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SUBJECT: Forwards response to NRC Bulletin 96-002, "Movement of Heavy
 Loads Over Spent Fuel, Over Fuel in Reactor Core or
 Safety-Related Equipment."

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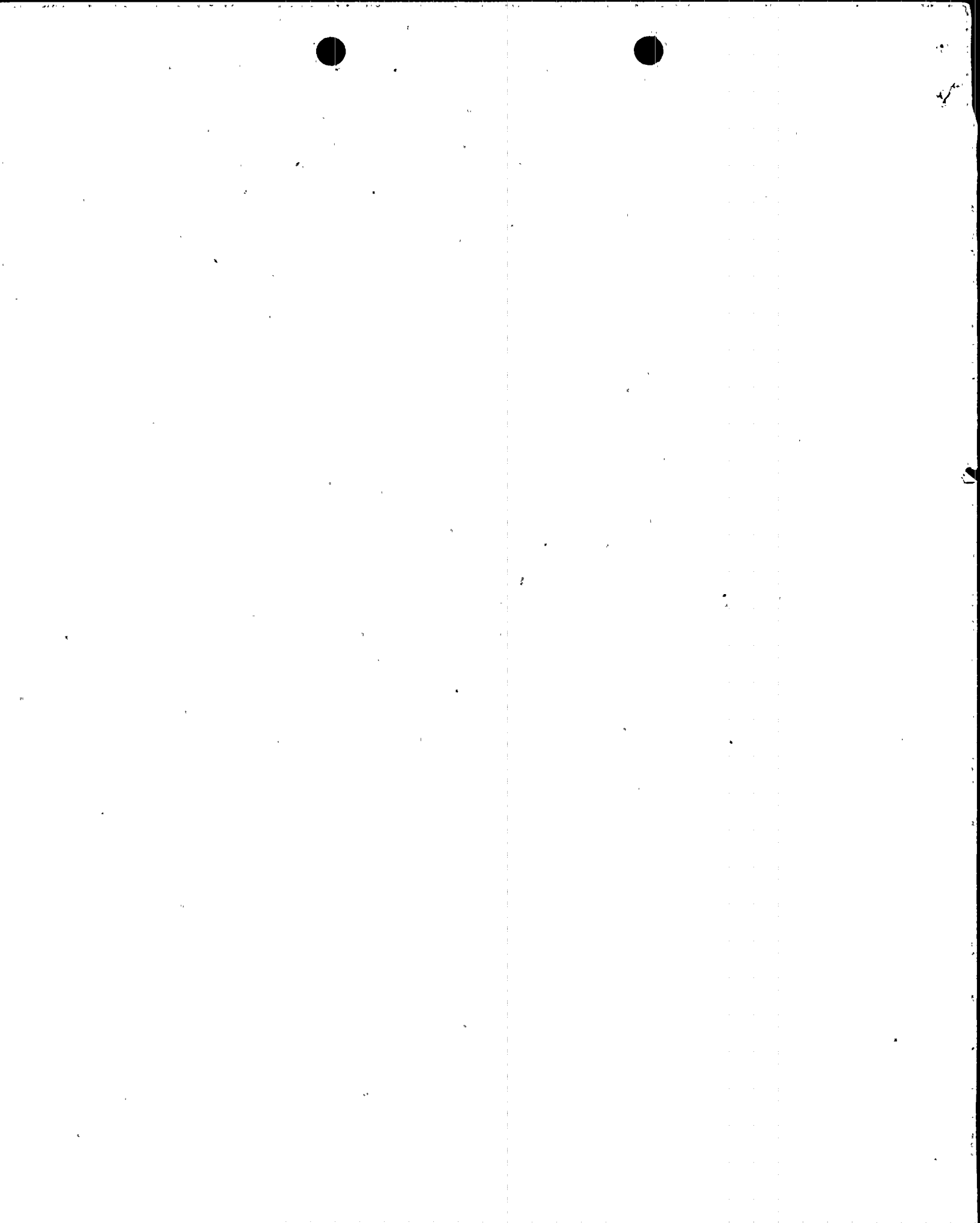
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Arizona Public Service

