

EOP: AP-PRZR.1	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 1 of 10
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ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

Richard L. King
RESPONSIBLE MANAGER

4-14-99
EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: _____

9904200387 990414
PDR ADDCK 05000244
PDR

EOP: AP-PRZR.1	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 2 of 10
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A. PURPOSE - This procedure provides the actions necessary to mitigate the consequences of abnormal PRZR pressure.

B. ENTRY CONDITIONS/SYMPTOMS

1. ENTRY CONDITIONS - This procedure is entered from:

- a. AP-CVCS.1, CVCS LEAK, or
- b. AP-FW.1, PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER, or
- c. AP-IA.1, LOSS OF INSTRUMENT AIR, or
- d. AP-RCC.1, CONTINUOUS CONTROL ROD WITHDRAWAL OR INSERTION, or
- e. AP-RCS.1, REACTOR COOLANT LEAK, or
- f. AP-TURB.1, TURBINE TRIP WITHOUT RX TRIP REQUIRED, or
- g. AP-TURB.2, TURBINE LOAD REJECTION, when PRZR pressure can NOT be controlled.

2. SYMPTOMS - The symptoms of ABNORMAL PRZR PRESSURE are;

- a. Annunciator F-19, PRZR PORV OUTLET HI TEMP 145°F, lit, or
- b. Annunciator F-18, PRZR SAFETY VLV OUTLET HI TEMP 145°F, lit, or
- c. Annunciator AA-13, PRESSURIZER SAFETY VALVE POSITION, lit, or
- d. Annunciator F-1, PRT LIQUID HI TEMP 220°F, lit, or
- e. Annunciator F-9, PRT HI PRESS 5 PSI, lit, or
- f. Annunciator F-17, PRT LEVEL 60.8 % 84.5, lit, or
- g. Annunciator F-2, PRESSURIZER HI PRESS 2310 PSI, lit, or
- h. Annunciator F-10, PRESSURIZER LO PRESS 2205 PSI, lit, or
- i. Annunciator F-6, PRESSURIZER HEATER BREAKER TRIP, lit.
- j. Annunciator F-26, PRESSURIZER HI PRESS CHANNEL ALERT 2377 PSI.

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 3 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
<p>*****</p> <p style="text-align: center;"><u>CAUTION</u></p> <p>IF, AT ANY TIME DURING THIS PROCEDURE, A REACTOR TRIP OR SI OCCURS, E-0, REACTOR TRIP OR SAFETY INJECTION, SHALL BE PERFORMED.</p> <p>*****</p> <p><u>NOTE:</u> o Actual PRZR pressure should be verified by more than 1 indicator.</p> <p> o Refer to ITS section 3.4.1 for limiting conditions for operations.</p>		
1	<p>Check PRZR Pressure:</p> <ul style="list-style-type: none"> o All 4 narrow range channels - APPROXIMATELY EQUAL o All 4 narrow range channels - TRENDING TOGETHER 	<p><u>IF</u> one pressure channel deviates significantly from the other 3, <u>THEN</u> perform the following:</p> <ul style="list-style-type: none"> a. <u>IF</u> the controlling PRZR pressure channel has failed, <u>THEN</u> place controller, 431K, in MANUAL and adjust output to restore PRZR pressure. b. Refer to ER-INST.1, REACTOR PROTECTION BISTABLE DEFEAT AFTER INSTRUMENTATION LOOP FAILURE.
2	<p>Check Reactor Power - STABLE</p>	<p><u>IF</u> the abnormal PRZR pressure is a result of a power transient, <u>THEN</u> evaluate conditions and go to the appropriate plant procedure.</p>

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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3 Check PRZR Pressure:

- | | |
|--------------------------------------|---|
| a. Pressure - LESS THAN 2235 PSIG | a. Go to Step 12. |
| b. Pressure - GREATER THAN 2000 PSIG | b. <u>IF</u> pressure decreasing uncontrollably, <u>THEN</u> trip the reactor and go to E-0, REACTOR TRIP <u>OR</u> SAFETY INJECTION. |

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CAUTION

OBSERVE D/G LOADING LIMITS OF 2300 KW FOR 1/2 HOUR, 2250 KW FOR 2 HOURS, AND 1950 KW FOR CONTINUOUS SERVICE.

.....

4 Check PRZR Heater Status:

- | | |
|--|---|
| a. PRZR heater control group -
BREAKER CLOSED | a. Place PRZR heater control group switch to TRIP to reset, then place switch to CLOSE and let return to AUTO. |
| b. PRZR heater backup group - ON | b. <u>IF</u> PRZR pressure less than 2220 psig, <u>THEN</u> energize PRZR backup heaters.

<u>IF</u> PRZR backup heater breaker has tripped, <u>THEN</u> perform the following:

1) Place the breaker switch to OFF to reset breaker.

2) Place the breaker switch to ON to energize heaters.

3) Verify load increase on Bus 16. |

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 5 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
5	<p>Verify Normal PRZR Spray Valves - CLOSED</p> <ul style="list-style-type: none"> • PCV-431A • PCV-431B 	<p>Place controllers in MANUAL at 0% demand. <u>IF</u> valves can <u>NOT</u> be closed, <u>THEN</u> perform the following:</p> <ol style="list-style-type: none"> Trip the reactor. Trip the associated RCP. Go to E-0, REACTOR TRIP <u>OR</u> SAFETY INJECTION.
<p><u>NOTE:</u> With PRZR pressure controller 431K in manual, PORV-431C will not be operable in the automatic mode.</p>		
6	<p>Check PRZR Pressure Controller, 431K, Demand - LESS THAN 50%</p>	<p>Place 431K in MANUAL and decrease output to energize PRZR heaters.</p>

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 6 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
7	Check PRZR PORVs:	
a.	PORVs - CLOSED	<p>a. Manually close PORVs.</p> <p><u>IF</u> any valve can <u>NOT</u> be closed, <u>THEN</u> manually close the associated block valve.</p> <ul style="list-style-type: none"> • MOV-516 for PCV-430 • MOV-515 for PCV-431C
b.	Annunciator F-19, PRZR PORV OUTLET HI TEMP 145°F - EXTINGUISHED	<p>b. <u>IF</u> PORV leakage is indicated, <u>THEN</u> perform the following:</p> <ol style="list-style-type: none"> 1) Close PORV block valves one at a time <u>AND</u> check if pressure stabilizes. <ul style="list-style-type: none"> • MOV-515 • MOV-516 2) <u>IF</u> a leaking PRZR PORV is identified, <u>THEN</u> restore any nonleaking PORV to operable <u>AND</u> go to Step 8.
c.	Go to Step 9	

EOP: AP-PRZR.1	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 7 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
8	Attempt To Reseat Any Leaking PORV:	
	a. Verify affected PORV block valve - CLOSED	a. Close the affected PORV block valve.
	b. Cycle the leaking PORV open then closed	
	c. Open affected PORV block valve	
	d. Verify leakage has stopped	d. <u>IF</u> leakage continues, <u>THEN</u> perform the following: 1) Reclose leaking PORV block valve. 2) Refer to ITS section 3.4.11 for required actions.
	e. Go to Step 11	
9	Check PRZR Safety Valves:	<u>IF</u> safety valve leakage indicated, <u>THEN</u> perform the following:
	o Position indicator - LESS THAN 0.1 INCH	a. Verify leakage within limits of ITS section 3.4.13.
	o Annunciator F-18, PRZR SAFETY VLV OUTLET HI TEMP 145°F - EXTINGUISHED	b. Go to Step 17.
	o Annunciator AA-13, PRESSURIZER SAFETY VALVE POSITION - EXTINGUISHED	

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 8 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
10	Check AUX Spray Valve, AOV-296 - CLOSED	Manually close valve. <u>IF</u> valve can <u>NOT</u> be closed, <u>THEN</u> consider isolating letdown and closing charging flow control valve, HCV-142.
11	Check PRZR Pressure Control Restored: a. Pressure - TRENDING TO 2235 PSIG b. Go to Step 17	a. <u>IF</u> pressure continues to decrease, <u>THEN</u> return to Step 1.
12	Check PRZR Backup Heaters - OFF	Verify PRZR backup heater switch in AUTO, <u>AND</u> ensure proper operation.
<u>NOTE:</u> With PRZR pressure controller 431K in manual, PORV-431C will not be operable in the automatic mode.		
13	Check PRZR Pressure Controller, 431K, Demand - INCREASING	Place 431K in MANUAL at 50% demand.
<u>NOTE:</u> o If auxiliary spray is in use, spray flow may be increased by closing normal charging valve AOV-294 and normal PRZR spray valves. o The ΔT between the REGEN Hx charging outlet and the PRZR shall NOT exceed 320°F.		
14	Check RCPs - ANY RUNNING	<u>IF</u> letdown in service, <u>THEN</u> perform the following: a. Open AUX spray valve, AOV-296, as necessary to control PRZR pressure. b. Go to Step 16.

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 9 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
<p><u>NOTE:</u> If either RCP is off, then the associated PRZR spray valve controller should be closed by placing in MANUAL with demand at 0%.</p>		
15	<p>Check PRZR Spray Valve Controllers Demand - GREATER THAN 0%</p> <ul style="list-style-type: none"> • PCV-431A • PCV-431B 	<p>Place PRZR spray valve controllers in MANUAL and open as required to control pressure.</p>
16	<p>Check PRZR Pressure - TRENDING TO 2235 PSIG</p>	<p><u>IF</u> PRZR pressure continues to increase uncontrollably, <u>THEN</u> trip the reactor and go to E-0, REACTOR TRIP or SAFETY INJECTION.</p>
17	<p>Check PRT Indications:</p> <ul style="list-style-type: none"> a. Level - BETWEEN 61% AND 84% b. Pressure - APPROXIMATELY 1.5 PSIG AND STABLE c. Temperature - AT CNMT AMBIENT TEMPERATURE AND STABLE 	<ul style="list-style-type: none"> a. Drain PRT to RCDT using PRT drain valve, AOV-526. b. Open PRT vent, AOV-527. <u>IF</u> PRT pressure will <u>NOT</u> decrease, <u>THEN</u> open PRT drain valve, AOV-526, to reduce pressure. c. To feed and bleed the PRT, perform the following: <ul style="list-style-type: none"> 1) Start a RMW pump. 2) Verify RMW to CNMT isolation valve, AOV-508, open. 3) Feed and bleed the PRT using PRT fill valve from RMW, AOV-548, and PRT drain valve, AOV-526.

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 10 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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NOTE: Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.

18 Notify Higher Supervision

19 Notify Reactor Engineer for
Transient Monitoring Program

-END-

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 1 of 10
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Robert L. Orm
RESPONSIBLE MANAGER

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9904200389 990414
PDR ADDCK 05000244
PDR

GOP: AP-PRZR.1	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 2 of 10
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A. PURPOSE - This procedure provides the actions necessary to mitigate the consequences of abnormal PRZR pressure.

B. ENTRY CONDITIONS/SYMPTOMS

1. ENTRY CONDITIONS - This procedure is entered from:

- a. AP-CVCS.1, CVCS LEAK, or
- b. AP-FW.1, PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER, or
- c. AP-IA.1, LOSS OF INSTRUMENT AIR, or
- d. AP-RCC.1, CONTINUOUS CONTROL ROD WITHDRAWAL OR INSERTION, or
- e. AP-RCS.1, REACTOR COOLANT LEAK, or
- f. AP-TURB.1, TURBINE TRIP WITHOUT RX TRIP REQUIRED, or
- g. AP-TURB.2, TURBINE LOAD REJECTION, when PRZR pressure can NOT be controlled.

2. SYMPTOMS - The symptoms of ABNORMAL PRZR PRESSURE are;

- a. Annunciator F-19, PRZR PORV OUTLET HI TEMP 145°F, lit, or
- b. Annunciator F-18, PRZR SAFETY VLV OUTLET HI TEMP 145°F, lit, or
- c. Annunciator AA-13, PRESSURIZER SAFETY VALVE POSITION, lit, or
- d. Annunciator F-1, PRT LIQUID HI TEMP 220°F, lit, or
- e. Annunciator F-9, PRT HI PRESS 5 PSI, lit, or
- f. Annunciator F-17, PRT LEVEL 60.8 % 84.5, lit, or
- g. Annunciator F-2, PRESSURIZER HI PRESS 2310 PSI, lit, or
- h. Annunciator F-10, PRESSURIZER LO PRESS 2205 PSI, lit, or
- i. Annunciator F-6, PRESSURIZER HEATER BREAKER TRIP, lit.
- j. Annunciator F-26, PRESSURIZER HI PRESS CHANNEL ALERT 2377 PSI.

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 3 of 10
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STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

IF, AT ANY TIME DURING THIS PROCEDURE, A REACTOR TRIP OR SI OCCURS, E-0, REACTOR TRIP OR SAFETY INJECTION, SHALL BE PERFORMED.

- *****
- NOTE:
- o Actual PRZR pressure should be verified by more than 1 indicator.
 - o Refer to ITS section 3.4.1 for limiting conditions for operations.

1 Check PRZR Pressure:

- o All 4 narrow range channels - APPROXIMATELY EQUAL
- o All 4 narrow range channels - TRENDING TOGETHER

IF one pressure channel deviates significantly from the other 3, THEN perform the following:

- a. IF the controlling PRZR pressure channel has failed, THEN place controller, 431K, in MANUAL and adjust output to restore PRZR pressure.
- b. Refer to ER-INST.1, REACTOR PROTECTION BISTABLE DEFEAT AFTER INSTRUMENTATION LOOP FAILURE.

2 Check Reactor Power - STABLE

IF the abnormal PRZR pressure is a result of a power transient, THEN evaluate conditions and go to the appropriate plant procedure.

SOP: AP-PRZR.1	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 4 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3 Check PRZR Pressure:	a. Pressure - LESS THAN 2235 PSIG b. Pressure - GREATER THAN 2000 PSIG	a. Go to Step 12. b. <u>IF</u> pressure decreasing uncontrollably, <u>THEN</u> trip the reactor and go to E-0, REACTOR TRIP <u>OR</u> SAFETY INJECTION.
..... <div data-bbox="768 714 900 756" data-label="Section-Header"> <p style="text-align: center;"><u>CAUTION</u></p> </div> <div data-bbox="214 777 1462 850" data-label="Text"> <p>OBSERVE D/G LOADING LIMITS OF 2300 KW FOR 1/2 HOUR, 2250 KW FOR 2 HOURS, AND 1950 KW FOR CONTINUOUS SERVICE.</p> </div>		
4 Check PRZR Heater Status:	a. PRZR heater control group - BREAKER CLOSED b. PRZR heater backup group - ON	a. Place PRZR heater control group switch to TRIP to reset, then place switch to CLOSE and let return to AUTO. b. <u>IF</u> PRZR pressure less than 2220 psig, <u>THEN</u> energize PRZR backup heaters. <u>IF</u> PRZR backup heater breaker has tripped, <u>THEN</u> perform the following: <ol style="list-style-type: none"> 1) Place the breaker switch to OFF to reset breaker. 2) Place the breaker switch to ON to energize heaters. 3) Verify load increase on Bus 16.

EOP: AP-PRZR.1-	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 5 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
5	Verify Normal PRZR Spray Valves - CLOSED <ul style="list-style-type: none">• PCV-431A• PCV-431B	Place controllers in MANUAL at 0% demand. <u>IF</u> valves can <u>NOT</u> be closed, <u>THEN</u> perform the following: <ul style="list-style-type: none">a. Trip the reactor.b. Trip the associated RCP.c. Go to E-0, REACTOR TRIP <u>OR</u> SAFETY INJECTION.
<u>NOTE:</u> With PRZR pressure controller 431K in manual, PORV-431C will not be operable in the automatic mode.		
6	Check PRZR Pressure Controller, 431K, Demand - LESS THAN 50%	Place 431K in MANUAL and decrease output to energize PRZR heaters.

EOP:	TITLE:	REV: 10
AP-PRZR.1.	ABNORMAL PRESSURIZER PRESSURE	PAGE 6 of 10

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

7 Check PRZR PORVs:

a. PORVs - CLOSED

a. Manually close PORVs.

IF any valve can NOT be closed,
THEN manually close the
associated block valve.

- MOV-516 for PCV-430
- MOV-515 for PCV-431C

b. Annunciator F-19, PRZR PORV
OUTLET HI TEMP 145°F -
EXTINGUISHED

b. IF PORV leakage is indicated,
THEN perform the following:

1) Close PORV block valves one
at a time AND check if
pressure stabilizes.

