

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8006240491 DOC DATE: 80/06/18 NOTARIZED: NO DOCKET #  
 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
 AUTH.NAME AUTHOR AFFILIATION  
 DOLAN,J.E. Indiana & Michigan Electric Co.  
 RECIP.NAME RECIPIENT AFFILIATION  
 DENTON,H.R. Office of Nuclear Reactor Regulation

SUBJECT: Notifies that fourth refueling outage initiated on 800530  
 startup date for Cycle 5 operation scheduled for 800731.

DISTRIBUTION CODE: A001S COPIES RECEIVED:LTR 1 ENCL 0 SIZE: 2  
 TITLE: General Distribution for after Issuance of Operating Lic

NOTES: I+E: 3 COPIES ALL MATERIAL

ACTION:	RECIPIENT		COPIES		RECIPIENT	COPIES	
	ID CODE/NAME		LTTR	ENCL		ID CODE/NAME	LTTR
	BC	VARGA, S.	05				
			7	7			
INTERNAL:	A/D CORE & CS21		1	0	A/D REACT SYS18	1	0
	CHEM ENG BR	17	1	0	EMERG PREP	16	1
	HANAUER, S.	20	1	0	I&E	12	2
	MOORE, V.	22	1	0	NRC PDR	02	1
	OELD	14	1	0	OR ASSESS BR	19	1
	QA BR	15	1	0	<u>REG FILE</u>	01	1
EXTERNAL:	ACRS	20	16	1	LPDR	03	1
	NSIC	04	1	1			

JUN 25 1980

TOTAL NUMBER OF COPIES REQUIRED: LTTR

ENCL

MA  
4

60



1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

101 102 103 104 105 106 107 108 109 110 111 112 113 114 115 116 117 118 119 120 121 122 123 124 125 126 127 128 129 130 131 132 133 134 135 136 137 138 139 140 141 142 143 144 145 146 147 148 149 150

151 152 153 154 155 156 157 158 159 160 161 162 163 164 165 166 167 168 169 170 171 172 173 174 175 176 177 178 179 180 181 182 183 184 185 186 187 188 189 190 191 192 193 194 195 196 197 198 199 200

201 202 203 204 205 206 207 208 209 210 211 212 213 214 215 216 217 218 219 220 221 222 223 224 225 226 227 228 229 230 231 232 233 234 235 236 237 238 239 240 241 242 243 244 245 246 247 248 249 250

251 252 253 254 255 256 257 258 259 260 261 262 263 264 265 266 267 268 269 270 271 272 273 274 275 276 277 278 279 280 281 282 283 284 285 286 287 288 289 290 291 292 293 294 295 296 297 298 299 300

301 302 303 304 305 306 307 308 309 310 311 312 313 314 315 316 317 318 319 320 321 322 323 324 325 326 327 328 329 330 331 332 333 334 335 336 337 338 339 340 341 342 343 344 345 346 347 348 349 350 351 352 353 354 355 356 357 358 359 360 361 362 363 364 365 366 367 368 369 370 371 372 373 374 375 376 377 378 379 380 381 382 383 384 385 386 387 388 389 390 391 392 393 394 395 396 397 398 399 400

401 402 403 404 405 406 407 408 409 410 411 412 413 414 415 416 417 418 419 420 421 422 423 424 425 426 427 428 429 430 431 432 433 434 435 436 437 438 439 440 441 442 443 444 445 446 447 448 449 450 451 452 453 454 455 456 457 458 459 460 461 462 463 464 465 466 467 468 469 470 471 472 473 474 475 476 477 478 479 480 481 482 483 484 485 486 487 488 489 490 491 492 493 494 495 496 497 498 499 500

# INDIANA & MICHIGAN ELECTRIC COMPANY

P. O. BOX 18  
BOWLING GREEN STATION  
NEW YORK, N. Y. 10004

June 18, 1980  
AEP:NRC:00426

Donald C. Cook Nuclear Plant Unit No. 1  
Docket No. 50-315  
License No. DPR-58  
Refueling Outage Plans

Mr. Harold R. Denton  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Denton:

This is to advise you that Unit No. 1 of the Donald C. Cook Nuclear Plant has been brought to an orderly shutdown on May 30, 1980 for its fourth refueling outage. This outage is expected to last 60 days. The startup date for Cycle 5 operation is presently scheduled for July 31, 1980.

During this refueling outage 63 fuel assemblies manufactured by Exxon Nuclear Corporation (ENC) and 2 fuel assemblies manufactured by Westinghouse Electric Corporation will be removed and replaced with 65 fresh ENC fuel assemblies enriched to 2.9 w/o of U-235. The Cycle 5 core will be operated with ENC fuel assemblies. The mechanical and thermal hydraulic designs of the reload fuel are identical to those for the fuel from the previous reloads.

The fuel performance and safety analysis methods and procedures are the same as those used for the previous reloads. The results of these analysis remain unaltered for Cycle 5 core inasmuch as the Plant Safety is concerned.

A001  
S  
110

P 8006240 491

Trial	Control (%)	Alcohol (%)
1	85	75
2	80	65
3	85	75
4	88	78
5	90	80
6	92	82
7	93	83
8	94	84
9	95	85
10	95	85

*(continued)*

Mr. Harold R. Denton

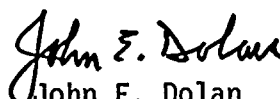
-2-

AEP:NRC:00426

Activities other than refueling include installation of new Steam Generator Feedwater elbows with thermal sleeves, water addition to the ice baskets and completion of the installation of a Unit - specific auxiliary feedwater system.

The AEP Service Corporation Nuclear Safety Design Review Committee (offsite review committee) and the Plant Nuclear Safety Review Committee have reviewed the reload design and the other activities associated with the refueling outage and concluded that these activities neither constitute unreviewed safety questions as defined in 10 CFR 50.59c nor would compromise the health and safety of the general public.

Very truly yours,

  
John E. Dolan  
Vice President

cc: R. C. Callen  
G. Charnoff  
R. S. Hunter  
R. W. Jurgensen  
D. V. Shaller - Bridgman