer the report was granted to us by Mr. D. W. Hayes of your Staff.

Very truly yours,


R. S. Hunter
Vice President

cc: John E. Dolan - Columbus
R. C. Callen
G. Charnoff
R. W. Jurgensen
D. V. Shaller - Bridgman
Joe Williams, Jr.
Region III Resident Inspector-Bridgman

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R. S. Hunter, being duly sworn, deposes and says that he is a Vice President of the licensee Indiana & Michigan Electric Company, that he has read the foregoing response to I.E. Inspection Report Nos. 50-315/81-19 and 50-316/81-22 dated August 31, 1981 and knows the contents thereof; and that said contents are true to the best of his knowledge and belief.



Vice President

Sworn and subscribed to before me this
19th day of October, 1981.

C. Robert Roll.

Notary Public

C. ROBERT ROLL
NOTARY PUBLIC, STATE OF NEW YORK
No. 41-3332850
Qualified in Queens County
Commission Expires March 30, 1983





ATTACHMENT 1 TO AEP:NRC:0621

Response to Item 1: This item in the inspection report refers to AEP Design Division Organization and Procedures Manual, General Design Procedure No. 8, Rev. 2 dated October 31, 1980 and notes that the engineering calculations must be stamped, signed and dated by the originator and the checker. The inspector noted that, contrary to the above, AEP calculation No. 2-258, dated December 3, 1979 had not been signed and dated by the checker. We have verified our document files and noted that the computer input data for Problem 2-258 was checked and signed by both the originator and checker and that the signed copy existed in the Supervising Designer's file at the time of Mr. Yin's audit. Since then the copy of the calculation which Mr. Yin audited has been signed and dated. To avoid the possibility of having unsigned copies of the original computer calculations being filed in the problem folders, Procedure No. 8 is being revised to include the requirement that the copies of computer calculations in the record files should show the evidence of a check and be dated. This revision to the procedure will be issued prior to October 31, 1981 under Revision No. 4.

Response to Item 2: This item refers to paragraphs 6.a.(4) and 6.d.(1) of the inspection report. These refer to EDS calculations No. 2-215 "Chemical and Volume Control, 1-CCS-5, 3" and 4" CS Line from CPN-37", Revision 1, dated October 18, 1980, and TES calculation No. 2-178, "Technical Report, TR-4128-2, Stress Analysis of Essential Service Water to Auxiliary Feed Pump Piping (2-ESW-5) for D. C. Cook Nuclear Plant," dated January 8, 1980. The computer analyses showed that portions of these piping systems were stressed in excess of our FSAR allowable value for an earthquake loading. The inspector noted that modifications were made to these piping systems to reduce the stresses to be within our FSAR allowable limits but that a written report had not been sent to the NRC Region III Office.

As an immediate corrective action, Indiana & Michigan Electric Company reported these two problems to the NRC Region III office on September 16, 1981, via LER No. RO-81-044/03L-0.

In addition, we are reviewing the 393 package compiled as a result of our efforts to comply with the requirements of IE Bulletin 79-14. We initiated this review program shortly after Mr. Yin's visit to our New York offices with the purpose of identifying those instances where reporting of pipe stress values exceeding the FSAR allowable limits was not done. The review effort includes not only identifying reportability problems, but also a QA check on all packages to assure proper problem documentation closure.

During this review some cases of stress values exceeding the FSAR allowable limits have been found and will be reported. We anticipate that this review will be completed by December 31, 1981.

We believe that we have satisfactorily responded to the requirements of the Bulletin as evidenced, in part, by our decision to shut down Unit 2 on December 24, 1979, due to significant nonconformances found in the hydrogen skimmer system which possibly could have resulted in an operability problem. Additionally, we note that following a December 29, 1980 management meeting between the NRC and I&MECo., our professionalism and responsiveness to the Bulletin was favorably commented upon by members of the NRC Staff.

ATTACHMENT 2 TO AEP:NRC:0621

This attachment provides you with information on our activities concerning Item 7 on the "Details" Section of Mr. Yin's report.

Westinghouse has advised us that the setting of the rear pump restraint gap clearance to $1/16$ inch + $1/32$ " - 0" was satisfactory.

In addition, we have gone to Westinghouse with all measured gap information on the major reactor coolant loop restraints in an effort to resolve other discrepancies with the Westinghouse recommended values. Westinghouse has advised us that the present restraint gap clearances for both units is not a safety issue. However, Westinghouse suggests that the gap settings be changed to agree with what they are recommending for nuclear plants today.

Where required, we are measuring and re-shimming the gaps in Unit 2 during the current outage and it is our intention to have the same operations done in Unit 1 during an upcoming outage so as to meet the Westinghouse recommended settings.