- MOV-515
- MOV-516

2) IF a leaking PRZR PORV is
identified, THEN restore any
nonleaking PORV to operable
AND go to Step 8.

c. Go to Step 9

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

8 Attempt To Reseat Any Leaking PORV:

- a. Verify affected PORV block valve
- CLOSED
- b. Cycle the leaking PORV open then closed
- c. Open affected PORV block valve
- d. Verify leakage has stopped

- a. Close the affected PORV block valve.

- d. IF leakage continues, THEN perform the following:

- 1). Reclose leaking PORV block valve.
- 2) Refer to ITS section 3.4.11 for required actions.

- e. Go to Step 11

9 Check PRZR Safety Valves:

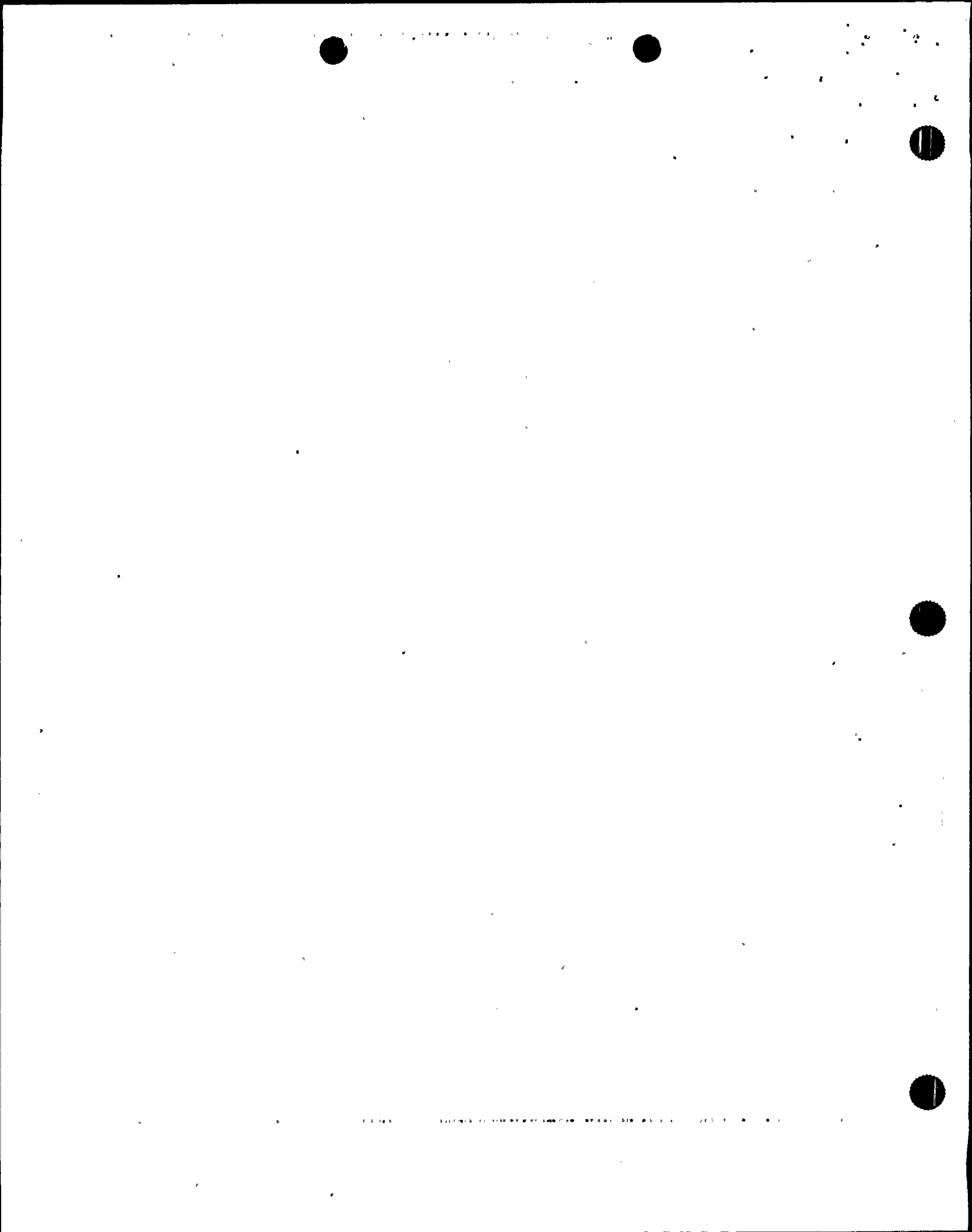
- o Position indicator - LESS THAN 0.1 INCH
- o Annunciator F-18, PRZR SAFETY VLV OUTLET HI TEMP 145°F - EXTINGUISHED
- o Annunciator AA-13, PRESSURIZER SAFETY VALVE POSITION - EXTINGUISHED

- IF safety valve leakage indicated, THEN perform the following:

- a. Verify leakage within limits of ITS section 3.4.13.
- b. Go to Step 17.

EOP:	TITLE:	REV: 10
AP-PRZR.1.	ABNORMAL PRESSURIZER PRESSURE	PAGE 8 of 10

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
10	Check AUX Spray Valve, AOV-296 - CLOSED	Manually close valve. <u>IF</u> valve can <u>NOT</u> be closed, <u>THEN</u> consider isolating letdown and closing charging flow control valve, HCV-142.
11	Check PRZR Pressure Control Restored: a. Pressure - TRENDING TO 2235 PSIG b. Go to Step 17	a. <u>IF</u> pressure continues to decrease, <u>THEN</u> return to Step 1.
12	Check PRZR Backup Heaters - OFF	Verify PRZR backup heater switch in AUTO, <u>AND</u> ensure proper operation.
<u>NOTE:</u> With PRZR pressure controller 431K in manual, PORV-431C will not be operable in the automatic mode.		
13	Check PRZR Pressure Controller, 431K, Demand - INCREASING	Place 431K in MANUAL at 50% demand.
<u>NOTE:</u> o If auxiliary spray is in use, spray flow may be increased by closing normal charging valve AOV-294 and normal PRZR spray valves. o The ΔT between the REGEN Hx charging outlet and the PRZR shall NOT exceed 320°F.		
14	Check RCPs - ANY RUNNING	<u>IF</u> letdown in service, <u>THEN</u> perform the following: a. Open AUX spray valve, AOV-296, as necessary to control PRZR pressure. b. Go to Step 16.



EOP:	TITLE:	REV: 10
AP-PRZR.1.	ABNORMAL PRESSURIZER PRESSURE	PAGE 9 of 10

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: If either RCP is off, then the associated PRZR spray valve controller should be closed by placing in MANUAL with demand at 0%.

15 Check PRZR Spray Valve
Controllers Demand - GREATER
THAN 0%

- PCV-431A
- PCV-431B

Place PRZR spray valve controllers in MANUAL and open as required to control pressure.

16 Check PRZR Pressure -
TRENDING TO 2235 PSIG

IF PRZR pressure continues to increase uncontrollably, THEN trip the reactor and go to E-0, REACTOR TRIP or SAFETY INJECTION.

17 Check PRT Indications:

a. Level - BETWEEN 61% AND 84%

b. Pressure - APPROXIMATELY
1.5 PSIG AND STABLE

c. Temperature - AT CNMT AMBIENT
TEMPERATURE AND STABLE

a. Drain PRT to RCDT using PRT
drain valve, AOV-526.

b. Open PRT vent, AOV-527. IF PRT pressure will NOT decrease, THEN open PRT drain valve, AOV-526, to reduce pressure.

c. To feed and bleed the PRT,
perform the following:

1) Start a RMW pump.

2) Verify RMW to CNMT isolation
valve, AOV-508, open.

3) Feed and bleed the PRT using
PRT fill valve from RMW,
AOV-548, and PRT drain valve,
AOV-526.

EOP: AP-PRZR.1.	TITLE: ABNORMAL PRESSURIZER PRESSURE	REV: 10 PAGE 10 of 10
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	<p><u>NOTE:</u> Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.</p>	
18	Notify Higher Supervision	
19	Notify Reactor Engineer for Transient Monitoring Program	
	-END-	

Distribution Sheet

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A002 - OR Submittal:Inadequate Core Cooling (Item II.F.2) GL 82-28

Docket: 05000244



A Subsidiary of RGS Energy Group, Inc.

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JOSEPH A. WIDAY
VICE PRESIDENT & PLANT MANAGER
GINNA STATION

March 15, 2001

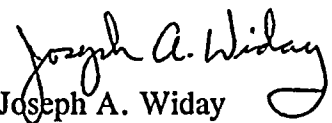
U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,


Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index
AP-ELEC.14/16

A002

ML010860442

REPORT NO. 01
REPORT: NPS0200
DOC TYPE: PRAP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
ABNORMAL PROCEDURE

02/08/01 PAGE: 1

PARAMETERS: DOC TYPES - PRFIG PRER PRAR PRAP

STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	014	01/09/01	05/01/98	05/01/03	EF
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	014	05/18/00	08/17/99	08/17/04	EF
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	012	05/18/00	08/17/99	08/17/04	EF
AP-CR.1	CONTROL ROOM INACCESSIBILITY	016	01/11/00	01/11/00	01/11/05	EF
AP-CVCS.1	CVCS LEAK	012	05/01/98	05/01/98	05/01/03	EF
AP-CVCS.3	LOSS OF ALL CHARGING FLOW	002	02/11/00	02/26/99	02/26/04	EF
AP-CW.1	LOSS OF A CIRC WATER PUMP	010	07/16/98	05/01/98	05/01/03	EF
AP-ELEC.1	LOSS OF 12A AND/OR 12B BUSES	020	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.2	SAFEGUARD BUSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	009	03/22/99	03/22/99	03/22/04	EF
AP-ELEC.3	LOSS OF 12A AND/OR 12B TRANSFORMER (BELOW 350 F)	008	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	002	02/08/01	06/09/97	06/09/02	EF
AP-ELEC.17/18	LOSS OF SAFEGUARDS BUS 17/18	002	10/18/99	06/09/97	06/09/02	EF
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	012	02/11/00	02/27/98	02/27/03	EF
AP-IA.1	LOSS OF INSTRUMENT AIR	017	12/02/99	05/01/98	05/01/03	EF
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	011	12/02/99	12/02/99	12/02/04	EF
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	006	02/24/96	05/14/98	05/14/03	EF
AP-RCC.2	RCC/RPI MALFUNCTION	008	11/16/98	02/06/97	02/06/02	EF
AP-RCC.3	DROPPED ROD RECOVERY	004	11/16/98	02/27/98	02/27/03	EF
AP-RCP.1	RCP SEAL MALFUNCTION	013	06/09/00	05/01/98	05/01/03	EF
AP-RCS.1	REACTOR COOLANT LEAK	015	09/08/00	05/01/98	05/01/03	EF
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	010	12/14/98	05/01/98	05/01/03	EF
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	007	08/05/97	08/05/97	08/05/02	EF
AP-RCS.4	SHUTDOWN LOCA	011	12/02/99	05/01/98	05/01/03	EF
AP-RHR.1	LOSS OF RHR	015	02/08/01	05/01/98	05/01/03	EF

REPORT NO. 01
REPORT: NPSP0200
DOC TYPE: PRAP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
ABNORMAL PROCEDURE

02/08/01 PAGE: 2

PARAMETERS: DOC TYPES - PRFIG PRER PRAR PRAP STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	009	10/13/00	03/31/00	03/31/05	EF
AP-SG.1	STEAM GENERATOR TUBE LEAK	000	09/08/00	09/08/00	09/08/05	EF
AP-SW.1	SERVICE WATER LEAK	015	10/18/99	06/03/98	06/03/03	EF
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	010	02/12/99	10/10/97	10/10/02	EF
AP-TURB.2	TURBINE LOAD REJECTION	017	02/11/00	05/13/98	05/13/03	EF
AP-TURB.3	TURBINE VIBRATION	010	02/11/00	02/10/98	02/10/03	EF
AP-TURB.4	LOSS OF CONDENSER VACUUM	014	05/01/98	05/01/98	05/01/03	EF
AP-TURB.5	RAPID LOAD REDUCTION	005	06/09/00	06/09/00	06/09/05	EF
TOTAL FOR PRAP	32					

EQP:

AP-ELEC.14/16

TITLE:

LOSS OF SAFEGUARDS BUS 14/16

REV: 2

PAGE 1 of 17

ROCHESTER GAS AND ELECTRIC CORPORATION

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CONTROLLED COPY NUMBER

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RESPONSIBLE MANAGER

2-8-2001
EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: _____

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 2 PAGE 2 of 17
-----------------------	--	------------------------

- A. PURPOSE - This procedure provides actions to respond to a loss of AC Emergency Bus 14 or Bus 16.
- B. ENTRY CONDITIONS/SYMPTOMS
 - 2. SYMPTOMS - The symptoms of a LOSS OF SAFEGUARDS BUS 14/16 are;
 - a. Annunciator J-7, 480V MAIN OR TIE BREAKER TRIP, lit, or
 - b. Annunciator J-29, 480V TRANSFORMER BREAKER TRIP, lit.

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 2 PAGE 3 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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CAUTION

- o IF A LOSS OF BUS 12A OR 12B HAS OCCURRED, THEN NO OUTSIDE, SHOULD BE PERFORMED.
- o IF A TURBINE RUNBACK HAS OCCURRED, THEN AP-TURB.2, TURBINE LOAD REJECTION, SHOULD BE PERFORMED.
- o OBSERVE D/G LOADING LIMITS OF 2300 KW FOR 1/2 HOUR, 2250 KW FOR 2 HOURS, AND 1950 KW FOR CONTINUOUS SERVICE.
- o DO NOT ATTEMPT TO ENERGIZE A BUS THAT IS POTENTIALLY FAULTED.

NOTE: Conditions should be evaluated for site contingency reporting (Refer to EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION).

1 Establish Manual Rod Control

- | | |
|---|---|
| <ul style="list-style-type: none"> a. Place Rod Control Bank Selector Switch to MANUAL b. Verify control rod motion stops c. Manually move control rods as necessary | <ul style="list-style-type: none"> b. Manually trip the reactor and go to E-0, REACTOR TRIP OR SAFETY INJECTION. |
|---|---|

2 Verify Emergency D/G Associated With Affected Bus - RUNNING AND LOADED

Attempt to start and load emergency D/G(s) manually. (Refer to ER-D/G.1, RESTORING D/Gs)

- o Bus 14 - D/G A
- o Bus 16 - D/G B

EOP:

TITLE:

REV: 2

AP-ELEC.14/16

LOSS OF SAFEGUARDS BUS 14/16

PAGE 4 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

3 Verify At Least One Train of
AC Emergency Busses Energized
to at Least 420 Volts:

o Bus 14 and Bus 18

-OR-

o Bus 16 and Bus 17

Go to ECA-0.0, LOSS OF ALL AC POWER
step 1.

CAUTION

IF CCW FLOW TO A RCP IS INTERRUPTED FOR GREATER THAN 2 MINUTES OR IF EITHER
RCP MOTOR BEARING TEMPERATURE EXCEEDS 200°F, THEN TRIP THE AFFECTED RCP.

4 Verify CCW Pump Status

a. At least one CCW Pump - RUNNING

a. Start one CCW pump (124 KW)

1) IF neither CCW pump can be
started, THEN perform the
following:

a) Trip the reactor.

b) Trip BOTH RCP's.

c) Go to E-0, REACTOR TRIP OR
SAFETY INJECTION.

b. Annunciator A-22, CCW PUMP
DISCHARGE LO PRESS 60 PSIG -
EXTINGUISHED

b. Start second CCW pump (124 KW).

EOP:

TITLE:

REV: 2

AP-ELEC.14/16

LOSS OF SAFEGUARDS BUS 14/16

PAGE 5 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

5 Verify Charging Pump Status-
AT LEAST ONE RUNNING

Secure letdown flowpaths

- o Close loop B cold leg to REGEN Hx, AOV-427.
- o Ensure closed loop A cold leg to EXCESS LETDOWN Hx, AOV-310.
- o Ensure closed EXCESS LETDOWN HCV-123.

6 Monitor S/G Level Control:

Place MFW regulating valves in MANUAL and control feed flow as necessary.