PALO VERDE NUCLEAR GENERATING STATION
P.O. BOX 52034 PHOENIX, ARIZONA 85072-2034

WILLIAM L. STEWART
EXECUTIVE VICE PRESIDENT
NUCLEAR

102-03733-WLS/AKK/DRL
July 16, 1996

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-37
Washington, DC 20555-0001

Dear Sirs:

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Units 1, 2, and 3
Docket Nos. STN 50-528/529/530
Response to NRC Bulletin 96-02, "Movement of Heavy Loads Over
Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related
Equipment," dated April 11, 1996

Enclosure 1 contains Arizona Public Service Company's (APS) response, as required by NRC Bulletin 96-02, concerning handling of heavy loads while at power.

APS confirms that the required review of plans and capabilities for handling of heavy loads while the reactor is at power, is complete. The review has determined that APS is in compliance with its current licensing basis. APS does not plan to request changes to its heavy load handling capability within the next two years.

Should you have any questions, please contact Scott A. Bauer at (602) 393-5978.

Sincerely,



WLS/AKK/DRL/rv

Enclosure:

cc: L. J. Callan (all w/Enclosure)
K. E. Perkins
J. W. Clifford
K. E. Johnston

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STATE OF ARIZONA)
) s.s..
COUNTY OF MARICOPA)

I, W. L. Stewart, represent that I am Executive Vice President - Nuclear, that the foregoing document has been signed by me on behalf of Arizona Public Service Company with my full authority to do so, and that to the best of my knowledge and belief, the statements made herein are true and correct.

W. L. Stewart

W. L. Stewart

Sworn To Before Me This 16 Day Of July, 1996

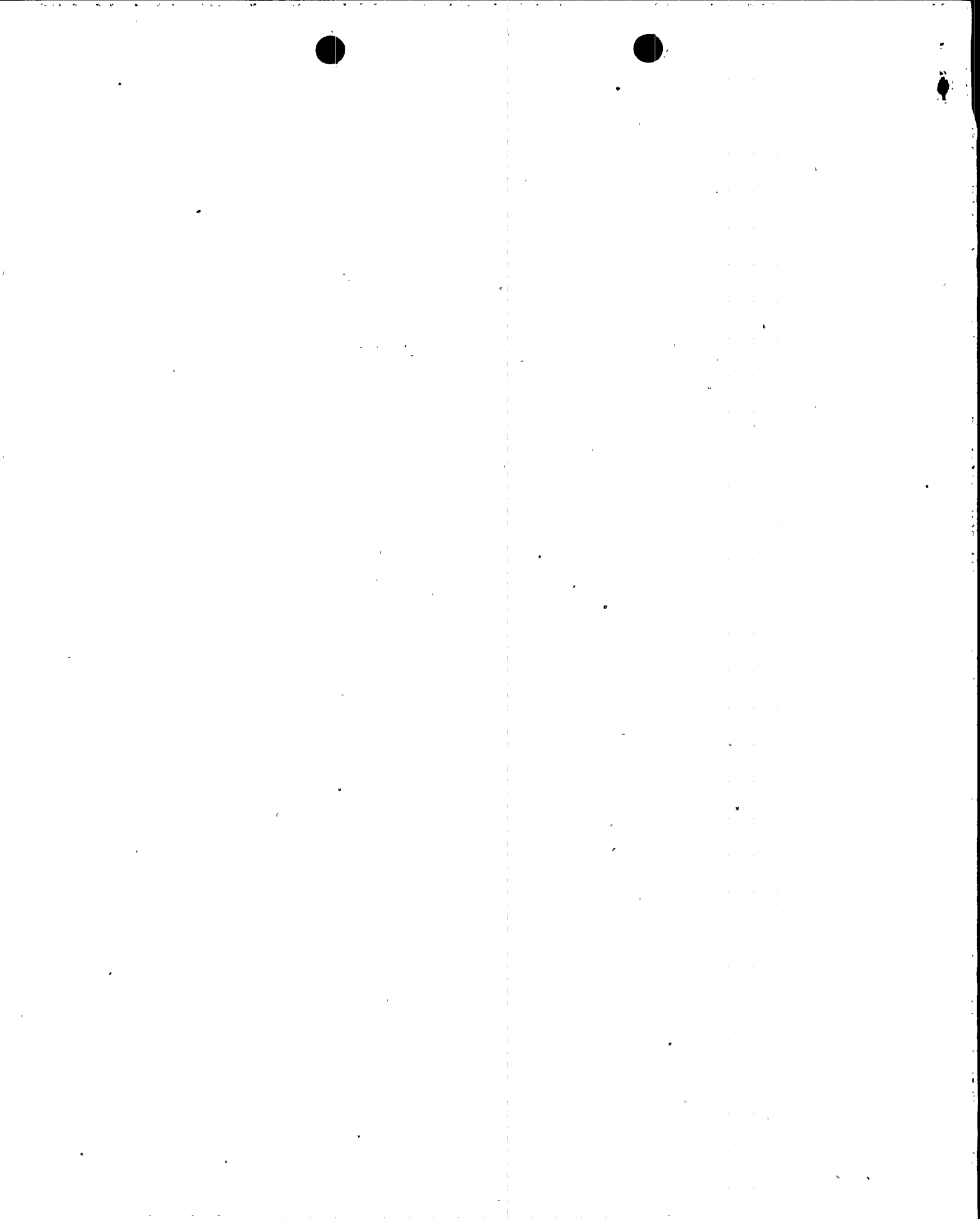
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Notary Public

