

# OPERATING DATA REPORT

DOCKET NO. 50-315  
 DATE 8-4-80  
 COMPLETED BY B. Gillett  
 TELEPHONE 616-564-5901

## OPERATING STATUS

1. Unit Name: D. C. Cook Unit 1  
 2. Reporting Period: July  
 3. Licensed Thermal Power (MWt): 3250  
 4. Nameplate Rating (Gross MWe): 1089  
 5. Design Electrical Rating (Net MWe): 1054  
 6. Maximum Dependable Capacity (Gross MWe): 1080  
 7. Maximum Dependable Capacity (Net MWe): 1044  
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):  
 10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,111	48,935
12. Number Of Hours Reactor Was Critical	0	3,207.9	36,196.6
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	0	3,199.7	35,282.9
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	0	10,118,931	99,185,236
17. Gross Electrical Energy Generated (MWH)	0	3,362,810	32,520,150
18. Net Electrical Energy Generated (MWH)	0	3,245,802	31,243,116
19. Unit Service Factor	0	62.6	75.4
20. Unit Availability Factor	0	62.6	75.4
21. Unit Capacity Factor (Using MDC Net)	0	60.8	66.3
22. Unit Capacity Factor (Using DER Net)	0	60.3	61.8
23. Unit Forced Outage Rate	0	11.6	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: August 6, 1980

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315

UNIT 1

DATE 8-1-80

COMPLETED BY W. T. Gillett

TELEPHONE 616-465-5901

MONTH July 1980

DAY . AVERAGE DAILY POWER LEVEL  
(MWE-Net)

1	_____
2	_____
3	_____
4	_____
5	_____
6	_____
7	_____
8	_____
9	_____
10	_____
11	_____
12	_____
13	_____
14	_____
15	_____
16	_____

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17	_____
18	_____
19	_____
20	_____
21	_____
22	_____
23	_____
24	_____
25	_____
26	_____
27	_____
28	_____
29	_____
30	_____
31	_____

## INSTRUCTIONS

On this format list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1980

DOCKET NO. 50-315  
 UNIT NAME D.C.Cook-Unit  
 DATE 8-6-80  
 COMPLETED BY B.A. Svensson  
 TELEPHONE (616) 465-5901

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
160	800530	S	744	B&C	1	N.A.	ZZ	ZZZZZZ	The Unit was removed from service for Cycle IV - V refueling outage at 2228 hours EDT on 800530. The Unit remained out of service the entire month.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

(9/77)

## INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely<sup>1</sup>. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

**NUMBER.** This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

**DATE.** This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

**TYPE.** Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

**DURATION.** Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

**REASON.** Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

**METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER.** Categorize by number designation

<sup>1</sup>Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

**LICENSEE EVENT REPORT.** Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

**SYSTEM CODE.** The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

**COMPONENT CODE.** Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error: list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

**CAUSE-& CORRECTIVE ACTION TO PREVENT RECURRENCE.** Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Socket No.: 50-315  
Unit Name: D. C. Cook Unit #1  
Completed By: R. S. Lease  
Telephone: (616) 465-5901  
Date: August 7, 1980  
Page: 1 of 1

MONTHLY OPERATING ACTIVITIES -- JULY, 1980

Highlights :

The Unit has been shut down the entire reporting period for its fourth Refueling Outage. The time of shut down was 2228 hours 5/30/80.

Electrical generation for the month was 0.

Summary :

- 7/2/80 -- Reactor Head was set on the vessel at 0530 hours.
- 7/9/80 -- The Reactor Plant entered Mode 5 with the initial bolting of the reactor head.
- 7/20/80 -- Safety Train "A" blackout testing was successfully completed.
- 7/21/80 -- Safety Train "B" blackout testing was successfully completed.
- 7/23/80 -- Filling and venting of the Reactor Coolant System, from the  $\frac{1}{2}$  loop level was initiated at 1812 hours.
- 7/26/80 -- A steam bubble was formed in the Pressurizer at 0310 hours. System pressure was increased to 325 psig for hand rotation of the four Reactor Coolant Pumps. The #14 pump would not rotate when the maximum recommended manual force was applied.
- 7/27/80 -- Reactor Coolant System pressure was reduced to 0 and the system drained to the  $\frac{1}{2}$  loop level by 0830 hours. This was to allow inspection of the thrust bearing of the #14 Reactor Coolant Pump.
- 7/30/80 -- Filling and venting of the Reactor Coolant System was initiated at 0048 hours and completed at 1928 hours 3/31/80.



DOCKET NO.	50 - 315
UNIT NAME	D. C. Cook - Unit No. 1
DATE	8-6-80
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901
PAGE	1 of 1

MAJOR SAFETY-RELATED MAINTENANCE

JULY, 1980

- M-1      The stem on DG-227C, LCD diesel jacket water sample valve was broken with the valve open. The valve was replaced.
- M-2      NSW-244-3, non-essential service water containment isolation check valve failed Type C leak test. Seat and disc were lapped and valve reassembled.
- M-3      NSW-415-2, non-essential service water containment isolation check valve failed Type C leak test. Valve body cleaned and new disc installed.
- M-4      No. 1 steam generator blowdown regulating valve, DRV-311, had a body to bonnet leak. Disassembled valve, repaired steam cuts and reassembled with new gaskets. Had valve tested.
- M-5      The reciprocating charging pump discharge valve, CS-386, was leaking by. Disassembled valve and lapped seat. Reassembled with a new stem and plug.
- M-6      The sample pump for radiation monitors R-11 and R-12 would not run. Replaced the pump and had it tested.
- M-7      The motor driven auxiliary feedpump was inspected to check condition of the outboard pump bearing. Replaced the rotating assembly and tested the pump.
- M-8      The west centrifugal charging pump had a broken shaft. Replaced the rotating assembly and retested the pump.
- M-9      Safety injection check valves on the lines to the reactor coolant hot legs of all four loops, SI-158L1, L2, L3 and L4 had body to bonnet leaks. Replaced bonnet gaskets in all four valves.

10/10/10

Dear Sir,

I have the pleasure to acknowledge the receipt of your letter of the 10th inst. in relation to the above matter.

I am sorry to hear that you are having trouble with the machine. I will try to get it fixed as soon as possible.

I am sure that you will be satisfied with the results.

Very truly yours,

J. H. Smith