- o S/G level - TRENDING TO 52%
- o MFW regulating valves - CONTROLLING IN AUTO

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 2 PAGE 6 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
7	Verify Bus 14 - ENERGIZED TO AT LEAST 420 VOLTS	<p>Perform the following:</p> <ul style="list-style-type: none"> a. Close loop B cold leg to REGEN Hx, AOV-427. b. <u>IF</u> steam dump is armed, <u>THEN</u> place STEAM DUMP MODE SELECTOR Switch to MANUAL. c. Ensure only one charging pump operating. d. Transfer Inst Bus B to maintenance supply. e. Return steam dump to AUTO, if desired. f. Ensure the following equipment operating as necessary: <ul style="list-style-type: none"> • CCW Pump B • PRZR Backup Heaters • CNMT Recirc Fans B and C • Boric Acid Pump B • RMW Pump B • Reactor Compartment Cooling Fan B • Penetration Cooling Fan B g. <u>IF</u> Bus 14 can <u>NOT</u> be energized, <u>THEN</u>: <ul style="list-style-type: none"> o Provide alternate room cooling for D/G A. o Cross-connect D/G B fuel oil transfer pump to D/G A (Refer to ER-D/G.1).

EOP:

TITLE:

AP-ELEC.14/16

LOSS OF SAFEGUARDS BUS 14/16

REV: 2

PAGE 7 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

8 Verify Bus 16 - ENERGIZED TO
AT LEAST 420 VOLTS

Perform the following:

a. Ensure the following equipment
operating as necessary:

- CCW Pump A
- Charging Pump A
- PRZR Proportional Heaters
- CNMT Recirc Fans A and D
- Boric Acid Pump A
- RMW Pump A
- Reactor Compartment Cooling
Fan A
- Penetration Cooling Fan A

b. IF Bus 16 can NOT be energized,
THEN:

- o Provide alternate room
cooling for D/G B.
- o Cross-connect D/G A fuel oil
transfer pump to D/G B (Refer
to ER-D/G.1).

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: IF VCT level decreased to 5%, charging pump suction will swap to the RWST. This may required a load reduction.

9. Check VCT Makeup System:

a. Verify the following:

- 1) RMW mode selector switch in
AUTO
- 2) RMW control armed - RED LIGHT
LIT

b. Check VCT level:

- o Level GREATER THAN 20%
-OR-
- o Level - STABLE OR INCREASING

a. Adjust controls as necessary.

b. Check letdown divert valve, LCV-112A, aligned to VCT.

Manually increase VCT makeup
flow as follows:

- 1) Ensure BA transfer pumps and
RMW pumps running.
- 2) Adjust RMW flow control
valve, HCV-111, to increase
RMW flow.
- 3) Increase boric acid flow as
necessary to maintain
required concentration.

IF VCT level can NOT be
maintained, THEN refer to
ER-CVCS.1, REACTOR MAKEUP
CONTROL MALFUNCTION, if
necessary.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

10 Check Charging Pump Suction
Aligned To VCT:

a. VCT level - GREATER THAN 20%

a. IF VCT level can NOT be
maintained greater than 5%, THEN
perform the following:

1) Ensure charging pump suction
aligned to RWST

o LCV-112B open

o LCV-112C closed

2) Continue with Step 11. WHEN
VCT level greater than 20%,
THEN do Step 10b.

b. Verify charging pumps aligned to
VCT

o LCV-112C open

o LCV-112B closed

b. Manually align valves as
necessary.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: When restarting equipment for recovery, it is preferable to start equipment on busses being supplied from offsite power, if possible.

11 Check CVCS Operation:

a. Charging pumps - AT LEAST ONE RUNNING

a. IF charging pump(s) available, THEN perform the following:

- 1) Start charging pumps as necessary.
- 2) Establish greater than 20 gpm charging line flow.

IF NO charging pumps available, THEN go to step 13

b. Charging line flow - GREATER THAN 20 GPM

b. Establish charging line flow to REGEN Hx - GREATER THAN 20 GPM

c. Check letdown indications:

c. Perform the following:

- o Check PRZR level - GREATER THAN 13%
- o Letdown flow - APPROXIMATELY 40 GPM
- o Letdown flow - STABLE

- 1) Close loop B cold leg to REGEN Hx, AOV-427.
- 2) Close letdown orifice valves (AOV-200A, AOV-200B, and AOV-202)
- 3) IF PRZR level greater than 13%, THEN go to Step 12. IF NOT, THEN continue with Step 14. WHEN PRZR level greater than 13%, THEN do Steps 12 and 13.

d. Adjust charging pump speed and HCV-142 as necessary to restore PRZR level and labyrinth seal D/P

e. Go to Step 13

EOP:	TITLE:	REV: 2
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	PAGE 11 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: Steps 12 and 13 may be performed concurrently.

12 Establish Normal Letdown:

- a. Establish charging line flow to REGEN Hx - GREATER THAN 20 GPM
- b. Place letdown controllers in MANUAL at 40% open
 - TCV-130
 - PCV-135
- c. Open AOV-427
- d. Open letdown orifice valves as necessary
- e. Place TCV-130 in AUTO at 105°F
- f. Place PCV-135 in AUTO at 250 psig
- g. Adjust charging pump speed and HCV-142 as necessary

Perform the following steps in sequence to establish excess letdown, if desired:

- o Place excess letdown divert valve, AOV-312, to NORMAL
- o Ensure CCW from excess letdown open, AOV-745
- o Ensure RCP seal return isolation valve open, MOV-313
- o Open excess letdown isolation valve, AOV-310
- o Slowly open HCV-123 to maintain excess letdown temperature less than 195°F and pressure less than 100 psig

13 Verify PRZR Heaters Restored:

- o PRZR proportional heater breaker - CLOSED
- o PRZR backup heater breaker - RESET/IN AUTO

IF adequate D/G capacity available for PRZR heaters (400 kw each bank), THEN perform the following:

- a. Reset and close PRZR proportional heater breaker if necessary.
- b. Reset PRZR backup heater breaker and return to AUTO if necessary.

IF adequate D/G capacity NOT available, THEN refer to ER-PRZR.1.

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 2 PAGE 12 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
14	Verify Normal Rod Control Restored:	
a.	Annunciator C-5, PPCS ROD SEQUENCE OR ROD DEVIATION - EXTINGUISHED	a. <u>IF</u> alarm is due to a loss of power to MRPI, <u>THEN</u> maintain rods in manual <u>AND</u> minimize rod motion. <u>IF</u> alarm is due to actual rod misalignment, <u>THEN</u> refer to AP-RCC.2, RCC/RPI MALFUNCTION, while continuing with this procedure.
b.	Annunciator E-28, POWER RANGE ROD DROP ROD STOP - EXTINGUISHED	b. Perform the following: 1) Place rod control bank selector switch in MANUAL. 2) Reset NIS rod drop rod stop signals (at NIS racks) as necessary.
c.	Annunciator F-15, RCS TAVG DEV 4°F - EXTINGUISHED	c. Go to step 15
d.	Place rod control bank selector switch in AUTO if desired	

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

15 Establish Stable Plant Conditions:

a. Check Tav_g - TRENDING TO TREFa. Insert control rods or, if necessary, decrease turbine load to match Tav_g to Tref.

b. Check PRZR pressure - TRENDING TO 2235 PSIG

b. Verify proper operation of PRZR heaters and spray or take manual control of PRZR pressure controller 431K.

c. Check PRZR level - TRENDING TO PROGRAM

c. Verify proper operation of charging pump speed controllers or take manual control of speed controllers to control PRZR level.

16 Restore Normal Electric System Alignment:

a. Verify circuit 767 and/or 751 - AVAILABLE

a. Continue with Step 17. WHEN offsite power available, THEN do Steps 16b and c.

b. Verify all emergency AC bus normal feed breakers - CLOSED

b. Perform the following:

- Bus 14
- Bus 16
- Bus 17
- Bus 18

1) Restore emergency AC busses to normal power supply (Refer to ER-ELEC.1, RESTORATION OF OFFSITE POWER)

2) IF normal power is restored to all AC emergency buses, THEN return to step 7. IF NOT, THEN go to step 17.

c. Stop any unloaded emergency D/G and place in standby (Refer to T-27.4)

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 2 PAGE 14 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

17 Establish Normal Plant Conditions:

- | | |
|---|--|
| <p>a. Verify 2 charging pumps - RUNNING</p>

<p>b. Verify at least 2 CNMT recirc fans - RUNNING</p>
<p>c. Check CCW pumps - ONLY ONE RUNNING</p>
<p>d. Check radiation monitoring systems:</p> <ul style="list-style-type: none"> o CNMT vent sample pump - RUNNING o Plant vent sample pump - RUNNING o All area and process monitors operating as required | <p>a. Perform the following:</p> <ul style="list-style-type: none"> 1) Manually start charging pumps as necessary. 2) Place selected charging pump speed controller in AUTO if desired. <p>b. Start CNMT recirc fans as necessary (240 kw each).</p> <p>c. Locally verify two CCW pumps running. <u>THEN</u> manually stop one pump.</p> <p>d. Restore sample pumps and radiation monitors as necessary. (Refer to CHA-RETS-ODCM).</p> |
|---|--|

18 Check Status Of DC System Loads:

- | | |
|---|--|
| <p>a. Verify TDAFW pump DC oil pump - OFF IN AUTO</p> | <p>a. Perform the following:</p> <ul style="list-style-type: none"> 1) Direct AO to locally check TDAFW AC oil pump running. <u>IF</u> not running. <u>THEN</u> start pump from MCB. 2) Stop TDAFW pump DC oil pump. |
|---|--|

EOP:	TITLE:	REV: 2
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	PAGE 15 of 17

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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NOTE: Inst Bus C provides power to all MCB manual controllers.

19 Check Status of Battery
Chargers:

a. Battery Chargers 1A OR 1A1 -
ENERGIZED.

a. IF BOTH battery chargers are
deenergized, THEN direct the
Electricians to crosstie TSC
battery charger to main battery
A (Refer to ATTACHMENT TRANSFER
BATTERY TO TSC).

b. Battery Chargers 1B OR 1B1 -
ENERGIZED

b. IF BOTH battery chargers are
deenergized, THEN direct the
Electricians to crosstie TSC
battery charger to main battery
B (Refer to ATTACHMENT TRANSFER
BATTERY TO TSC).

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
20	Restore Equipment Alignment:	
a.	Verify annunciator L-1, AUX BLDG VENT SYSTEM CONTROL PANEL - EXTINGUISHED	a. Dispatch AO to restore AUX BLDG ventilation (Refer to T-35A, AUX AND INTERMEDIATE BUILDING VENTILATION STARTUP AND SHUTDOWN)
b.	Restore affected bus equipment as desired	
	o SFP Cooling	
	o Penetration cooling fans	
	o Reactor compartment cooling fans	
	o Hydrogen panel	
	o PA system inverter (Battery Room A)	
	o Auxiliary Bldg lighting (normal supply MCC D, manual throwover to MCC C) (located at MCC C)	
	o Fire system (Refer to SC-3.16.2.3)	
c.	Check control board annunciator panels - ALARM STATUS VALID FOR PLANT CONDITIONS	c. Perform alarm response procedures for unexpected alarms.
d.	Verify control board valve alignment - NORMAL (Refer to O-6.13, DAILY SURVEILLANCE LOG)	d. Manually align valves as necessary.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.

- 21 Verify emergency AC bus normal feed breakers closed
- o Bus 14
 - o Bus 16

Return to Step 7

- 22 Verify Inst Bus B on normal supply

Place Inst Bus B on normal supply (Refer to ER-INST.3, INSTRUMENT BUS POWER RESTORATION).

- 23 Reset UV relay targets on undervoltage cabinets
- o Bus 14
 - o Bus 16

- 24 Notify Higher Supervision

- 25 Return To Procedure Or Guidance In Effect

-END-

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 2 PAGE 1 of 1
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AP-ELEC.14/16 APPENDIX LIST

TITLE

- 1) ATTACHMENT TRANSFER BATTERY TO TSC (ATT-24.0)

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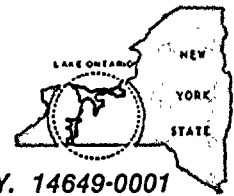
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March 12, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555
Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are revisions to Ginna Station Emergency Plan Implementing Procedures (EPIPs).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,


Peter S. Polfleit
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

PSP/jtw

A045

ML010750076

PROCEDURE

REVISION NUMBER

EPIP 1-6	12
EPIP 3-1	15
EPIP 3-2	9
EPIP 3-4	8
EPIP 5-2	24
EPIP 5-5	12
EPIP 5-7	31

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JOSEPH A. WIDAY
VICE PRESIDENT & PLANT MANAGER
GINNA STATION

February 28, 2001

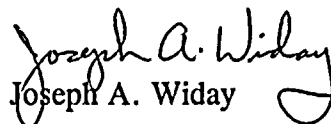
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Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,


Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

E Index
ECA Index
FIG Index
E-1, Rev 21
E-3, Rev 27
ECA-3.3, Rev 24
FIG-4.0, Rev 2

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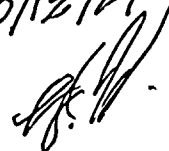
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50-244

2/8/01

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JOSEPH A. WIDAY
VICE PRESIDENT & PLANT MANAGER
GINNA STATION

February 8, 2001

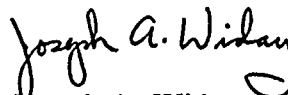
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Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,


Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index
FIG Index
AP-ELEC.14/16, rev 2
AP-RHR.1, rev 15
FIG-3.1, rev 1

010530028

A002

REPORT NO. 01
REPORT: NPSF0200
DOC TYPE: PRAP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
ABNORMAL PROCEDURE

10/16/00 PAGE: 1

PARAMETERS: DOC TYPES - PRATT PRAP PRES PRFIG STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	013	10/30/98	05/01/98	05/01/03	EF
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	014	05/18/00	08/17/99	08/17/04	EF
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	012	05/18/00	08/17/99	08/17/04	EF
AP-CR.1	CONTROL ROOM INACCESSIBILITY	016	01/11/00	01/11/00	01/11/05	EF
AP-CVCS.1	CVCS LEAK	012	05/01/98	05/01/98	05/01/03	EF
AP-CVCS.3	LOSS OF ALL CHARGING FLOW	002	02/11/00	02/26/99	02/26/04	EF
AP-CW.1	LOSS OF A CIRC WATER PUMP	010	07/16/98	05/01/98	05/01/03	EF
AP-ELEC.1	LOSS OF 12A AND/OR 12B BUSES	020	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.2	SAFEGUARD BUSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	009	03/22/99	03/22/99	03/22/04	EF
AP-ELEC.3	LOSS OF 12A AND/OR 12B TRANSFORMER (BELOW 350 F)	008	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	001	09/08/00	06/09/97	06/09/02	EF
AP-ELEC.17/18	LOSS OF SAFEGUARDS BUS 17/18	002	10/18/99	06/09/97	06/09/02	EF
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	012	02/11/00	02/27/98	02/27/03	EF
AP-IA.1	LOSS OF INSTRUMENT AIR	017	12/02/99	05/01/98	05/01/03	EF
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	011	12/02/99	12/02/99	12/02/04	EF
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	006	02/24/96	05/14/98	05/14/03	EF
AP-RCC.2	RCC/RPI MALFUNCTION	008	11/16/98	02/06/97	02/06/02	EF
AP-RCC.3	DROPPED ROD RECOVERY	004	11/16/98	02/27/98	02/27/03	EF
AP-RCP.1	RCP SEAL MALFUNCTION	013	06/09/00	05/01/98	05/01/03	EF
AP-RCS.1	REACTOR COOLANT LEAK	015	09/08/00	05/01/98	05/01/03	EF
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	010	12/14/98	05/01/98	05/01/03	EF
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	007	08/05/97	08/05/97	08/05/02	EF
AP-RCS.4	SHUTDOWN LOCA	011	12/02/99	05/01/98	05/01/03	EF
AP-RHR.1	LOSS OF RHR	014	10/13/00	05/01/98	05/01/03	EF

REPORT NO. 01
REPORT: NPSP0200
DOC TYPE: PRAP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
ABNORMAL PROCEDURE

10/16/00 PAGE: 2

PARAMETERS: DOC TYPES - PRATT PRAP PRES PRFIG STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	009	10/13/00	03/31/00	03/31/05	EF
AP-SG.1	STEAM GENERATOR TUBE LEAK	000	09/08/00	09/08/00	09/08/05	EF
AP-SW.1	SERVICE WATER LEAK	015	10/18/99	06/03/98	06/03/03	EF
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	010	02/12/99	10/10/97	10/10/02	EF
AP-TURB.2	TURBINE LOAD REJECTION	017	02/11/00	05/13/98	05/13/03	EF
AP-TURB.3	TURBINE VIBRATION	010	02/11/00	02/10/98	02/10/03	EF
AP-TURB.4	LOSS OF CONDENSER VACUUM	014	05/01/98	05/01/98	05/01/03	EF
AP-TURB.5	RAPID LOAD REDUCTION	005	06/09/00	06/09/00	06/09/05	EF
TOTAL FOR PRAP	32					

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 1 of 17
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ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

Reside
RESPONSIBLE MANAGER

9-8-2000
EFFECTIVE DATE

CATEGORY 1.0

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EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 2 of 17
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- A. PURPOSE - This procedure provides actions to respond to a loss of AC Emergency Bus 14 or Bus 16.
- B. ENTRY CONDITIONS/SYMPTOMS
 - 2. SYMPTOMS - The symptoms of a LOSS OF SAFEGUARDS BUS 14/16 are;
 - a. Annunciator J-7, 480V MAIN OR TIE BREAKER TRIP, lit, or
 - b. Annunciator J-29, 480V TRANSFORMER BREAKER TRIP, lit.

