

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315  
 UNIT 1  
 DATE 6-4-79  
 COMPLETED BY W.T. Gillett  
 TELEPHONE 616-465-5901

MONTH May 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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# OPERATING DATA REPORT

DOCKET NO. 50-315  
 DATE 6-4-79  
 COMPLETED BY W.T. Gillett  
 TELEPHONE 616-465-5901

## OPERATING STATUS

1. Unit Name: Donald C. Cook 1
2. Reporting Period: May 1979
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1089
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	3,623	38,707
12. Number Of Hours Reactor Was Critical	0	2,211.9	29,423.6
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	0	2,205.1	28,618.4
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	0	6,942,086	78,279,877
17. Gross Electrical Energy Generated (MWH)	0	2,306,110	25,595,770
18. Net Electrical Energy Generated (MWH)	0	2,224,982	24,562,159
19. Unit Service Factor	0	60.9	76.3
20. Unit Availability Factor	0	60.9	76.3
21. Unit Capacity Factor (Using MDC Net)	0	58.8	67.4
22. Unit Capacity Factor (Using DER Net)	0	58.2	61.8
23. Unit Forced Outage Rate	0	3.7	6.0
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY, 1979

DOCKET NO. 50-315  
 UNIT NAME D.C. Cook-Unit  
 DATE 6-13-79  
 COMPLETED BY B.A. Svensson  
 TELEPHONE (616) 465-5901

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
149	790406	S	744.0	B,C	3	N.A.	ZZ	ZZZZZZ	The Unit was removed for Cycle III - IV refueling outage on April 6, 1979. The scheduled refueling outage has been extended to perform testing of concrete expansion anchor bolts per I.E. Bulletin No. 79-02 and to repair cracks identified in the main feedwater piping. The Unit remained out of service at the end of the month.

1  
 F: Forced  
 S: Scheduled

2  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

3  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

4  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

5  
 Exhibit I - Same Source

(9/77)

## INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely<sup>1</sup>. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

**NUMBER.** This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

**DATE.** This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

**TYPE.** Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

**DURATION.** Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

**REASON.** Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

**METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER.** Categorize by number designation

<sup>1</sup>Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

**LICENSEE EVENT REPORT.** Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

**SYSTEM CODE.** The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

**COMPONENT CODE.** Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

**CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE.** Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-315  
Unit Name: D. C. Cook Unit #1  
Completed By: R. S. Lease  
Telephone: (616) 465-5901  
Date: June 11, 1979  
Page: 1 of 1

MONTHLY OPERATING EXPERIENCES -- MAY, 1979

Highlights

This Unit was in its third annual refueling outage the entire reporting period.

Summary

It became necessary to remove the Reactor Vessel Head the second time due to possible fuel damage in locations C-8 and N-8. New secondary sources were installed in this area and after the head was reinstalled we were notified that the source assembly would not fit in these locations without possible fuel damage.

05/21/79 -- After cracks in feedwater lines of Unit 2 had been identified, radiographs were taken of the feedwater attachment to the Unit 1 Steam Generators and indication was that 3 of the 4 had developed cracks. The pipe elbow that attaches to the Steam Generator is being replaced on all 4 Steam Generators.

05/22/79 -- The Reactor Vessel Head was removed at 0930 hours.

The fuel assemblies in the C-8 and N-8 locations did indicate minor damage and they were replaced with fuel assemblies from a previous core. Of the two new source assemblies, one was damaged to the point that it could not be extracted from the fuel assembly. A third new source was used to replace the damaged one and the two sources were relocated in the core to positions B-7 and P-9.

05/27/79 -- The Reactor Vessel Head was again set down on the flange at 1710 hours.

DOCKET NO.	50 - 315
UNIT NAME	D. C. Cook - Unit No. 1
DATE	6-13-79
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901

MAJOR SAFETY-RELATED MAINTENANCE

MAY, 1979

- M-1      NESW containment isolation check valve, NSW-417-4, did not pass leak test. Cleaned valve internals.
- M-2      DCR-205 and DCR-206, containment isolation valves, did not pass leak test. The valves' diaphragms were replaced.
- M-3      North waste gas compressor would not operate. Replaced diaphragm on suction valve RRV-378. Retested satisfactorily.
- M-4      CCR-460 and 462, component cooling water containment isolation valves, did not pass leak test. Valves were disassembled, cleaned and retested satisfactorily.
- M-5      NSW-415-1, NESW containment isolation check valve did not pass leak test. Seats were lapped. Retest following repairs was satisfactory.
- M-6      Pressurizer relief valve, NRV-151, leaked by. Replaced seat ring and checked the seat ring fit to the plug. Retest was satisfactory.
- M-7      Minor surface indications in reactor vessel hot leg to safe-end welds on loops 2 and 3 were ground out. Necessary NDE was performed following grinding.
- M-8      Excess letdown heat exchanger component cooling water containment isolation valves, CCR-460 and 462 failed to pass leak rate test. Replaced the valve cages, plugs and stems, pins and gaskets. Valves retested satisfactorily.
- M-9      Auxiliary feedwater isolation valve to #2 steam generator, FMO-222 leaked by due to internal wear. Valve was removed from the line, returned to manufacturer and repaired. Valve was then reinstalled and tested satisfactorily. All required NDE including hydrostatic test was performed.
- M-10      Discharge valve from the boric acid blender to the volume control tank, QRV-451 was leaking by. Replaced the valve diaphragm. Valve retested satisfactorily.
- M-11      Letdown line from regenerative heat exchanger regulating valve, QRV-161, had a leaking diaphragm in the valve operator. Replaced operator diaphragm. Valve tested satisfactorily.
- M-12      The RHR pump discharge valve to the RCS cold legs, IMO-234, motor operator came loose from the valve and damaged power cable. Power cable was replaced and valve operation verified.