OPERATING DATA REPORT

DOCKET NO. 50-315
DATE 10/6/80
COMPLETED BY W. T. Gillett
TELEPHONE 616-465-5901

OPERATING STATUS

1. Unit Name: Donald C. Cook 1
2. Reporting Period: September 1980
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1089
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	6,575	50,399
12. Number Of Hours Reactor Was Critical	696	4,548.3	37,510
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	691.7	4,466	36,549.2
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	2,100,551	13,721,043	102,787,349
17. Gross Electrical Energy Generated (MWH)	698,140	4,545,040	33,702,380
18. Net Electrical Energy Generated (MWH)	672,667	4,381,662	32,378,976
19. Unit Service Factor	96.1	67.9	75.7
20. Unit Availability Factor	96.1	67.9	75.7
21. Unit Capacity Factor (Using MDC Net)	89.5	63.8	66.5
22. Unit Capacity Factor (Using DER Net)	88.6	63.2	62.2
23. Unit Forced Outage Rate	3.9	9.8	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

8010200415

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315

UNIT 1

DATE 10/6/80

COMPLETED BY W. T. Gillett

TELEPHONE 616-465-5901

MONTH September 1980

DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	670	17	1032
2	976	18	1034
3	998	19	861
4	999	20	---
5	1001	21	620
6	1003	22	1008
7	1015	23	1010
8	1015	24	1026
9	1016	25	1029
10	1027	26	1032
11	1031	27	1032
12	1030	28	1037
13	654	29	1030
14	821	30	1033
15	1023	31	
16	1028		

INSTRUCTIONS

On this format list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50 - 315
UNIT NAME D.C. Cook - Unit 1
DATE 10-14-80
COMPLETED BY B.A. Svensson
TELEPHONE (616) 465-5901

REPORT MONTH September, 1980

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
166	800913	S	0	B	H	N.A.	ZZ	ZZZZZZ	Reactor power reduced to 55% to remove the west main feedpump turbine from service to repair oil leaks in the feedpump turbine lube-oil tank. Reactor power returned to 98% 800915.
167	800919	F	28.3	B	1	80-023/03L	IB	INSTRU	Unit removed from service to repair suspected leak in sensing line for No. 2 steam generator narrow range level channel BLP-122. Leak found to be pinhole leak in a socket weld downstream of root valve in the sensing line. Unit returned to service 800921, and 97% reactor power reached 800922.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely[1]. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE.. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT =. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following critieria:

A. If a component failed, use the component directly involved.

B. If not a component failure, use the related component: e.g., wrong valve operated through error: list valve as component.

C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

[1]Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

MONTHLY OPERATING ACTIVITIES -- SEPTEMBER, 1980

Highlights :

The Unit entered the reporting period operating at 45% power, in recovery from a reactor trip from 95% power that occurred at 0447 hours 8/31/80.

There was one outage during the reporting period which is detailed in the Summary.

Gross electrical generation for the month was 698,140 wmh.

Summary :

9/01/80 -- The Unit was returned to 95% power over a 13 hour ramp starting at 0725 hours.

The 50' Wind Speed and Direction Instruments were inoperable for a 6 hour period, while repairs were made. This instrument is common to both Units.

9/02/80 -- Reactor power was increased from 95% to 97% over a 5 hour ramp starting at 1736 hours.

9/06/80 -- Containment Radiation Monitors R-11 and R-12 were inoperable for a 7 hour period to replace the sample pump.

9/09/80 -- The North Control Rod Drive Motor Generator Set was removed from service at 0447 hours due to a failed bearing. Repairs were made and the Motor Generator Set was returned to service at 2210 hours, 9/15/80.

The 150' Wind Direction Instrument was declared inoperable at 1331 hours. Repairs were made and this was returned to service at 1110 hours 9/14/80. This instrument is common to both Units.

9/10/80 -- Unit loading was increased from 97% to 98% power over a 2.5 hour ramp starting at 1700 hours.

Summary (cont.) :

9/11/80 -- 345KV Generator Breaker K-1 was open for a 1.5 hour period while Reserve Power Transformer #5 was test energized.

9/13/80 -- Unit loading was reduced to 55% power over a 2 hour ramp, starting at 0500 hours. This was to remove the West Main Feed Pump from service to repair oil leaks in the Lubricating Oil Tank. The West Main Feed Pump was returned to service at 2315 hours. Unit loading was held at the 55% level due to reduced system demands.

9/14/80 -- Unit power was increased to 93% power over a 7.25 hour ramp starting at 0550 hours.

9/15/80 -- Power was returned to 98% over a 5 hour ramp starting at 0100 hours.

9/19/80 -- Containment Recirculation Fan CEQ-1 was inoperable for a 11.5 hour period while a second breaker was installed in its power circuit.

The Unit was unloaded to the 6% power level over a 2.25 hour ramp starting at 2000 hours. The Turbine Generator was removed from service at 2214 hours. The reason for the outage was to repair a suspected leak in the sensing line for No. 12 Steam Generator Narrow Range Level Channel No. 3. The leak was located at a socket weld elbow downstream of the root valve.

9/20/80 -- To improve working conditions within the Containment, the Reactor was made subcritical at 0030 hours and the Reactor Plant was cooled down to 370°F and depressurized to 1600 psig by 0600 hours.

All Reactor Coolant Pumps were placed in service at 1900 hours and the Reactor Coolant System was returned to normal operating temperature and pressure by 2237 hours.

9/21/80 -- Reactor was returned to criticality at 0029 hours. Main Turbine was rolled at 0155 hours and the Unit paralleled to the system at 0233 hours.

The Unit was loaded to 94% power by 2020 hours.

The Unit was returned to 97% power over a 3 hour ramp starting at 2100 hours.

Summary (cont.) :

9/23/80 -- The Unit was loaded to 99% power over a 4 hour ramp starting at 1015 hours.

The West Essential Service Water pump was inoperable for a 15.5 hour period, while adjustments were made to the pump.

East Safeguard Fan 1-HV-AES-2 was removed from service at 1415 hours to run a Performance check on its charcoal filter. This fan was returned to operable status at 1415 hours 9/25/80.

9/27/80 -- Power was reduced to 95% for testing of the Main Turbine Control Valves. The Control Valve testing was not accomplished due to the Turbine Control System becoming unstable when setup in the testing Mode. Total time below 99% power was 2.75 hours.

9/30/80 -- Vent Stack Radiation Monitors R-25 and R-26 were inoperable for a 9.25 hour period for replacement of the sample pump.

AB Emergency Diesel Generator was inoperable five times during the reporting period for a total time of 37.25 hours, while adjustments were made to the engine.

DOCKET NO.	50 - 315
UNIT NAME	D. C. Cook - Unit No. 1
DATE	10-14-80
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901
PAGE	1 of 1

MAJOR SAFETY-RELATED MAINTENANCE

SEPTEMBER, 1980

M-1 A weld leak developed at the root valve for instrument IPI-220, west containment spray pump discharge pressure indication. The pipe nipple was cracked next to the weld. The nipple was cut off and replaced. Applicable NDE was performed.

M-2 A leak was suspected in the sensing line for No. 2 steam generator level instrument. The unit was removed from service and investigation revealed a pinhole leak in a socket weld downstream of the root valve for instrument BLP-122. The leak was repaired by welding and applicable NDE was completed.