EOP:

AP-ELEC.14/16

TITLE:

LOSS OF SAFEGUARDS BUS 14/16

REV: 1

PAGE 3 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

- o IF A LOSS OF BUS 12A OR 12B HAS OCCURRED, THEN NO OUTSIDE, SHOULD BE PERFORMED.
 - o IF A TURBINE RUNBACK HAS OCCURRED, THEN AP-TURB.2, TURBINE LOAD REJECTION, SHOULD BE PERFORMED.
 - o OBSERVE D/G LOADING LIMITS OF 2300 KW FOR 1/2 HOUR, 2250 KW FOR 2 HOURS, AND 1950 KW FOR CONTINUOUS SERVICE.
 - o DO NOT ATTEMPT TO ENERGIZE A BUS THAT IS POTENTIALLY FAULTED.
- *****

NOTE: Conditions should be evaluated for site contingency reporting (Refer to EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION).

1 Establish Manual Rod Control

a. Place Rod Control Bank Selector Switch to MANUAL

b. Verify control rod motion stops

c. Manually move control rods as necessary

b. Manually trip the reactor and go to E-0, REACTOR TRIP OR SAFETY INJECTION.

2 Verify Emergency D/G Associated With Affected Bus - RUNNING AND LOADED

Attempt to start and load emergency D/G(s) manually. (Refer to ER-D/G.1, RESTORING D/Gs)

o Bus 14 - D/G A

o Bus 16 - D/G B

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

3 Verify At Least One Train of
AC Emergency Busses Energized
to at Least 420 Volts:

o Bus 14 and Bus 18

-OR-

o Bus 16 and Bus 17

Go to ECA-0.0, LOSS OF ALL AC POWER
step 1.

CAUTION

IF CCW FLOW TO A RCP IS INTERRUPTED FOR GREATER THAN 2 MINUTES OR IF EITHER
RCP MOTOR BEARING TEMPERATURE EXCEEDS 200°F, THEN TRIP THE AFFECTED RCP.

4 Verify CCW Pump Status

a. At least one CCW Pump - RUNNING

a. Start one CCW pump (124 KW)

1) IF neither CCW pump can be
started, THEN perform the
following:

a) Trip the reactor.

b) Trip BOTH RCP's.

c) Go to E-0, REACTOR TRIP OR
SAFETY INJECTION.

b. Annunciator A-22, CCW PUMP
DISCHARGE LO PRESS 60 PSIG -
EXTINGUISHED

b. Start second CCW pump (124 KW).

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 5. of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
5	Verify Charging Pump Status- AT LEAST ONE RUNNING	Secure letdown flowpaths <ul style="list-style-type: none"> o Close loop B cold leg to REGEN Hx, AOV-427. o Ensure closed loop A cold leg to EXCESS LETDOWN Hx, AOV-310. o Ensure closed EXCESS LETDOWN HCV-123.
6	Monitor S/G Level Control: <ul style="list-style-type: none"> o S/G level - TRENDING TO 52% o MFW regulating valves - CONTROLLING IN AUTO 	Place MFW regulating valves in MANUAL and control feed flow as necessary.

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 6 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
7	Verify Bus 14 - ENERGIZED TO AT LEAST 420 VOLTS	<p>Perform the following:</p> <ol style="list-style-type: none"> Close loop B cold leg to REGEN Hx, AOV-427. <u>IF</u> steam dump is armed, <u>THEN</u> place STEAM DUMP MODE SELECTOR Switch to MANUAL. Ensure only one charging pump operating. Transfer Inst Bus B to maintenance supply. Return steam dump to AUTO, if desired. Ensure the following equipment operating as necessary: <ul style="list-style-type: none"> • CCW Pump B • PRZR Backup Heaters • CNMT Recirc Fans B and C • Boric Acid Pump B • RMW Pump B • Reactor Compartment Cooling Fan B • Penetration Cooling Fan B <u>IF</u> Bus 14 can <u>NOT</u> be energized, <u>THEN</u>: <ul style="list-style-type: none"> o Provide alternate room cooling for D/G A. o Cross-connect D/G B fuel oil transfer pump to D/G A (Refer to T-27.4 Attachment A).

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 7 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
8	Verify Bus 16 - ENERGIZED TO AT LEAST 420 VOLTS	Perform the following: a. Ensure the following equipment operating as necessary: <ul style="list-style-type: none">• CCW Pump A• Charging Pump A• PRZR Proportional Heaters• CNMT Recirc Fans A and D• Boric Acid Pump A• RMW Pump A• Reactor Compartment Cooling Fan A• Penetration Cooling Fan A b. <u>IF</u> Bus 16 can <u>NOT</u> be energized, <u>THEN</u> : <ul style="list-style-type: none">o Provide alternate room cooling for D/G B.o Cross-connect D/G A fuel oil transfer pump to D/G B (Refer to T-27.4 Attachment B).

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: IF VCT level decreased to 5%, charging pump suction will swap to the RWST. This may required a load reduction.

9 Check VCT Makeup System:

a. Verify the following:

- 1) RMW mode selector switch in AUTO
- 2) RMW control armed - RED LIGHT LIT

b. Check VCT level:

- o Level GREATER THAN 20%

-OR-

- o Level - STABLE OR INCREASING

a. Adjust controls as necessary.

b. Check letdown divert valve, LCV-112A, aligned to VCT.

Manually increase VCT makeup flow as follows:

- 1) Ensure BA transfer pumps and RMW pumps running.
- 2) Adjust RMW flow control valve, HCV-111, to increase RMW flow.
- 3) Increase boric acid flow as necessary to maintain required concentration.

IF VCT level can NOT be maintained, THEN refer to ER-CVCS.1, REACTOR MAKEUP CONTROL MALFUNCTION, if necessary.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

10 Check Charging Pump Suction
Aligned To VCT:

a. VCT level - GREATER THAN 20%

a. IF VCT level can NOT be
maintained greater than 5%, THEN
perform the following:

1) Ensure charging pump suction
aligned to RWST

o LCV-112B open

o LCV-112C closed

2) Continue with Step 11. WHEN
VCT level greater than 20%,
THEN do Step 10b.

b. Verify charging pumps aligned to
VCT

b. Manually align valves as
necessary.

o LCV-112C open

o LCV-112B closed

EOP:	TITLE:	REV: 1
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	PAGE 10 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: When restarting equipment for recovery, it is preferable to start equipment on busses being supplied from offsite power, if possible.

11 Check CVCS Operation:

a. Charging pumps - AT LEAST ONE RUNNING

a. IF charging pump(s) available, THEN perform the following:

- 1) Start charging pumps as necessary.
- 2) Establish greater than 20 gpm charging line flow.

IF NO charging pumps available, THEN go to step 13

b. Charging line flow - GREATER THAN 20 GPM

b. Establish charging line flow to REGEN Hx - GREATER THAN 20 GPM

c. Check letdown indications:

c. Perform the following:

- o Check PRZR level - GREATER THAN 13%
- o Letdown flow - APPROXIMATELY 40 GPM
- o Letdown flow - STABLE

- 1) Close loop B cold leg to REGEN Hx, AOV-427.
- 2) Close letdown orifice valves (AOV-200A, AOV-200B, and AOV-202)
- 3) IF PRZR level greater than 13%, THEN go to Step 12. IF NOT, THEN continue with Step 14. WHEN PRZR level greater than 13%, THEN do Steps 12 and 13.

d. Adjust charging pump speed and HCV-142 as necessary to restore PRZR level and labyrinth seal D/P .

e. Go to Step 13

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: Steps 12 and 13 may be performed concurrently.

12 Establish Normal Letdown:

- a. Establish charging line flow to REGEN Hx - GREATER THAN 20 GPM
- b. Place letdown controllers in MANUAL at 40% open
 - TCV-130
 - PCV-135
- c. Open AOV-427
- d. Open letdown orifice valves as necessary
- e. Place TCV-130 in AUTO at 105°F
- f. Place PCV-135 in AUTO at 250 psig
- g. Adjust charging pump speed and HCV-142 as necessary

Perform the following steps in sequence to establish excess letdown, if desired:

- o Place excess letdown divert valve, AOV-312, to NORMAL
- o Ensure CCW from excess letdown open, AOV-745
- o Ensure RCP seal return isolation valve open, MOV-313
- o Open excess letdown isolation valve, AOV-310
- o Slowly open HCV-123 to maintain excess letdown temperature less than 195°F and pressure less than 100 psig

13 Verify PRZR Heaters Restored:

- o PRZR proportional heater breaker - CLOSED
- o PRZR backup heater breaker - RESET/IN AUTO

IF adequate D/G capacity available for PRZR heaters (400 kw each bank), THEN perform the following:

- a. Reset and close PRZR proportional heater breaker if necessary.
- b. Reset PRZR backup heater breaker and return to AUTO if necessary.

IF adequate D/G capacity NOT available, THEN refer to ER-PRZR.1.

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 12 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
14	Verify Normal Rod Control Restored:	
a.	Annunciator C-5, PPCS ROD SEQUENCE OR ROD DEVIATION - EXTINGUISHED	a. <u>IF</u> alarm is due to a loss of power to MRPI, <u>THEN</u> maintain rods in manual <u>AND</u> minimize rod motion. <u>IF</u> alarm is due to actual rod misalignment, <u>THEN</u> refer to AP-RCC.2, RCC/RPI MALFUNCTION, while continuing with this procedure.
b.	Annunciator E-28, POWER RANGE ROD DROP ROD STOP - EXTINGUISHED	b. Perform the following: 1) Place rod control bank selector switch in MANUAL. 2) Reset NIS rod drop rod stop signals (at NIS racks) as necessary.
c.	Annunciator F-15, RCS TAVG DEV 4°F - EXTINGUISHED	c. Go to step 15
d.	Place rod control bank selector switch in AUTO if desired	

EOP:	TITLE:	REV: 1
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	PAGE 13 of 17

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
15 Establish Stable Plant Conditions:		
a. Check Tav _g - TRENDING TO TREF		a. Insert control rods or, if necessary, decrease turbine load to match Tav _g to Tref.
b. Check PRZR pressure - TRENDING TO 2235 PSIG		b. Verify proper operation of PRZR heaters and spray or take manual control of PRZR pressure controller 431K.
c. Check PRZR level - TRENDING TO PROGRAM		c. Verify proper operation of charging pump speed controllers or take manual control of speed controllers to control PRZR level.
16 Restore Normal Electric System Alignment:		
a. Verify circuit 767 and/or 751 - AVAILABLE		a. Continue with Step 17. <u>WHEN</u> offsite power available, <u>THEN</u> do Steps 16b and c.
b. Verify all emergency AC bus normal feed breakers - CLOSED		b. Perform the following:
<ul style="list-style-type: none"> • Bus 14 • Bus 16 • Bus 17 • Bus 18 		1) Restore emergency AC busses to normal power supply (Refer to ER-ELEC.1, RESTORATION OF OFFSITE POWER) 2) <u>IF</u> normal power is restored to all AC emergency buses, <u>THEN</u> return to step 7. <u>IF NOT</u> , <u>THEN</u> go to step 17.
c. Stop any unloaded emergency D/G and place in standby (Refer to T-27.4)		

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 14 of 17
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
17	Establish Normal Plant Conditions:	
a.	Verify 2 charging pumps - RUNNING	a. Perform the following: 1) Manually start charging pumps as necessary. 2) Place selected charging pump speed controller in AUTO if desired.
b.	Verify at least 2 CNMT recirc fans - RUNNING	b. Start CNMT recirc fans as necessary (240 kw each).
c.	Check CCW pumps - ONLY ONE RUNNING	c. Locally verify two CCW pumps running, <u>THEN</u> manually stop one pump.
d.	Check radiation monitoring systems: o CNMT vent sample pump - RUNNING o Plant vent sample pump - RUNNING o All area and process monitors operating as required	d. Restore sample pumps and radiation monitors as necessary. (Refer to CHA-RETS-ODCM).
18	Check Status Of DC System Loads:	
a.	Verify TDAFW pump DC oil pump - OFF IN AUTO	a. Perform the following: 1) Direct AO to locally check TDAFW AC oil pump running. <u>IF</u> not running, <u>THEN</u> start pump from MCB. 2) Stop TDAFW pump DC oil pump.

EOP:

TITLE:

REV: 1

AP-ELEC.14/16

LOSS OF SAFEGUARDS BUS 14/16

PAGE 15 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: Inst Bus C provides power to all MCB manual controllers.

19 Check Status of Battery
Chargers:

a. Battery Chargers 1A OR 1A1 -
ENERGIZED.

a. IF BOTH battery chargers are
deenergized, THEN direct the
Electricians to crosstie TSC
battery charger to main battery
A (Refer to ATTACHMENT TRANSFER
BATTERY TO TSC).

b. Battery Chargers 1B OR 1B1 -
ENERGIZED

b. IF BOTH battery chargers are
deenergized, THEN direct the
Electricians to crosstie TSC
battery charger to main battery
B (Refer to ATTACHMENT TRANSFER
BATTERY TO TSC).

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

20 Restore Equipment Alignment:

- | | |
|---|---|
| a. Verify annunciator L-1, AUX BLDG VENT SYSTEM CONTROL PANEL - EXTINGUISHED | a. Dispatch AO to restore AUX BLDG ventilation (Refer to T-35A, AUX AND INTERMEDIATE BUILDING VENTILATION STARTUP AND SHUTDOWN) |
| b. Restore affected bus equipment as desired | |
| o SFP Cooling | |
| o Penetration cooling fans | |
| o Reactor compartment cooling fans | |
| o Hydrogen panel | |
| o PA system inverter (Battery Room A) | |
| o Auxiliary Bldg lighting (normal supply MCC D, manual throwover to MCC C) (located at MCC C) | |
| o Fire system (Refer to SC-3.16.2.3) | |
| c. Check control board annunciator panels - ALARM STATUS VALID FOR PLANT CONDITIONS | c. Perform alarm response procedures for unexpected alarms. |
| d. Verify control board valve alignment - NORMAL (Refer to O-6.13, DAILY SURVEILLANCE LOG) | d. Manually align valves as necessary. |

EOP:

AP-ELEC.14/16

TITLE:

LOSS OF SAFEGUARDS BUS 14/16

REV: 1

PAGE 17 of 17

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.

21 Verify emergency AC bus
normal feed breakers closed

Return to Step 7

o Bus 14

o Bus 16

22 Verify Inst Bus B on normal
supply

Place Inst Bus B on normal supply
(Refer to ER-INST.3, INSTRUMENT BUS
POWER RESTORATION).

23 Reset UV relay targets on
undervoltage cabinets

o Bus 14

o Bus 16

24 Notify Higher Supervision

25 Return To Procedure Or
Guidance In Effect

-END-

EOP: AP-ELEC.14/16	TITLE: LOSS OF SAFEGUARDS BUS 14/16	REV: 1 PAGE 1 of 1
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AP-ELEC.14/16 APPENDIX LIST

TITLE

- 1) ATTACHMENT TRANSFER BATTERY TO TSC (ATT-24.0)

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 1 of 13
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ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

Richard A. [Signature]
RESPONSIBLE MANAGER

10-13-2000
EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: _____

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 2 of 13
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A. PURPOSE - This procedure provides guidance in the event of a loss of RHR cooling at or above normal loop levels. (i.e. RCS loop levels of 64 inches or greater)

B. ENTRY CONDITIONS/SYMPTOMS

1. ENTRY CONDITIONS - This procedure is entered from;

- a. FR-C.3, RESPONSE TO SATURATED CORE COOLING, or
- b. AP-ELEC.3, LOSS OF 12A AND/OR 12B TRANSFORMER (BELOW 350°F), when RHR flow can NOT be restored, or
- c. AP-CCW.3, LOSS OF CCW - PLANT SHUTDOWN when CCW is inadequate for RHR cooling

2. SYMPTOMS - The following are symptoms of LOSS OF RHR;

- a. No RHR pumps running, or
- b. Annunciator A-20, RESIDUAL HEAT REMOVAL LOOP LO FLOW 2900 GPM (Set at 400 GPM per 0-2.2 in RHR Cooling mode), lit, or
- c. Unexpected increase in temperature while on RHR cooling, or
- d. Erratic or no flow on FI-626, RHR Loop Flow, or
- e. Annunciator J-9, SAFEGUARD BREAKER TRIP, lit.