My Commission Expires

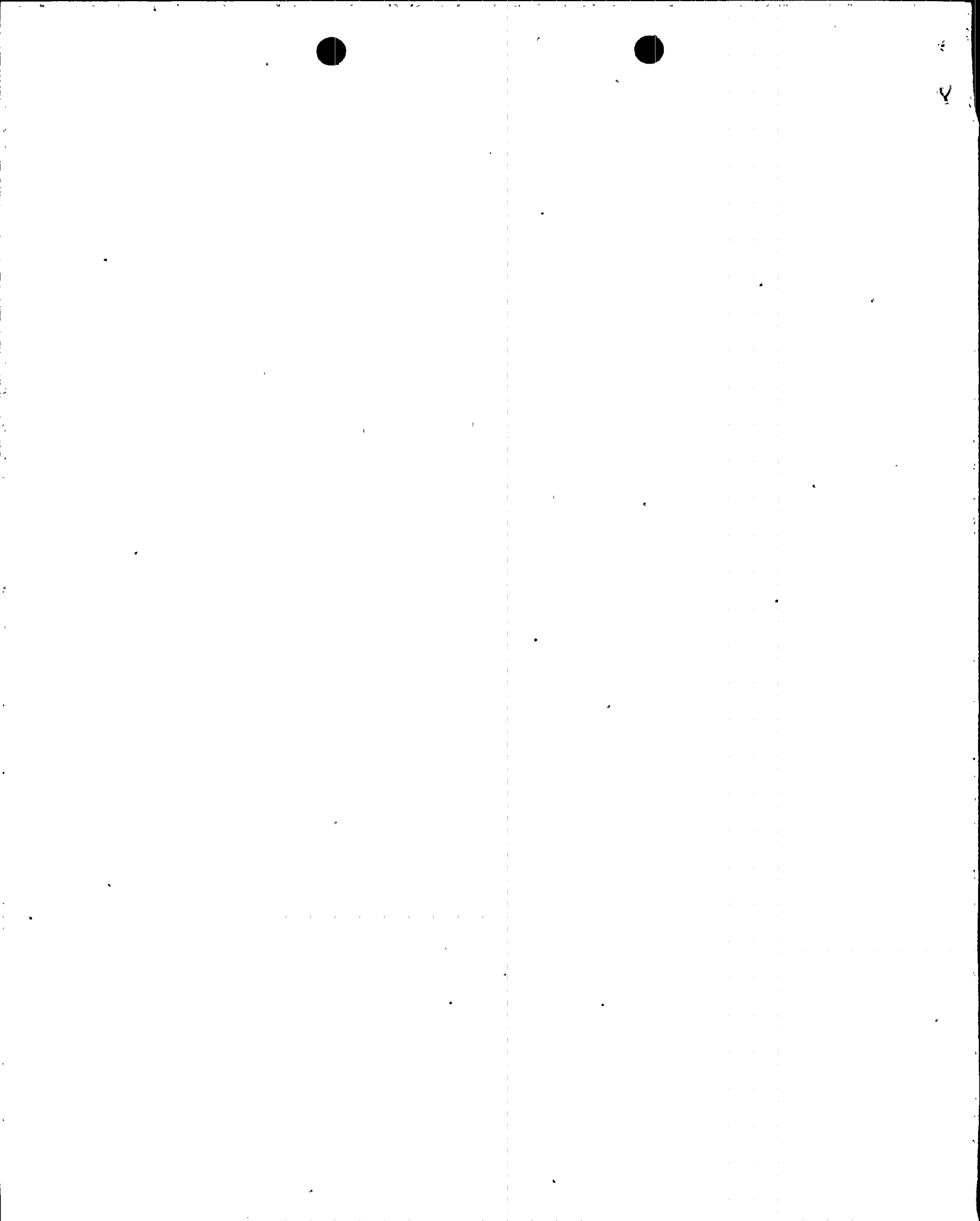
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ENCLOSURE 1

