

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-316

UNIT 2

DATE 7/5/79

COMPLETED BY W.T. Gillett

TELEPHONE 616-465-5901

MONTH June 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>-</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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OPERATING DATA REPORT

DOCKET NO. 50-316
 DATE 7-5-79
 COMPLETED BY W. T. Gillett
 TELEPHONE 616-465-5901

OPERATING STATUS

1. Unit Name: Donald C. Cook 2
2. Reporting Period: June 1979
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	4,343	13,103
12. Number Of Hours Reactor Was Critical	0	3,260.4	8,491.1
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	3,203.7	7,931.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	10,558,347	23,663,454
17. Gross Electrical Energy Generated (MWH)	0	3,436,930	7,422,460
18. Net Electrical Energy Generated (MWH)	0	3,318,379	7,132,378
19. Unit Service Factor	0	73.8	75.5
20. Unit Availability Factor	0	73.8	75.5
21. Unit Capacity Factor (Using MDC Net)	0	70.6	68.0
22. Unit Capacity Factor (Using DER Net)	0	69.5	66.8
23. Unit Forced Outage Rate	100	26.2	17.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period. Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JUNE, 1979

DOCKET NO. 50-316
 UNIT NAME D.C. Cook-Unit 2
 DATE 7-11-79
 COMPLETED BY B.A. Svensson
 TELEPHONE (616) 465-5901

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
59	790519	F	720	B	1	79-019/01T-0	FW	PIPE XX	The Unit was removed from service on May 19, 1979 to repair cracks in the 16 in. feedwater elbows adjacent to the feedwater elbow/steam generator nozzle weld on the four steam generators. The Unit remained out of service at the end of the month.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

(9/77)

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error: list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

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For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-316
Unit Name: D. C. Cook Unit #2
Completed By: R. S. Lease
Telephone: (616) 465-5901
Date: July 13, 1979
Page: 1 of 1

MONTHLY OPERATING EXPERIENCES -- JUNE, 1979

Highlights

This Unit has been in Mode 5 operation the entire reporting period except as noted in the summary. Repairs and modifications of feedwater piping have been taking place. This problem was identified in the May report.

Summary

- 06/19/79 -- #3 Steam Generator was successfully hydrostatically tested after repair of its feedwater piping.
- 06/20/79 -- #1 Steam Generator was successfully hydrostatically tested after repair of its feedwater piping.
- 06/21/79 -- #2 Steam Generator was successfully hydrostatically tested after repair of its feedwater piping.
- 06/25/79 -- #4 Steam Generator was successfully hydrostatically tested after repair of its feedwater piping.
- 06/29/79 -- Heatup of the Unit commenced with entry into Mode 4 at 1225 hours.
- 06/30/79 -- The Reactor Plant entered Mode 3 at 0947 hours.

At 2000 hours excessive lake water in/leakage was detected in the Condensate System. The trouble was traced to several massive tube failures in the Main Condensers. Contamination was so great that it requires draining all the Condensate System and its storage tank.

The Reactor Plant was held in Mode 3 with Auxiliary Feedwater source from Unit 1.



[Faint handwritten notes at the bottom of the page]

DOCKET NO.	50 - 316
UNIT NAME	D. C. Cook - Unit No. 2
DATE	7-11-79
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901
PAGE	- 1 -

MAJOR SAFETY-RELATED MAINTENANCE

JUNE, 1979

- M-1 Non-essential service water to containment ventilation containment isolation check valve, NSW-417-4 failed to pass the leak rate test. Cleaned valve internals and had valve retested.
- M-2 Containment isolation valves for radiogas detector, ECR-31 and 32 failed to pass the leak rate test. Lapped valve seats, cleaned internals and had valves retested.
- M-3 Containment isolation air sample check valve failed to pass the leak rate test. Cleaned dirt from valve internals and reassembled.
- M-4 Containment ventilation isolation valve, VCR-201 failed to pass leak rate test. Valve internals were cleaned and valve was reassembled.
- M-5 Safety injection containment isolation valves, ICM-250 and 251 failed to pass leak rate tests. Lapped the discs and seats, cleaned internals and had valves retested.
- M-6 Ventilation system containment isolation valve, VCR-105 and 205 failed to pass leak rate tests. Cleaned and adjusted both valves and had them retested.
- M-7 Ice condenser refrigeration containment isolation check valves, R-156 and R-157 failed to pass leak rate tests. Cleaned valve internals and had them retested.
- M-8 Ventilation system containment isolation valves, VCR-104 and 204 failed to pass leak rate test. Adjusted both valves to obtain tighter seal when closed and had them retested.
- M-9 Safety feature safety valves discharge line containment isolation check valve, SI-189 failed to pass leak rate test. Cleaned valve internals and had valve retested.
- M-10 Service water containment isolation valves, WCR-962 and 967 failed to pass leak rate tests. Cleaned both valves internally and had them retested.
- M-11 Ventilation system containment isolation valves, VCR-103 and 203 failed to pass leak rate tests. Cleaned valve seats and adjusted for tighter seals when closed. Had valves retested.

DOCKET NO.	<u>50 - 316</u>
UNIT NAME	<u>D. C. Cook - Unit No. 2</u>
DATE	<u>7-11-79</u>
COMPLETED BY	<u>B. A. Svensson</u>
TELEPHONE	<u>(616) 465-5901</u>
PAGE	<u>- 2 -</u>

MAJOR SAFETY-RELATED MAINTENANCE

JUNE, 1979

- M-12 Non-essential service water containment isolation valve, WCR-930 failed to pass leak rate test. Replaced the valve seat and gaskets. Had valve retested.
- M-13 Containment ventilation drains containment isolation valves, DCR-620 and 621 failed to pass leak rate tests. Cleaned valve internals and replaced gaskets. The valves tested satisfactorily.



SECRET - UNCLASSIFIED

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