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 3 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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CAUTION

DO NOT START ANOTHER RHR PUMP UNTIL THE CAUSE OF THE ABNORMAL RHR INDICATIONS HAS BEEN DETERMINED. IF A RUNNING PUMP HAS TRIPPED FOR REASONS OTHER THAN LOSS OF SUCTION FLOW, THEN REDUNDANT PUMP MAY BE STARTED.

NOTE: Conditions should be evaluated for site contingency reporting (Refer to EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION).

1 Check PRZR Wide Range Level -
GREATER THAN 0 INCHES

IF RCS loop level indicator in service and loop level less than 64 inches, THEN go to AP-RHR.2, LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS.

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 4 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
2	Check If RHR Pump(s) Should Be Stopped:	
a.	RHR pump - ANY RUNNING	a. Go to Step 3.
b.	Check RHR pump flow - LESS THAN 1500 GPM PER PUMP	b. Decrease RHR flow as necessary. <u>IF</u> RHR flow can <u>NOT</u> be controlled, <u>THEN</u> perform the following: <ol style="list-style-type: none"> 1) Stop running RHR pump. 2) Dispatch an AO with a locked valve key to locally throttle RHR Hx outlet valves to approximately half open. <ul style="list-style-type: none"> • A RHR Hx, HCV-625 handwheel • B RHR Hx, HCV-624 handwheel 3) Start an RHR pump. 4) Direct AO to locally adjust RHR flow to less than 1500 gpm.
c.	RHR pumps cavitating: <ul style="list-style-type: none"> o RHR pump flow - OSCILLATING -OR- o RHR pump NPSH - APPROXIMATELY ZERO (PPCS group GD NPSH) 	c. Go to Step 17.
d.	Stop RHR pumps	

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 5 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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CAUTION

- o DO NOT INITIATE ANY ACTIONS WHICH MAY ADD POSITIVE REACTIVITY TO THE CORE.
- o NOTIFY S/G OFFICE THAT CNMT BREATHING AIR MAY BE LOST.
- o IF REFUELING IN PROGRESS, THEN STOP REFUELING OPERATIONS (NOTIFY REFUELING SRO).

NOTE: Personnel remaining in CNMT to assist in event mitigation should consult Health Physics for changes in radiological concerns.

3 Initiate Actions To Protect
Personnel In CNMT:

- | | |
|--|---|
| a. Evacuate non-essential personnel from CNMT | |
| b. Verify all available CNMT RECIRC fan(s) - RUNNING | b. Manually start available CNMT RECIRC fans. |
| c. Initiate monitoring of CNMT area and process radiation monitors | c. Refer to appropriate alarm response procedures for required actions. |
| d. Verify CNMT penetrations with direct access to outside atmosphere - CLOSED (Refer to Attachment CNMT CLOSURE) | d. Within 4 hours, close all CNMT penetrations to outside atmosphere. |

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 6 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
4	Check RHR Cooling Valve Alignment - NORMAL (Refer to Attachment NORMAL RHR COOLING)	Manually or locally align valves as necessary.
<p style="text-align: center;">*****</p> <p style="text-align: center;"><u>CAUTION</u></p> <p>THE RHR HX OUTLET VALVES (HCV-624 AND HCV-625) WILL FAIL OPEN ON LOSS OF INSTRUMENT AIR PRESSURE.</p> <p style="text-align: center;">*****</p>		
5	Check IA System:	
	a. Verify 2 IA compressors - RUNNING	a. Manually start IA compressors as necessary (75 kw each). <u>IF</u> IA compressors can <u>NOT</u> be started manually, <u>THEN</u> dispatch AO to locally reset and start compressors (75 kw each).
	b. Check IA supply	b. <u>IF</u> IA pressure can <u>NOT</u> be restored, <u>THEN</u> perform the following:
	o Pressure - GREATER THAN 60 PSIG	1) Dispatch AO with a locked valve key to locally throttle RHR Hx outlet valves to approximately half open.
	o Pressure - STABLE OR INCREASING	<ul style="list-style-type: none"> • A RHR Hx. HCV-625 handwheel • B RHR Hx. HCV-624 handwheel
		2) <u>WHEN</u> conditions permit, <u>THEN</u> refer to AP-IA.1, LOSS OF INSTRUMENT AIR, to restore IA.

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 7 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
* 6	Monitor RCS Temperature - GREATER THAN 200°F	Go to Step 8.
<p>*****</p> <p style="text-align: center;"><u>CAUTION</u></p> <p>o CHANGES IN RCS PRESSURE COULD RESULT IN INACCURACIES IN RCS LOOP LEVEL INDICATION</p> <p>o UNSTABLE OR FLUCTUATING LEVEL INSTRUMENTS SHOULD NOT BE RELIED ON FOR INDICATION OF RCS INVENTORY.</p> <p>*****</p>		
7	Verify RCS Intact:	Perform the following:
	<ul style="list-style-type: none"> o PRZR level - GREATER THAN 5% AND STABLE o RCS pressure - STABLE o RCS subcooling based on core exit T/Cs - GREATER THAN 0°F USING FIGURE MIN SUBCOOLING o RCS vent paths - CLOSED 	<ul style="list-style-type: none"> a. Verify charging line flow control valve, HCV-142, open as necessary. b. Ensure charging line valve to loop B cold leg, AOV-294, open. c. Start charging pumps as necessary. d. Control charging pump speed and letdown flow as necessary to stabilize RCS conditions. <ul style="list-style-type: none"> • PRZR pressure • PRZR level • Loop level <p><u>IF</u> charging flow greater than 75 gpm with letdown isolated <u>OR</u> unable to verify RCS inventory, <u>THEN</u> go to AP-RCS.4, SHUTDOWN LOCA.</p>

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 8 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
8	<p>Establish Conditions To Start RHR Pump:</p> <p>a. RHR pump - AVAILABLE</p> <p>b. Verify CCW cooling to RHR system in service</p> <ul style="list-style-type: none"> o CCW pumps - AT LEAST ONE RUNNING o CCW to RHR Hxs, MOV-738A AND MOV-738B - OPEN AS NECESSARY <p>c. Close RHR pump flow control valves (controllers at 100% demand)</p> <ul style="list-style-type: none"> • HCV-624 • HCV-625 <p>d. Place RHR Hx bypass valve, HCV-626, to MANUAL and close valve</p>	<p>a. Perform the following:</p> <ol style="list-style-type: none"> 1) Start trending core exit TCs. 2) <u>IF</u> RCS closed, <u>THEN</u> go to Step 10. <u>IF</u> RCS open to atmosphere, <u>THEN</u> go to Step 16. <p>b. Perform the following:</p> <ol style="list-style-type: none"> 1) Ensure at least one CCW pump running. 2) Open MOV-738A and MOV-738B as necessary. <p><u>IF</u> CCW can <u>NOT</u> be restored, <u>THEN</u> continue with Step 9 while attempting to restore CCW (Refer to AP-CCW.3, LOSS OF CCW - PLANT SHUTDOWN).</p>

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 9 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
<p>*****</p> <p style="text-align: center;"><u>CAUTION</u></p> <p>STARTING AN RHR PUMP MAY RESULT IN AN RCS LEVEL OR PRESSURE DECREASE DUE TO SHRINK OR VOID COLLAPSE.</p> <p>*****</p>		
9	Restore RHR Flow:	
	a. Start one RHR pump - RHR PUMP RUNNING	a. Go to Step 9e.
	b. Check RHR flow - LESS THAN 1500 GPM PER PUMP	b. Manually adjust RHR flow as necessary.
	c. Adjust RHR Hx bypass flow control valve, HCV-626, to desired flowrate	
	d. Place RHR Hx bypass flow control valve, HCV-626, controller in AUTO	
	e. RHR flow - RESTORED	e. Perform the following:
		1) Start trending core exit T/Cs.
		2) <u>IF</u> RCS closed, <u>THEN</u> go to Step 10. <u>IF</u> RCS vented to atmosphere, <u>THEN</u> go to Step 16.
	f. Open RHR Hx outlet valves as necessary to control RCS temperature	
	• HCV-624	
	• HCV-625	

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 10 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
10 Monitor RCS Temperature:		
a. RCS temperature - STABLE OR DECREASING		a. <u>IF</u> RCS closed, <u>THEN</u> go to Step 11. <u>IF</u> RCS open to atmosphere, <u>THEN</u> go to Step 16.
b. Go to Step 19		
11 Check Any S/G Level - GREATER THAN 17%		Verify at least 200 gpm AFW flow available. <u>IF NOT</u> , <u>THEN</u> go to Step 17.
12 Check RCS Pressure - GREATER THAN 300 PSIG		Increase RCS pressure to greater than 300 psig. <u>IF</u> RCS pressure can <u>NOT</u> be increased, <u>THEN</u> go to Step 17.
13 Check RCP Status - ANY RCP RUNNING		Perform the followig: a. Establish conditions for starting an RCP. o Verify bus 11A or 11B energized. o Refer to Attachment RCP START. b. Start one RCP. <u>IF</u> an RCP can <u>NOT</u> be started, <u>THEN</u> verify natural circulation. (Refer to Attachment NC.) <u>IF</u> natural circulation <u>NOT</u> verified, <u>THEN</u> increase dumping steam.



EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 11 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
14	<p>Establish Condenser Steam Dump Manual Control:</p> <p>a. Verify condenser available:</p> <ul style="list-style-type: none"> o Any MSIV - OPEN o Annunciator G-15, STEAM DUMP ARMED - LIT <p>b. Place condenser steam dump controller HC-484 in MANUAL</p> <p>c. Place steam dump mode selector switch to MANUAL</p> <p>d. Open steam dump valves as necessary to stabilize RCS temperature</p>	<p>a. Perform the following:</p> <ul style="list-style-type: none"> 1) Place S/G ARV controller in MANUAL and open ARVs as necessary to stabilize RCS temperature. 2) Go to Step 15.
15	<p>Monitor RCS Temperature:</p> <p>a. RCS temperature - STABLE OR DECREASING</p> <p>b. Go to Step 18</p>	<p>a. <u>IF</u> dumping steam does <u>NOT</u> provide adequate cooling, <u>THEN</u> perform the following:</p> <ul style="list-style-type: none"> 1) Initiate S/G blowdown from both S/Gs. 2) Maintain both S/G levels stable by controlling AFW flow. 3) Go to Step 17.



EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 12 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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16 Check RCS Conditions:

- | | |
|--|---|
| <ul style="list-style-type: none"> a. Rx vessel head - REMOVED b. Stop refueling operations if in progress c. Verify Refueling Cavity Level - GREATER THAN 23 FEET ABOVE VESSEL FLANGE d. Verify refueling cavity sweep fans - RUNNING | <ul style="list-style-type: none"> a. Go to Step 17. c. Increase refueling cavity level to greater than 23 feet (Refer to O-15.3, FILLING REFUELING CANAL). d. Locally start refueling cavity sweep fans if available. |
|--|---|

17 Check CCW System Operation:

- o CCW pumps - AT LEAST ONE RUNNING
- o CCW to RHR Hxs, MOV-738A AND MOV-738B - OPEN AS NECESSARY
- o Annunciator A-21, COMP COOLING HX OUT HI TEMP - EXTINGUISHED
- o Annunciator A-22, CCW PUMP DISCHARGE LO PRESS - EXTINGUISHED
- o Annunciator A-30, CCW PUMP INLET HEADER HI TEMP - EXTINGUISHED

To restore CCW cooling to RHR Hxs, perform the following:

- a. Ensure the standby CCW pump is running.
- b. Open MOV-738A and MOV-738B as necessary.

IF CCW can NOT be restored, THEN continue attempts to restore CCW (Refer to AP-CCW.3, LOSS OF CCW - PLANT SHUTDOWN).

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 13 of 13
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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NOTE: Consult with Plant Staff to determine alternatives for long term cooling.

18 Monitor RHR Cooling:

- o RHR cooling - RESTORED
- o RCS temperature - STABLE OR DECREASING

Perform the following:

- a. Evaluate alternatives for long term cooling (Consult Plant Staff)
 - Consider establishing secondary heat sink
 - Refer to ER-RHR.1, RCDT PUMP OPERATION FOR CORE COOLING
 - Consider RCS feed and bleed
- b. Continue attempts to restore RHR to operable.
- c. Return to Step 3.

NOTE: Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.

19 Notify Higher Supervision

20 Return to Procedure Or Guidance In Effect

-END-

EOP: AP-RHR.1	TITLE: LOSS OF RHR	REV: 14 PAGE 1 of 1
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AP-RHR.1 APPENDIX LIST

TITLE

- 1) FIGURE MIN SUBCOOLING (FIG-1.0)
- 2) ATTACHMENT NORMAL RHR COOLING (ATT-14.0)
- 3) ATTACHMENT RCP START (ATT-15.0)
- 4) ATTACHMENT NC (ATT-13.0)
- 5) ATTACHMENT CNMT CLOSURE (ATT-3.1)

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Docket: 05000244



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VICE PRESIDENT & PLANT MANAGER
GINNA STATION

February 1, 2001

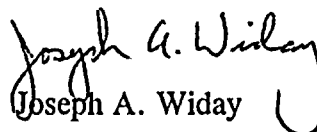
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Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,


Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

ATT Index
E Index
ES Index
ATT-2.1, rev 5
E-1, rev 20
ES-1.3, rev 30

A002

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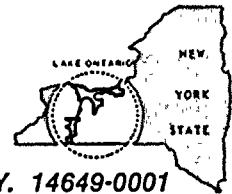
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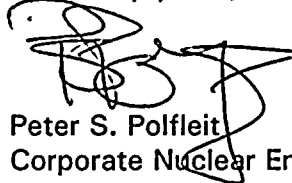
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R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are revisions to Ginna Station Emergency Plan Implementing Procedures (EPIPs).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,



Peter S. Polfleit
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
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Dr. Robert C. Mecredy (2 copies of letter only)

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PROCEDURE

EPIP 1-0

EPIP 2-1

EPIP 5-1

EPIP 5-2

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December 4, 2000

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Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are revisions to Ginna Station Emergency Plan Implementing Procedures (EPIPs).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,

Peter S. Polfleit
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
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GINNA STATION

October 13, 2000

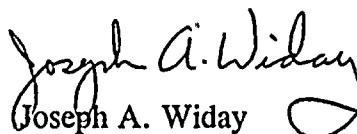
U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,


Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index	AP-RHR.2, Rev. 9
ATT Index	ES-1.3, Rev. 29
ES Index	FIG-2.0, Rev. 2
FIG Index	FIG-4.0, Rev. 1
ATT-15.0, Rev. 6	
ATT-16.2, Rev. 1	
ATT-17.0, Rev. 10	
AP-RHR.1, Rev. 14	

A002

003762431



Distribution Sheet

50-244
10/6/00
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11/2/00
E.J.H.

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Revision to Emergency Plan Implementing Procedures.

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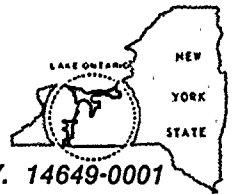
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A045 - OR Submittal: Emergency Preparedness Plans, Implementing Procedures, Correspondence

Docket: 05000244



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TELEPHONE
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October 6, 2000

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555
Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are revisions to Ginna Station Emergency Plan Implementing Procedures (EPIPs).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,

Peter S. Polfle
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

PSP/jtw

A045

003759925

PROCEDURE

REVISION NUMBER

EPIP 1-9

19

EPIP 2-18

12

EPIP 4-7

15

EPIP 5-1

18

EPIP 5-7

29

50-244
9/19/00

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Revised
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Revision 14 for Technical Requirements Manual for Ginna Station.