RESPONSE TO NRC BULLETIN 96-02



In accordance with the requested action of NRC Bulletin 96-02, APS has performed a review of plans and capabilities for handling heavy loads while the reactor is at power in accordance with existing regulatory guidance.

Required Response 1

Required Response 1 states:

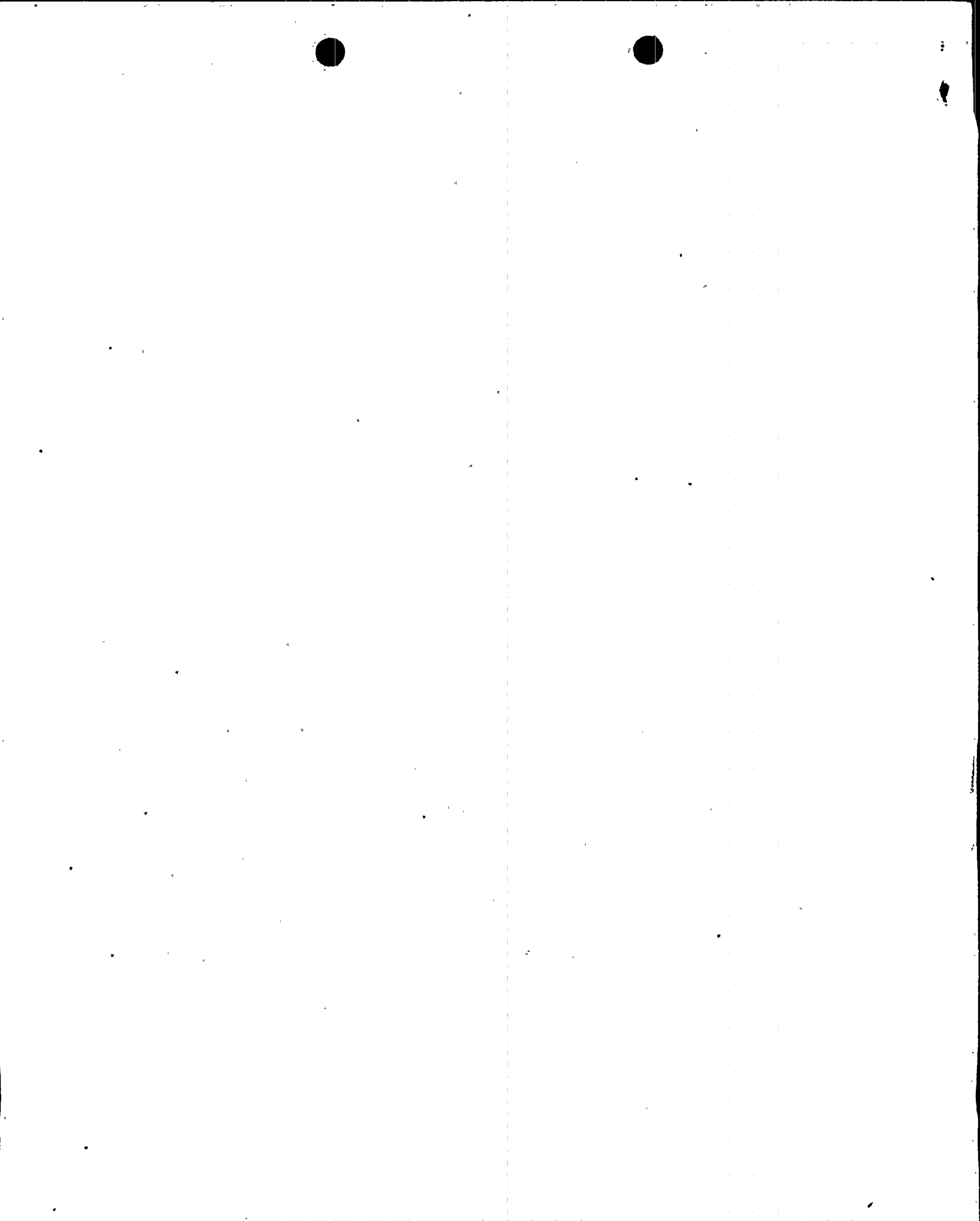
- (1) For licensees planning to implement activities involving the handling of heavy loads over spent fuel, fuel in the reactor core, or safety-related equipment within the next 2 years from the date of this bulletin, provide the following:
 - A report, within 30 days of the date of this bulletin, that addresses the licensee's review of its plans and capabilities to handle heavy loads while the reactor is at power (in all modes other than cold shutdown, refueling, and defueled) in accordance with existing regulatory guidelines. The report should also indicate whether the activities are within the licensing basis and should include, if necessary, a schedule for submission of a license amendment request. Additionally, the report should indicate whether changes to Technical Specifications will be required.