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9/24

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A001 - OR Submittal: General Distribution for Power Reactor (10 CFR 50) Submittals

Docket: 05000244

50-244

ROCHESTER GAS & ELECTRIC CORPORATION

DOCUMENT CONTROL

Document Transmittal

To: DOCUMENT CONTROL DESK
US NRC
ATT: GUY VISSING
PROJECT DOCTORATE 1-1
WASHINGTON DC 20555

Transmittal: 200009017 055
Date: 09/19/2000
Holder: 1217

LISTED BELOW ARE NEW OR REVISED CONTROLLED DOCUMENTS. REVIEW THE ATTACHED INSTRUCTION LETTER (IF APPLICABLE), AND UPDATE YOUR CONTROLLED DOCUMENT IN ACCORDANCE WITH EP-3-S-0901

Doc Id	Doc Title	Rev	Rev Date	Qty
TRM	TECHNICAL REQUIREMENTS MANUAL FOR GINNA STATION	014	09/20/2000	1

Receipt Acknowledgement and Action Taken:

Signature _____ Date _____

Return ENTIRE FORM to: Rochester Gas & Electric Corporation
Ginna Station - Records Management
1503 Lake Road
Ontario, New York 14519

ADD/

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Rochester Gas and Electric Corporation
Inter-Office Correspondence

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RECORDS MANAGEMENT

September 18, 2000

Subject: Technical Requirements Manual (TRM) Revision 14

To: Distribution

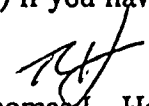
Attached are the revised pages for the Ginna Station Technical Requirements Manual (TRM). The changes included within Revision 14 are summarized as follows:

1. Section TR 3.9.1 has been revised to include requirements for the storage of recently offloaded spent fuel in region 1 of the spent fuel pool. A new Bases section is also included in this change.
2. The entire TRM is being reissued due to changes in software and formatting. The general formatting changes are summarized in the attachment to this correspondence.

Revision 14 of the TRM is considered effective September 20, 2000. Instructions for the necessary changes to your controlled copy of the ITS are as follows:

<u>Volume</u>	<u>Section</u>	<u>Remove</u>	<u>Insert</u>
III	TRM	all pages	new pages

Please contact Tom Harding (extension 3384) if you have any questions.


Thomas L. Harding

The following is a listing of the changes that are being generically applied to the TRM

- 1) The solid line at the bottom or top of a page of continued text has been deleted.
- 2) The use of the words "(continued)" at the bottom or top of a page of continued text has been deleted. Within the Actions tables the page break is maintained only after a 1st level logic so that the continuation is implicit.
- 3) The sub-section title or topic is deleted from each additional page of the sub-section or topic.
- 4) Section 3.0 has been given a combined title due to the two subsections having the same numbering. "LIMITING CONDITION FOR OPERATION (LCO) AND SURVEILLANCE REQUIREMENT (SR) APPLICABILITY"
- 5) For all notes, the words "NOTE" or "NOTES" which currently appear in the middle of a dashed line are moved under the dashed line and the "-NOTE-" is generically used in all cases including plural.
- 6) The page numbers are restarted for each individual sub-section.
- 7) In all tables the footnotes now appear at the end of the table, not on every page of the table.

Distri95.txt

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50-244

9/11/00

Revised

10/18/00

[Signature]

9/18

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A002 - OR Submittal: Inadequate Core Cooling (Item II.F.2) GL 82-28

Docket: 05000244



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www.rge.com

JOSEPH A. WIDAY
VICE PRESIDENT & PLANT MANAGER
GINNA STATION

September 11, 2000

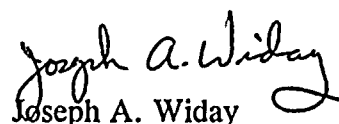
U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,


Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index	ATT-16.1, Rev 0
ATT Index	ATT-16.2, Rev 0
ES Index	ATT-24.0, Rev 0
AP-ELEC.1, Rev 20	ES-1.1, Rev 19
AP-ELEC.3, Rev 8	
AP-ELEC.14/16, Rev 1	
AP-RCS.1, Rev 15	
AP-SG.1, Rev 0	

A002

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Revision 42 to Emergency Plan Implementing Procedures for R.E. Ginna.

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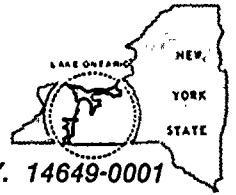
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Docket: 05000244



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001



TELEPHONE
AREA CODE 716 546-2700

September 11, 2000

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555
Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed is a revision to a Ginna Station Emergency Plan Implementing Procedure (EPIP).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,

Peter S. Polflett
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

PSP/jtw

H 4003755319

A045

PROCEDURE

EPIP 1-5

REVISION NUMBER

42

REPORT NO. 01
REPORT: NPSP0200
DOC TYPE: PREPIP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EMERGENCY PLAN IMPLEMENTING PROCEDURE

07/25/00 PAGE: 1

PARAMETERS: DOC TYPES - PREPIP

STATUS: EF

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
EPIP-1-0	GINNA STATION EVENT EVALUATION AND CLASSIFICATION	025	11/19/97	11/19/97	11/19/01	EF
EPIP-1-1	UNUSUAL EVENT	002	12/09/96	12/09/96	12/09/01	EF
EPIP-1-2	ALERT	003	12/09/96	12/09/96	12/09/01	EF
EPIP-1-3	SITE AREA EMERGENCY	005	12/09/96	01/23/98	01/20/02	EF
EPIP-1-4	GENERAL EMERGENCY	004	12/09/96	12/09/96	12/09/01	EF
EPIP-1-5	NOTIFICATIONS	041	07/25/00	07/25/00	07/25/05	EF
EPIP-1-6	SITE EVACUATION	011	07/25/00	07/25/00	07/25/05	EF
EPIP-1-7	ACCOUNTABILITY OF PERSONNEL	008	07/27/99	07/27/99	07/27/04	EF
EPIP-1-8	SEARCH AND RESCUE OPERATION	004	05/16/00	05/16/00	05/16/05	EF
EPIP-1-9	TECHNICAL SUPPORT CENTER ACTIVATION	018	06/21/00	06/21/00	06/21/05	EF
EPIP-1-10	OPERATIONAL SUPPORT CENTER (OSC) ACTIVATION	010	07/25/00	07/25/00	07/25/05	EF
EPIP-1-11	SURVEY CENTER ACTIVATION	021	05/16/00	05/16/00	05/16/05	EF
EPIP-1-12	REPAIR AND CORRECTIVE ACTION GUIDELINES DURING EMERGENCY SITUATIONS	007	06/21/00	06/21/00	06/21/05	EF
EPIP-1-13	LOCAL RADIATION EMERGENCY	003	08/04/95	01/23/98	01/23/02	EF
EPIP-1-15	USE OF THE HEALTH PHYSICS NETWORK HPN	005	04/24/96	03/03/99	03/03/04	EF
EPIP-1-16	RADIOACTIVE LIQUID RELEASE TO LAKE ONTARIO OR DEER CREEK	004	02/13/98	02/13/98	02/13/02	EF
EPIP-1-17	PLANNING FOR ADVERSE WEATHER	002	06/21/00	06/21/00	06/21/05	EF
EPIP-2-1	PROTECTIVE ACTION RECOMMENDATIONS	017	08/20/99	08/20/99	08/20/04	EF
EPIP-2-2	OBTAINING METEOROLOGICAL DATA AND FORECASTS AND THEIR USE IN EMERGENCY DOSE ASSESSMENT	009	02/13/98	02/13/98	02/13/02	EF
EPIP-2-3	EMERGENCY RELEASE RATE DETERMINATION	012	02/04/00	02/04/00	02/04/05	EF
EPIP-2-4	EMERGENCY DOSE PROJECTIONS - MANUAL METHOD	012	06/21/00	06/21/00	06/21/05	EF
EPIP-2-5	EMERGENCY DOSE PROJECTIONS PERSONAL COMPUTER METHOD	010	11/16/99	11/16/99	11/16/04	EF
EPIP-2-6	EMERGENCY DOSE PROJECTIONS - MIDAS PROGRAM	011	06/21/00	06/21/00	06/21/05	EF
EPIP-2-7	MANAGEMENT OF EMERGENCY SURVEY TEAMS	009	10/01/99	10/01/99	10/01/04	EF

REPORT NO. 01
REPORT: NPSPO200
DOC TYPE: PREPIP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EMERGENCY PLAN IMPLEMENTING PROCEDURE

07/25/00 PAGE: 2

PARAMETERS: DOC TYPES - PREPIP

STATUS: EF

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PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
EPIP-2-8	VOLUNTARY ACCEPTANCE OF EMERGENCY RADIATION EXPOSURE	005	05/16/00	05/16/00	05/16/05	EF
EPIP-2-9	ADMINISTRATION OF POTASSIUM IODIDE (KI)	003	12/05/97	12/05/97	12/05/01	EF
EPIP-2-10	INPLANT RADIATION SURVEYS	003	01/16/97	01/16/97	01/16/02	EF
EPIP-2-11	ONSITE SURVEYS	013	07/27/99	07/27/99	07/27/04	EF
EPIP-2-12	OFFSITE SURVEYS	016	07/27/99	07/27/99	07/27/04	EF
EPIP-2-13	IODINE AND PARTICULATE ACTIVITY DETERMINATION FROM AIR SAMPLES	008	07/27/99	07/27/99	07/27/04	EF
EPIP-2-14	POST PLUME ENVIRONMENTAL SAMPLING	013	08/20/99	08/20/99	08/20/04	EF
EPIP-2-15	POST PLUME EVALUATION OF OFFSITE DOSES DUE TO DEPOSITION	004	03/06/98	03/06/98	03/06/03	EF
EPIP-2-16	CORE DAMAGE ESTIMATION	010	02/25/00	02/25/00	02/25/05	EF
EPIP-2-17	HYPOTHETICAL (PRE-RELEASE) DOSE ESTIMATES	005	11/16/99	11/16/99	11/16/04	EF
EPIP-2-18	CONTROL ROOM DOSE ASSESSMENT	011	05/16/00	05/16/00	05/16/05	EF
EPIP-3-1	EMERGENCY OPERATIONS FACILITY (EOF) ACTIVATION AND OPERATIONS	014	02/11/00	02/11/00	02/11/05	EF
EPIP-3-2	ENGINEERING SUPPORT CENTER (ESC)	008	02/25/00	02/25/00	02/25/05	EF
EPIP-3-3	IMMEDIATE ENTRY	007	06/21/00	06/21/00	06/21/05	EF
EPIP-3-4	EMERGENCY TERMINATION AND RECOVERY	007	05/28/99	05/28/99	05/28/04	EF
EPIP-3-7	SECURITY DURING EMERGENCIES	009	11/16/99	11/16/99	11/16/04	EF
EPIP-4-1	PUBLIC INFORMATION RESPONSE TO AN UNUSUAL EVENT	006	02/13/98	02/13/98	02/13/02	EF
EPIP-4-3	ACCIDENTAL ACTIVATION OF GINNA EMERGENCY NOTIFICATION SYSTEM SIRENS	008	02/13/98	02/13/98	02/13/02	EF
EPIP-4-6	JOINT EMERGENCY NEWS CENTER ACTIVATION	008	02/11/00	02/11/00	02/11/05	EF
EPIP-4-7	PUBLIC INFORMATION ORGANIZATION STAFFING	014	07/25/00	07/25/00	07/25/05	EF
EPIP-5-1	OFFSITE EMERGENCY RESPONSE FACILITIES AND EQUIPMENT PERIODIC INVENTORY CHECKS AND TESTS	017	06/21/00	06/21/00	06/21/05	EF
EPIP-5-2	ONSITE EMERGENCY RESPONSE FACILITIES AND EQUIPMENT PERIODIC INVENTORY CHECKS AND TESTS	022	05/24/00	05/24/00	05/24/05	EF
EPIP-5-5	CONDUCT OF DRILLS AND EXERCISES	011	02/25/00	02/25/00	02/25/05	EF

REPORT NO. 01
REPORT: NPSP0200
DOC TYPE: PREPIP

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EMERGENCY PLAN IMPLEMENTING PROCEDURE

07/25/00 PAGE: 3

PARAMETERS: DOC TYPES - PREPIP

STATUS: EF

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
EPIP-5-6	ANNUAL REVIEW OF NUCLEAR EMERGENCY RESPONSE PLAN (NERP)	004	05/28/99	05/28/99	05/28/04	EF
EPIP-5-7	EMERGENCY ORGANIZATION	028	07/25/00	07/25/00	07/25/05	EF
EPIP-5-9	TESTING THE OFF HOURS CALL-IN PROCEDURE AND QUARTERLY TELEPHONE NUMBER CHECK	006	05/28/99	05/28/99	05/28/04	EF
EPIP-5-10	EMERGENCY RESPONSE DATA SYSTEM (ERDS)	005	09/05/97	09/05/97	09/05/02	EF
NERP	ANNUAL UPDATE OF NUCLEAR EMERGENCY RESPONSE PLAN	019	12/09/99	12/09/99	12/09/04	EF
TOTAL FOR PREPIP	52					

ROCKSTER GAS & ELECTRIC CORPORATION

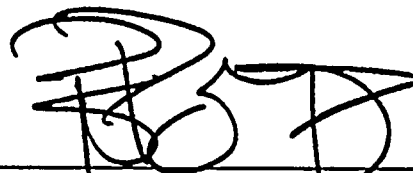
GINNA STATION

Controlled Copy Number 23

Procedure Number EPIP 1-5

Revision Number 41

NOTIFICATIONS



Responsible Manager

07/25/2000

Effective Date

Category 1.0

This procedure contains 24 pages

EPIP 1-5

NOTIFICATIONS

1.0 PURPOSE

The purpose of this procedure is to specify the means by which notifications are made to station personnel for all emergency action levels, to expedite the notification of selected RG&E personnel to augment the emergency response organization and notify offsite agencies.

2.0 RESPONSIBILITY

- 2.1 The Shift Supervisor, Emergency Coordinator or EOF/Recovery Manager is responsible for making the decision to notify offsite agencies.
- 2.2 Ginna Station Control Room personnel are responsible for implementing this procedure.
- 2.3 Community Alert Network (CAN) is responsible for activating the onsite/offsite responders.
- 2.4 The Corporate Nuclear Emergency Planner is responsible for maintaining the station call lists up to date on a quarterly basis.

3.0 REFERENCES

- 3.1 Developmental References
 - 3.1.1 Nuclear Emergency Response Plan
- 3.2 Implementing References
 - 3.2.1 EPIP 1-0, Ginna Station Event Evaluation and Classification
 - 3.2.2 EPIP 2-1, Protective Action Recommendations (PARs)
 - 3.2.3 O-9.3, NRC Immediate Notification
 - 3.2.4 10 CFR 26, Fitness for Duty Programs
 - 3.2.5 P-9, Radiation Monitoring System

3.2.6 EPIP 2-2, Obtaining Meteorological Data and Forecasts and their use in Emergency Dose Assessment

3.2.7 EPIP 4-7, Public Information Organization Staffing

3.2.8 EPIP 5-7, Emergency Organization

4.0 PRECAUTIONS

4.1 New York State, Wayne and Monroe Counties must be notified of all Emergency Classifications within 15 minutes of a declaration.

4.2 The Licensee should notify the USNRC immediately after notification of the appropriate State and local agencies and not later than one hour after the time the licensee declares one of the Emergency Classes.

4.3 Attachment 4 is a specialized notification list of people and organizations who may not require immediate notification but may need to be contacted during an emergency.

5.0 PREREQUISITES

An Emergency has been declared in accordance with EPIP 1-0, Ginna Station Event Evaluation and Classification or offsite assistance has been requested by RG&E personnel.

6.0 ACTIONS

6.1 Shift Supervisor, Emergency Coordinator, EOF/Recovery Manager

6.1.1 Ensure that notifications of all emergency declarations to New York State, Wayne and Monroe Counties are made within 15 minutes of declaring an emergency, in accordance with Attachment 3.

6.1.2 The licensee should notify the USNRC immediately after notification of the appropriate State or local agencies and not later than one hour after the time the licensee declares one of the Emergency Classes using procedure O-9.3 "NRC Immediate Notification".

6.1.3 Upon notification of an Unusual Event at Ginna Station, direct the control room personnel to implement section 6.2.1 of this procedure. If the event is an Alert or higher, implement section 6.2.2.

6.1.4 If additional assistance is required, refer to the NOG E-Plan phone list (in the RG&E telephone directory) in the Control Room and all Emergency Response Facilities, for phone numbers of station personnel.