Response

NUREG-0857, "Safety Evaluation Report Related to the Operation of Palo Verde Nuclear Generating Station, Units 1, 2, and 3," Supplement 5, Appendix D, "Control of Heavy Loads at Nuclear Power Plants, Palo Verde Nuclear Generating Station, Plants Number 1, 2, and 3 (Phase I)" documents the acceptance of Palo Verde's control of heavy loads. This included a review of the establishment of safe load paths necessary to meet the requirements of Guideline 1 of NUREG-0612, Article 5.1.1(1) as well as the method for deviating from approved load paths. Supplement 7, Section 9.1.4 to the SER documents that Phase II of NUREG-0612 did not need to be implemented.

In order to ensure continued compliance with Guideline 1 of NUREG-0612, Article 5.1.1(1), APS performed a review of plant procedures used to implement safe load paths for heavy loads at PVNGS. The review has determined that load paths currently in use meet Guideline 1 of NUREG-0612, Article 5.1.1(1).

It is concluded that no license amendments or Technical Specification amendments will be required at this time.



Required Response 2

Required Response 2 states:

- (2) For licensees planning to perform activities involving the handling of heavy loads over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power (in all modes other than cold shutdown, refueling, and defueled) and that involve a potential load drop accident that has not previously been evaluated in the FSAR, submit a license amendment request in advance (6-9 months) of the planned movement of the loads so as to afford the staff sufficient time to perform an appropriate review.

Response

At the present time, APS does not plan on performing activities involving the handling of heavy loads over the spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power and that involve a potential load drop accident that has not been previously evaluated in the PVNGS licensing basis; i.e., the PVNGS and CESSAR SERs. Should the need arise, APS will submit a license amendment request in advance as requested in the Bulletin, as appropriate.