6.2 Control Room Personnel

6.2.1 Unusual Event - Go to Attachment 1

6.2.2 Alert Classification or Higher - Go to Attachment 2

6.2.3 When offsite assistance has been requested - Go to Attachment 5

7.0 ATTACHMENTS

1. Unusual Event Notifications
2. Alert or Higher Notifications
3. Instructions for New York State Radiological Emergency Data Forms
 - 3a. New York State Radiological Emergency Data Form (Part 1)
 - 3b. New York State Radiological Emergency Data Form (Part 2)
 - 3c. Instructions for Event 1 and Event 2 Printouts and Plant Status Report
 - 3d. Event 1 Supplemental Information Form
 - 3e. Plant Status Report (PPCS not available)
4. Specialized Notification Call List
5. Notifications When Offsite Assistance has been requested
6. Emergency Planning Contingency Notification
7. Management Notification Roster
(This attachment is controlled by Nuclear Emergency Preparedness. It is not included as part of the distributed procedure)

UNUSUAL EVENT NOTIFICATIONS

1. Report information to NEW YORK STATE, WAYNE and MONROE counties within 15 minutes of declaring the emergency via RECS Line using **New York State Radiological Emergency Data Forms (Part 1) Attachment 3a**. Fax the **New York State Radiological Emergency Data Form (Part 1) Attachment 3a** to New York State, Wayne County, Monroe County, TSC, EOF, Survey Center and Joint Emergency News Center.
2. Notify USNRC immediately after the notification of the State and Counties, using procedure O-9.3, NRC Immediate Notification
3. Activate the following positions by stating the following:

"We have an UNUSUAL EVENT at Ginna Station based on

(Initiating Condition)

Please report to the Technical Support Center. The event was declared at _____ hrs. We need to remind you of the Fitness for Duty Requirements. Are you available to report for Duty at this time? If not, we are requesting that you standby so you can be notified for the next call in shift".

A .Plant Manager: Report to the TSC to support the Control Room with offsite communications.

Joe Widay	Business	3250	Will Report (YES/NO)
	Home	716-586-2679	
	Pager	716-528-3977	
	Cellular	716-748-4681	

OR

Dick Marchionda	Business	3699	Will Report (YES/NO)
	Home	315-926-0324	
	Pager	716-464-4403	
	Cellular	716-748-4682	

OR

Jack St. Martin	Business	3641	Will Report (YES/NO)
	Home	716-586-5676	
	Pager	716-464-5287	

OR

Tom Alexander	Business	3898	Will Report (YES/NO)
	Home	315-524-8084	
	Pager	716-783-7045	
	Cellular:	716-748-3233	

UNUSUAL EVENT NOTIFICATIONS

- B. Technical Assessment Manager: Report to the TSC to support the Control Room with offsite communications.

Jeff Wayland	Business	3317	Will Report (YES/NO)
	Home	315-524-2899	
	Pager	716-464-5445	

OR

Ron Ploof	Business	3673	Will Report (YES/NO)
	Home	716-381-9379	
	Pager	716-921-1722	

OR

Brian Flynn	Business	3734	Will Report (YES/NO)
	Home	716-293-1565	
	Pager	716-464-5134	

- C. Operations Assessment Manager: Report to the TSC to support the Control Room with offsite communications.

Terry White	Business	3667	Will Report (YES/NO)
	Home	716-226-9381	
	Pager	716-464-7382	
	Cellular	716-748-4683	

OR

Pete Sidelinger	Business	3509	Will Report (YES/NO)
	Home	716-671-3198	
	Pager	716-463-9830	

OR

Russ Lingl	Business	3415	Will Report (YES/NO)
	Home	716-392-5020	
	Pager	716-527-7841	
	Cellular	716-752-6692	

- D. Ho Nieh
- | | | |
|--|----------|---|
| | Business | 3265 |
| | Home | 315-986-7927 |
| | Pager | 1-800-944-2337 (then dial personal ID# 53133) |

OR

Chris Welch	Business	3265
	Home	(716) 425-2613
	Pager	1-800-944-2337 (then dial personal ID# 51578)

UNUSUAL EVENT NOTIFICATIONS

- E. Corporate Nuclear Emergency Planner: Inform government officials, public relations, PSC and financial department of the event.

Peter Polfleit Business 6772
 Home 716-654-5325
 Pager 716-527-2207
 Cellular 716-733-2207

OR

Frank Cordaro Business 3108
 Home 315-524-2924
 Pager 716-527-3650
 Cellular 716-729-4613

OR

Richard Watts Business 8706
 Home 716-425-2644
 Pager 716-527-3749
 Cellular 716-747-9760

OR

Jill Willoughby Business 4033
 Home 716-787-3156
 Pager 716-528-3295
 Cellular 716-732-7189

4. If the Unusual Event lasts greater than one (1) hour, report information using the New York State Radiological Emergency Data Forms (Part 1) Attachment 3a to New York State, Wayne County, Monroe County, TSC, EOF, Survey Center and Joint Emergency News Center each hour from the time the previous notification was made. Fax the New York State Radiological Emergency Data Form (Part 1) Attachment 3a to New York State, Wayne County, Monroe County, TSC, EOF, Survey Center and Joint Emergency News Center after each report.

ALERT OR HIGHER NOTIFICATIONS

1. Contact Community Alert Network (CAN) at 9-1-800-552-4226 (or at their back-up number of 9-1-518-862-0411). Inform the CAN operator the following information to activate the system:
 - a. This is _____. I am the Ginna Control Room Communicator with RG&E.
(your name)
 - b. My password is: Brookwood
 - c. My callback number is: _____
 - d. This is (circle one): an Actual Event a Drill
 - e. This Emergency Classification declared at: _____
(Time from RECS form)
 - f. Message to deliver (circle one):

Drill Alert Site Area Emergency General Emergency
 - g. My current time is: _____. Please start notifications now.
2. Report information to NEW YORK STATE, WAYNE and MONROE counties within 15 minutes of declaring the emergency via RECS Line using New York State Radiological Emergency Data Forms (Part 1) Attachment 3a. Fax the New York State Radiological Emergency Data Forms (Part 1) Attachment 3a to New York State, Wayne County, Monroe County, TSC, EOF, Survey Center and Joint Emergency News Center.
3. Notify Nuclear Emergency Preparedness of the event. Emergency Preparedness will verify actuation of the emergency response organization notification. If notifications have not begun, Emergency Preparedness will refer to Attachment 6 for contingency notification of one hour responders.

Peter Polfleit	Business	6772
	Home	716-654-5325
	Pager	716-527-2207
	Cellular	716-733-2207

OR

Frank Cordaro	Business	3108
	Home	315-524-2924
	Pager	716-527-3650
	Cellular	716-729-4613

ALERT OR HIGHER NOTIFICATIONS (Continued)

OR

Richard Watts	Business	8706
	Home	716-425-2644
	Pager	716-527-3749
	Cellular	716-747-9760

OR

Jill Willoughby	Business	4033
	Home	716-787-3156
	Pager	716-528-3295
	Cellular	716-732-7189

4. Notify USNRC immediately after the notification of the State and Counties, using procedure O-9.3, NRC Immediate Notification

5. NRC Resident Inspector: Informational call only

Ho Nieh	Business	3265
	Home	315-986-7927
	Pager	1-800-944-2337 (then dial personal ID# 53133)

OR

Chris Welch	Business	3265
	Home	716-425-2613
	Pager	1-800-944-2337 (then dial personal ID# 51578)

6. If the Alert of higher lasts greater than 30 minutes report information using the New York State Radiological Emergency Data Forms (Part 1) Attachment 3a to New York State, Wayne County, Monroe County every 30 minutes from the time the previous notification was made. Fax the New York State Radiological Emergency Data Form (Part 1) Attachment 3a to New York State, Wayne County, Monroe County, TSC, EOF, Survey Center and Joint Emergency News Center after each report.
7. Notify Energy Operations (8944) that Ginna has an emergency and to implement procedures to increase reliability of power to Ginna.
8. If requested by the TSC or EOF, the Control Room will fax the Event 1 Supplemental Information Form, Attachment 3d to the TSC and EOF.

NOTE: Event 1 and Event 2 printouts should not be transmitted by the Control Room, but should be faxed by the TSC Administrative/Communications Staff when it is sufficiently manned to do so.

9. Refer to Attachment 3c for Event 1 and Event 2 instructions.

INSTRUCTIONS FOR NEW YORK STATE RADIOLOGICAL EMERGENCY DATA FORMS

1. The New York State Radiological Emergency Data Form, (Part 1) Attachment 3a should be filled out with the assistance of the Emergency Coordinator or EOF/Recovery Manager and Radiation Protection personnel.
2. For training and drills/exercise, circle "B" - An Exercise. For actual events, circle "A" - NOT An Exercise.
3. The determination for item 6 is made by checking effluent monitor readings against the release rate limits given in procedure P-9.
4. For item 7 of the form, the Emergency Coordinator or EOF/Recovery Manager shall use EPIP 2-1, Protective Action Recommendations (PARs). PARs will only reflect RG&E recommendations, not actions implemented by offsite officials.
5. For item 8 of the form, enter the Emergency Action Level (EAL) number from EPIP 1-0, Ginna Station Event Evaluation and Classification.

NOTE: THE WIND SPEED INDICATOR AT THE 33 FOOT LEVEL IS DESIGNED TO MEASURE ONLY TO 50 MILES PER HOUR.

6. Obtain weather information, items 11-13 of the form, using the plant process computer (PPCS)
OR
If the PPCS is not available, use the Control Room weather indication on the RMS rack.
OR
The Radiation Protection Shift Technician or Dose Assessment Manager will determine the weather and stability class in accordance with procedure EPIP 2-2.
7. The communicator will initial the "prepared by" line at the bottom of the form. The SS/EC (or designee) will approve the form at the bottom prior to transmission. The communicator will ensure all forms are sent to the Corporate Nuclear Emergency Planner (CNEP) at the conclusion of the event.
1. Report the information on the completed New York State Radiological Emergency Data Form (Part 1) Attachment 3a to New York State, Wayne and Monroe Counties within 15 minutes of declaring the emergency using the RECS Line.
 - a. Pick up the receiver and depress "A" then "*" for all call. Wait 10 seconds then depress the "Push to Talk" bar on the handset and state:

"This is Ginna Station, please standby for roll call"
"New York State" (wait for response)
"Monroe County" (wait for response)
"Wayne County" (wait for response)
 - b. Report the information by reading the statement number and the statement including the designation letter.
(e.g. "Item four; Classification; "A" Unusual Event)
 - c. Upon completion of transmitting information, reset system by depressing "A" then "#".
 - d. Hang up receiver.

INSTRUCTIONS FOR NEW YORK STATE RADIOLOGICAL EMERGENCY DATA FORMS

9. IF RECS LINE IS OUT OF ORDER, perform the following:

Call Wayne County 9-1-315-946-9711 (Wayne County Warning Point)
Inform Wayne County, "This is a Ginna Emergency, please hold while we connect Monroe County and New York State". Press the conference button on the telephone.

Call Monroe County 9-528-2222 (Monroe County Warning Point)
Inform Monroe County, "This is a Ginna Emergency". Press the conference button on the telephone.
Wayne and Monroe should now be connected.

Roll call Wayne County_____ Monroe County_____

"Please hold while we connect New York State". Press the conference button on the telephone.

Call New York State 9-1-518-457-2200 (New York State Warning Point)
Inform New York State, "This is a Ginna Emergency". Press conference button on the telephone.
Wayne County, Monroe County and New York State should all be connected.

10. Data in items 15 through 20 of the New York State Radiological Emergency Data Form (Part 2) Attachment 3b should be filled out by the TSC/EOF Dose Assessment and transmitted by fax as information becomes available from the TSC/EOF.
11. Fax all New York State Radiological Emergency Data Forms to the following using the instructions on the fax machine:
- | | |
|-----------------------------|------------------|
| Wayne County | 9-1-315-946-9721 |
| Monroe County | 9-256-6355 |
| New York State | 9-1-518-457-9942 |
| TSC | 3927 |
| EOF | 9-262-5788 |
| Survey Center | 3612 |
| OSC Satellite | 3524 |
| Engineering Support Center | 3774 |
| Joint Emergency News Center | 6771 |
12. Blank copies of the New York State Radiological Emergency Data Form (Part 1 and Part 2) are available in the Control Room.
13. When a County or the State request to be notified only if conditions change or when the event is terminated, check with the State/County warning points to see if they agree. If they all agree, note this in section 8 of the next Part 1 Form notification. The facility with command and control will inform the other RG&E response facilities of the status of notifications. Perform a notification when conditions change or the event is terminated.
14. The New York State Radiological Emergency Data Form (Part 2); Attachment 3b, is filled out by the Dose Assessment Group. The form is transmitted via fax only. The form is transmitted after there has been a release above technical specifications (Part 1 form, Item 6). A release above Technical Specifications is defined as exceeding the "ODCM release rate" limits in P-9.

NOTE: See Attachment 3 for instructions.

NEW YORK STATE RADIOLOGICAL EMERGENCY DATA FORM (PART I)

"This is Ginna Station. Please stand by for roll call." "New York State" ☐ "Monroe County" ☐ "Wayne County" ☐

1. Message transmitted at: Date _____ Time _____ Via: A. RECS B. Other _____		2. This is: A. NOT an exercise B. An exercise	
3. Facility providing information: C. Ginna			
4. Classification: A. UNUSUAL EVENT D. GENERAL EMERGENCY G. TRANSPORTATION INCIDENT B. ALERT E. EMERGENCY TERMINATED C. SITE AREA EMERGENCY F. RECOVERY			
5. Classification Time: This Emergency Classification declared at: Date _____ Time _____			
6. Release of Radioactive Materials: (Refer to P-9) A. No Release above Technical Specifications C. Release to a Body of Water above Technical Specifications B. Release to the Atmosphere above Technical Specifications			
7. Protective Action RECOMMENDATIONS: (Refer to EPIP 2-1) A. No need for Protective Actions outside the site boundary B. Evacuate the following ERPAs W1 W2 W3 W4 W5 W6 W7 M1 M2 M3 M4 M5 M6 M7 M8 M9 C. Shelter all remaining ERPAs			
8. Brief Event Description: EAL # _____			
9. Plant Status: A. Stable C. Degrading E. Cold Shutdown B. Improving D. Hot Shutdown		10. Reactor Shutdown: (subcritical) A. Not Applicable B. Date _____ Time _____	
11. Wind Speed: A. _____ Miles/hour at elevation _____ feet		12. Wind Direction: From: _____ degrees at elevation _____ feet	
13. Stability Class: Unstable, Neutral, Stable	DO NOT REPORT Stability Class Work Sheet Temperature at 250 feet _____ °F Temperature at 33 feet _____ °F Temperature Difference _____ °F -1.74 -0.65 Unstable Neutral Stable -3 -2 -1 0 1 Temperature Difference	14. Reported By: Name _____ Area Code _____ Number _____	

"New York State copy?" ☐ "Monroe County copy?" ☐ "Wayne County copy?" ☐

FOR RG&E USE ONLY:

Time Prepared: _____
Prepared By: _____

Time Approved: _____
Approved By: _____

Completed form sent
to EP - Ginna Training _____

NEW YORK STATE RADIOLOGICAL EMERGENCY DATA FORM (PART II)

Telefax this data form to: ☐ New York State ☐ Monroe County ☐ Wayne County

15. Message transmitted at: Date _____ Time _____ Location/Facility Transmitted From: _____			
16. General Release Information A. Release > Tech Specs started: Date _____ Time _____ B. Release > Tech Specs expected to end: Date _____ Time _____ OR <input type="checkbox"/> Unknown C. Release > Tech Specs ended: Date _____ Time _____ D. Reactor Shutdown: N/A OR Date _____ Time _____ E. Wind Speed: _____ miles/hour at elevation _____ feet F. Wind Direction from: _____ degrees at elevation _____ feet G. Stability Class: PASQUILL A B C D E F G OR Other _____			
17. Atmospheric Release Information A. Release from: <input type="checkbox"/> Ground <input type="checkbox"/> Elevated D. Noble Gas Release Rate _____ Ci/sec B. Iodine/Noble Gas Ratio _____ E. Iodine Release Rate _____ Ci/sec C. Total Release Rate _____ Ci/sec F. Particulate Release Rate _____ Ci/sec			
18. Waterborne Release Information A. Volume of Release _____ gal or liters C. Radionuclides in Release _____ B. Total Concentration _____ μ Ci/ml D. Total Activity Released _____			
19. Dose Calculations (based on a release duration of _____ hours) Calculation is based on (circle one) A. Inplant Measurements B. Field Measurements C. Assumed Source Term			
Table below applies to (circle one) A. Atmosphere Release B. Waterborne Release			
Distance		Xu/Q	
		Dose	
		TEDE (rem)	CDE - Child Thyroid (rem)
Site Boundary			
2 Miles			
5 Miles			
10 Miles			
_____ Miles			
20. Field Measurements of Dose Rates or Surface Contamination/Disposition			
Miles/Sector OR Miles/Degrees	Location OR Sampling Point	Time of Reading	Dose Rate OR Contamination (Include Units)

FOR RG&E USE ONLY: Time Prepared: _____ Time Approved: _____
By: _____ By: _____

Completed form sent to CNEP - 49/2 _____

INSTRUCTIONS FOR EVENT 1 AND EVENT 2 PRINTOUTS AND PLANT STATUS REPORT

1. Assure the Plant Process Computer System (PPCS) is operational. If PPCS is not operational, go to step 5.

NOTE: OBTAIN EVENT 1 AND EVENT 2 PRINTOUTS FROM THE COMPUTER ANALYST IF THAT POSITION IS STAFFED, OTHERWISE PERFORM THE FOLLOWING STEP.

2. Obtain Event 1 and Event 2 printouts by entering:
GASR <return>
Computer response - Enter Group Name
EVENT1 <return>
Computer response - Select Printer Location
Press F1 for Control Room, F2 for EOF, F3 for TSC
GASR <return>
Computer response - Enter Group Name
EVENT2 <return>
Computer Response - Select Printer Location
Press F1 for Control Room, F2 for EOF, F3 for TSC
Place printout in the Event 1 & 2 group trend log book

NOTE: EVENT 1 AND EVENT 2 GROUP TREND (GTLOG) SHOULD BE PRINTED EVERY 15 MINUTES.

3. Initiate Event 1 & 2 group trend log (GT LOG) by entering:
GTLOG <return>
Computer response - Enter Name of Group to Log
EVENT1 <return>
Computer response - Enter Update Rate (30-1800 seconds)
60 <return>
Computer response - Enter Print Interval (1-30 minutes)
15 <return>
Computer Response - Select Printer Location (F1-F4)
Press F1 for F3 for TSC
Initiate Event 1 and Event 2 Group Trend Log in EOF by repeating step 3 and enter F2 when selecting printer location.
Place printouts in Emergency Coordinator or EOF/Recovery Manager notebook
4. Verify with the TSC computer analyst that the PPCX (plant computer data) is being transmitted to New York State, Wayne County and Monroe County via computer modem. If the PPCX (plant computer data) to offsite agencies is unavailable, perform step 2 and fax the printout to New York State, Wayne County and Monroe County.
5. If the PPCS is unavailable, the Plant Status Report (Attachment 3e) must be completed by the Control Room and faxed to the TSC for distribution to New York State, Wayne County, Monroe County and EOF.
6. When completing Attachment 3e, if the parameter is measurable (e.g. pressurizer level) use the numerical value. When the parameter is not measurable, the condition of any deviation from normal should be noted (e.g. core circulation - forced or natural).

EVENT 1 SUPPLEMENTAL INFORMATION FORM

61	Aux Feedwater System	_____Inservice	_____Standby	_____OOS
62	Safety Injection System	_____Inservice	_____Standby	_____OOS
63	Diesel Generators	_____Inservice	_____Standby	_____OOS
64	Containment Fan Cooler System	_____Inservice	_____Standby	_____OOS
65	Service Water System	_____Inservice	_____Standby	_____OOS
66	Post Accident Charcoal Filters	_____Inservice	_____Standby	_____OOS
67	Containment Spray Pumps	_____Inservice	_____Standby	_____OOS
68	Component Cooling System	_____Inservice	_____Standby	_____OOS
69	DC System	A_____v	B_____v	
70	NaOH Tank Level	_____%		

Time Completed: _____

Completed By: _____

PLANT STATUS REPORT (PPCS NOT AVAILABLE)

Plant Parameters		Plant Parameters		Radiation Monitoring	
Reactor Shutdown	YES/NO TIME	Auxiliary Feedwater System	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-1 Control Room	mRem/hr
RCS Pressure	PSIG	Safety Injection	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-2 Containment	mRem/hr
PRZR Level	%	Diesel Generators	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-9 Letdown	mRem/hr
Core Circulation	Forced/Natural	Service Water System	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-10 "A" Containment Iodine	CPM
Subcooled	°F	Cnmt Fan Coolers System	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-11 Containment Particulate	CPM
"A" S/G Level	%	Post Acc. Charcoal Filter	Damper Open / Damper Closed	R-12 Containment Gas	CPM
"B" S/G Level	%	Cnmt. Spray Cnmt. Spray Pumps	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-10 "B" Plant Vent Iodine	CPM
"A" S/G Pressure	PSIG	Comp. Cooling System	<input type="checkbox"/> Inservice <input type="checkbox"/> Standby <input type="checkbox"/> OOS	R-13 Plant Vent Particulate	CPM
"B" S/G Pressure	PSIG	D.C. System	/ Volts	R-14 Plant Vent Gas	CPM
Safeguard	Train B (16/17) EDG/Turbine/Offsite	NaOH Tank Level	%	R-29 Containment High Range	R/hr
Offsite Power	Available/Unavailable	RWST Level	%	R-30 Containment High Range	R/hr
Cnmt Pressure	PSIG	B.A. Tank Level	%	R-15 Air Ejector Gas	CPM
Sump "A" Level	FT	Wind Speed	MPH	*R-12A SPING Containment Gas	µCi/cc
Sump "B" Level	IN	Wind Direction (From)	Degrees	*R14A SPING Plant Vent Gas	µCi/cc
RCS Temp	°F	Temperature 33 FT	°F	*R-15A SPING Air Ejector Gas	µCi/cc
RVLIS	%	Temperature 250 FT	°F	R-31 Steam Line "A"	mRem/hr
CET	°F			R-32 Steam Line "B"	mRem/hr

R/hr = Roentgen/Hour
 µCi/cc = Microcuries/Cubic Centimeter
 mRem/hr = millirem/Hour

*SPING Unit readings may be deleted if
 radiation monitors R-12 and R-14 onTime
 scale.

Date _____
 Completed _____
 Completed By _____

SPECIALIZED NOTIFICATION LIST

Medical

- | | | |
|----|--|--|
| 1. | Ontario Volunteer Emergency Squad | 769-911 (Ginna Control Room Only)
(To request ambulance)
9-1-315-524-5751
(Business number) |
| 2. | Wayne County Emergency Dispatcher | 9-1-315-946-5304 |
| 3. | Rochester General Hospital,
Emergency Department Triage Nurse | 9-338-2300 |
| 4. | Rochester General Hospital Main Switchboard | 9-338-4000 |
| 5. | RG&E Medical Services | Office 8600
Alternate Office 4616
Answering Service 9-226-3800 |
| | Dr. Robert W. George
Dr. T. K. Oates
Dr. Alexander Kurchin | |
| 6. | Newark-Wayne Community Hospital | 9-1-315-332-2267 |

Police

- | | | |
|----|-------------------------------------|--------------------------------------|
| 1. | New York State Police Warning Point | 9-1-518-457-2200
9-1-315-457-6811 |
| 2. | Canandaigua State Police | 9-398-3200 |
| 3. | Williamson State Police | 9-1-800-962-0810 |
| 4. | Wayne County Sheriff | 9-1-315-946-9711 |
| 5. | Monroe County Sheriff | 9-428-5511 |

Fire

- | | | |
|----|-----------------------------------|--|
| 1. | Ontario Volunteer Fire Department | 769-911 (Ginna Control Room Only)
(To report fire)
9-1-315-524-2661
(Business number) |
|----|-----------------------------------|--|

SPECIALIZED NOTIFICATION LIST

Westinghouse Emergency Response Organization

Notify one Westinghouse contact using list in order shown. Provide available facts to individual and provide updates.

1.	Hank Sepp Director ESBU Emergency Response	Home Hotline	9-1-412-374-5282 9-1-412-856-4036 9-1-412-856-6121
2.	Dan Lipman ESBU Service Response Manager	Home	9-1-412-374-6920 9-1-412-744-3244
3.	Rose Cotton ESBU Emergency News Communications ENC Manager	Home	9-1-412-374-6805 9-1-412-963-6129
4.	Mike Young ESBU Emergency Response Technical Support Manager	Home	9-1-412-374-5081 9-1-412-243-7996
5.	Tom Hart ESBU Emergency Response Logistic Manager	Home Hotline Pager	9-1-412-374-6980 9-1-412-837-9486 9-1-412-837-1737 9-1-412-765-8886

Other

1.	Ontario Town Supervisor, Roy Hermann	Office Home	9-1-315-524-7105 9-1-315-524-8087
2.	Ontario Water Department		9-1-315-524-2941
3.	Plant Protection Department Kodak Park		9-722-2122
4.	Wayne County Emergency Operations Center		9-1-315-946-5663
5.	Director Wayne County Office of Disaster Preparedness - Thelma Wideman	Home	9-1-315-597-6291
6.	Monroe County Office of Emergency Preparedness (Nights, Weekends, Holidays)	Daytime Offhours	9-473-0710 9-528-2222

SPECIALIZED NOTIFICATION LIST (Cont'd.)

7.	Administrator, Monroe County Office of Emergency Preparedness - Mary Louise Meisenzahl	Home Pager	9-624-3194 9-428-5141
8.	University of Rochester Advance RAP Team - David Maillie	Home	9-275-3788 9-334-2428
9.	National Weather Service (Buffalo)		9-1-800-462-7751
10.	Radiation Management Consultants	Office Emergency Fax	9-1-215-824-1300 9-1-215-243-2990 9-1-215-824-1371
11.	Helgeson Nuclear Services Inc		9-1-415-846-3453
12.	James C. Hutton (NSARB)		9-1-716-381-8473
13.	Institute of Nuclear Power Operations		9-1-800-321-0614
14.	American Nuclear Insurers		9-1-203-677-7305
15.	Emergency Preparedness Canada	Phone Fax	9-1-613-991-7000 9-1-613-996-0995
16.	NYPA Environmental Laboratory Fulton, New York	Daytime	9-1-315-593-5740 9-1-315-593-5735
		Lab Manager pager	9-1-800-436-2732 enter pager # 713-6710 then your number
		Mgr Home #	9-1-315-342-0015
		RES on call pager	9-1-800-436-2732 enter pager # 713-6726 then your number

SPECIALIZED NOTIFICATION LIST (Cont'd.)

Company Personnel

1.	Thomson, Bill Manager, Radiation Protection and Chemistry	Business Home Pager	3219 315-342-5082 716-528-8561
2.	Richards, Thomas Chief Executive Officer	Business Home	8299 (716) 288-9186
3.	Lappan, George Manager of Corporate Communications	Business Home	8812 716-377-6858
4.	Mecredy, Robert Vice President Nuclear Operations	Business Home Pager	3494 716-381-6430 716-783-4900
5.	Wilkins, Paul Sr. Vice President Generation	Business Home Pager: Cellular	8076 716-248-2385 716-529-6426 716-451-9761
6.	Watts, Richard Manager, Nuclear Training	Business Home Pager Cellular	8706 716-425-2644 716-527-3749 716-747-9760

Nuclear Regulatory Commission

1.	Nuclear Regulatory Commission Region 1 - King of Prussia, PA	610-337-5000
2.	Radiation Assistance Program Dept of Energy Brookhaven National Lab	516-282-2200
3.	Commercial telephone system to NRC Operations Center (via Bethesda Central Office)	301-951-0550
4.	Commercial telephone system to NRC Communications Center (via Silver Spring Central Office)	301-427-4056
5.	Commercial telephone system to NRC Operator (via Bethesda Central Office)	301-492-8893

SPECIALIZED NOTIFICATION LIST (Cont'd.)**New York State**

- | | | |
|----|--|------------------------------|
| 1. | James Baranski,
State Emergency Management Office
(SEMO) | 518-457-8909 |
| 2. | SEMO Lake District | 315-331-4880 |
| 3. | NYS Department of Health
Rochester Office | 716-262-2010 |
| 4. | New York State Emergency
Operations Center (EOC) Albany | 518-454-3337 |
| 5. | EOC Albany - Dose Assessment | 518-454-3321
518-454-2176 |

Federal Emergency Management Agency (FEMA)

- | | | |
|----|--|------------------------------|
| 1. | Emergency Information Coordination
Center | 202-634-7800
202-646-2400 |
|----|--|------------------------------|

NOTIFICATIONS WHEN OFFSITE ASSISTANCE HAS BEEN REQUESTED

1. When offsite assistance has been requested activate:

- Security
- Nuclear Management
- Emergency Planning

Examples of initiating events that could require offsite assistance are:

- Fire
- Medical Emergency
- Security Event
- HAZMAT Incident
- Natural Events (such as flooding, earthquakes or severe weather)

2. Security

Contact Security at 3210, so that they can make preparations for the arrival of the emergency vehicles and personnel.

3. Nuclear Management

Notify the following individuals:

"This is the Ginna Control Room. We have requested offsite assistance from _____. Can you be the Nuclear Management contact for this event? Your duties are (a) act as the RG&E lead for this event and (b) act as the liaison between the Control Room and the corporation."

Nuclear Management (One person required to respond)

	Joe Widay	Business	3250	Available (YES/NO)
		Home	716-586-2679	
		Pager	716-528-3977	
		Cellular	716-748-4681	
OR	Dick Marchionda	Business	3699	Available (YES/NO)
		Home	315-926-0324	
		Pager	716-464-4403	
		Cellular	716-748-4682	
OR	Bob Mecredy	Business	8069	Available (YES/NO)
		Home	716-381-6430	
		Pager	716-783-4900	

The nuclear management representative may call other nuclear managers or members of the Ginna leadership team.

NOTIFICATIONS WHEN OFFSITE ASSISTANCE HAS BEEN REQUESTED**4. Emergency Planning**

Notify the following individuals:

"This is the Ginna Control Room. We have requested offsite assistance from _____. Can you be the Emergency Planning contact for this event? Your duties are (a) activate Public Relations and (b) act as the liaison between the Control Room and government agencies.

_____ is acting as the Nuclear Management lead for this event. He can be reached at _____."

Nuclear Emergency Preparedness (One person required to respond)

OR	Peter Polfleit	Business	6772
		Home	716-654-5325
		Pager	716-527-2207
		Cellular	716-733-2207

OR	Frank Cordaro	Business	3108
		Home	315-524-2924
		Pager	716-527-3650
		Cellular	716-729-4613

OR	Richard Watts	Business	8706
		Home	716-425-2644
		Pager	716-527-3749
		Cellular	716-747-9760

	Jill Willoughby	Business	4033
		Home	716-787-3156
		Pager	716-528-3295
		Cellular	716-732-7189

The Emergency Planning representative will call the duty public information officer (PIO) via the ECC at 771-2233, and inform them of the event. The duty PIO will determine if a media announcement is warranted. The Emergency Planning representative will also contact Wayne County, Monroe County and New York State officials to brief them on offsite resources being used

NOTIFICATIONS WHEN OFFSITE ASSISTANCE HAS BEEN REQUESTED

5. Contact the NRC resident inspector

Ho Nieh	Business	3265
	Home	315-986-7927
	Pager	1-800-944-2337 (then dial personal ID# 53133)

OR

Chris Welch	Business	3265
	Home	716-425-2613
	Pager	1-800-944-2337 (then dial personal ID# 51578)

EMERGENCY PLANNING CONTINGENCY NOTIFICATION

67. Upon verification that the Community Alert Network System or Group Page for one hour response positions does not activate or function properly, begin manual notification process.
68. Notify other Nuclear Emergency Preparedness staff members to assist with contingency notifications.
69. The following one hour response positions should be filled by contacting a minimum of one responder for each position by individual page or by home, office or cellular phone number. Refer to EPIP 4-7, Public Information Organization Staffing, and EPIP 5-7, Emergency Organization.
 - TSC Emergency Coordinator
 - Operations Assessment Manager
 - Technical Assessment Manager
 - Communicator
 - TSC Dose Assessment Manager
 - RP/Chemistry Manager
 - Maintenance Assessment Manager
 - Survey Center Manager

 - EOF Recovery Manager
 - Nuclear Operations Manager
 - Engineering Manager
 - EOF Dose Assessment Manager

 - News Center Manager
4. Inform the responder of the current emergency classification and instruct them to report to their emergency duty location immediately. Inform them of the fitness for duty requirements.