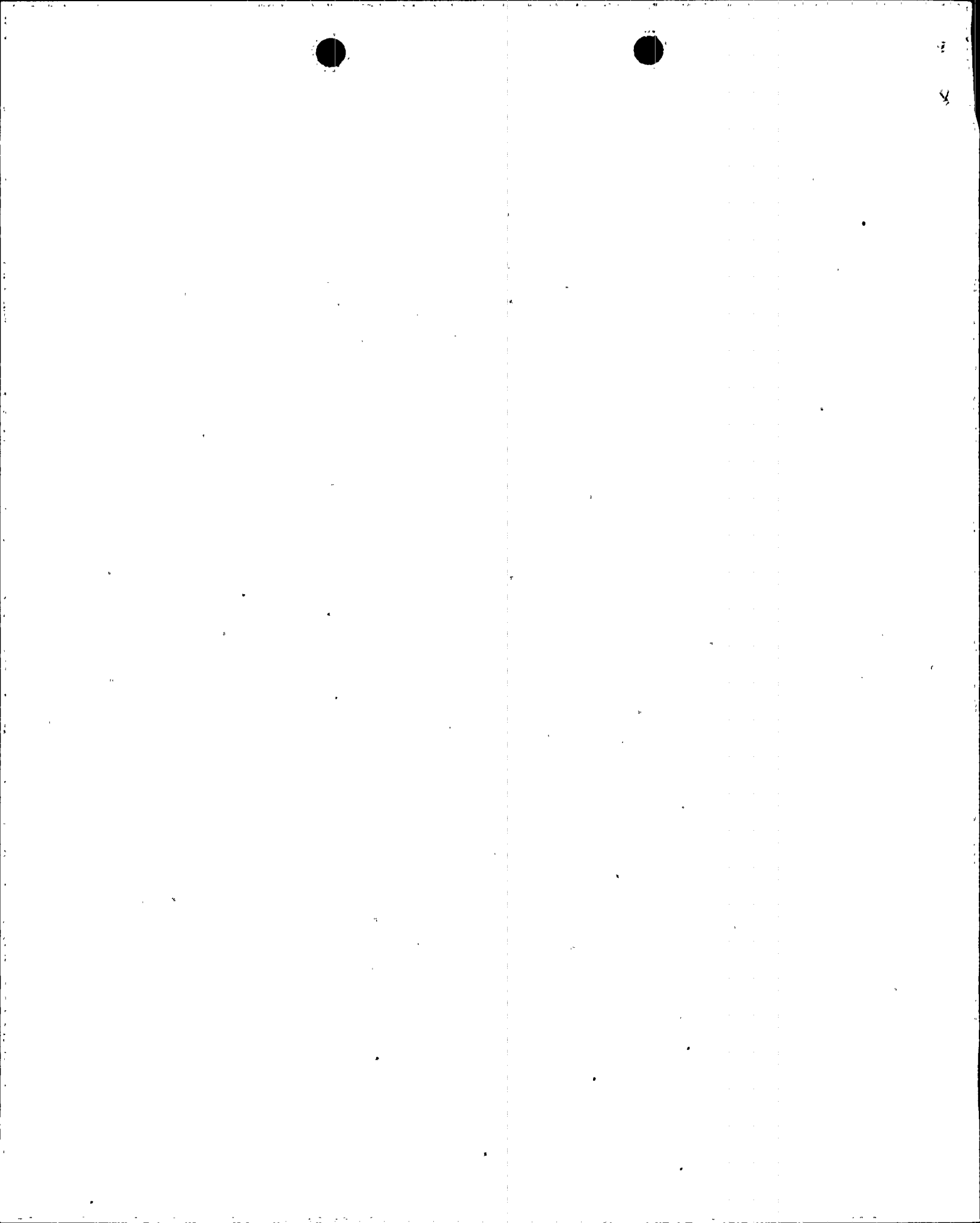
Required Response 3

Required Response 3 states:

- (3) For licensees planning to move dry storage casks over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power (in all modes other than cold shutdown, refueling, and defueled) included in item 2 above, a statement of the capability of performing the actions necessary for safe shutdown in the presence of radiological source term that may result from a breach of the dry cask, damage to the fuel, and damage to safety-related equipment as a result of a load drop inside the facility.

Response

PVNGS does not currently have plans for the movement of dry storage casks over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power. Appropriate evaluations will be performed, and submitted as necessary, when plans are made to utilize dry storage casks.



Required Response 4

Required Response 4 states:

- (4) For licensees planning to perform activities involving the handling of heavy loads over spent fuel, fuel in the reactor core, or safety-related equipment while the reactor is at power (in all modes other than cold shutdown, refueling, and defueled), determine whether changes to Technical Specifications will be required in order to allow the handling of heavy loads (e.g., the dry storage canister shield plug) over fuel assemblies in the spent fuel pool and submit the appropriate information in advance (6-9 months) of the planned movement of the loads for NRC review and approval.

Response

No changes to Technical Specifications are required at this time.

