

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9702030046 DOC. DATE: 97/01/23 NOTARIZED: NO DOCKET #
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 WIDAY, J.A. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 VISSING, G.S.

Rev 5/14/97 W

SUBJECT: Forwards revised Emergency Operating Procedures for plant.

DISTRIBUTION CODE: A002D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5+250 C
 TITLE: OR Submittal: Inadequate Core Cooling (Item II.F.2) GL 82-28 A

NOTES: License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244 T

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	
	PD1-1 PD	1 1	VISSING, G.	1 1	E
INTERNAL:	FILE CENTER 01	1 1	NRR/DRPM/PECB	1 1	G
	NRR/DSSA/SRXB	1 1	RES DE	1 1	O
	RES/DET/EIB	1 1	RES/DSIR/RPSB	1 1	R
EXTERNAL: NOAC		1 1	NRC PDR	1 1	Y

C
A
T
E
G
O
R
Y

1

D
O
C
U
M
E
N
T

NOTE TO ALL "RIDS" RECIPIENTS:
 PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,
 ROOM OWFN 5D-5(EXT. 415-2083) TO ELIMINATE YOUR NAME FROM
 DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 10 ENCL 10



2

100

100

100

100

100

100

100

100

100

100

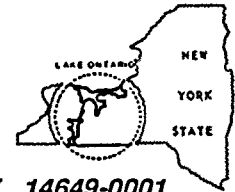
100

100

100

100

100



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

JOSEPH A. WIDAY
Plant Manager
Ginna Nuclear Plant

TELEPHONE
AREA CODE 716 546-2700

January 23, 1997

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

Joseph A. Widay
Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s): 030021

E Index	E-1, Rev. 14
ECA Index	E-3, Rev. 19
ES Index	ECA-2.1, Rev. 14
FR Index	ECA-3.3, Rev. 15

ES-1.3, Rev. 20
FR-C.1, Rev. 13
FR-C.2, Rev. 12
FR-P.1, Rev. 14

(9702030046 970123
PDR ADDCK 05000244
F PDR)

11/1
A002

REPORT NO. 01
REPORT: NPSP0200
DOC TYPE: PRES

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EQUIPMENT SUB-PROCEDURE

01/10/97 PAGE: 3

PARAMETERS: DOC TYPES - PRER PRES PRFR

STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
ES-0.0	REDIAGNOSIS	009	04/07/94	04/07/94	04/07/99	EF
ES-0.1	REACTOR TRIP RESPONSE	012	06/03/96	04/07/94	04/07/99	EF
ES-0.2	NATURAL CIRCULATION COOLDOWN	009	06/03/96	04/20/95	04/20/00	EF
ES-0.3	NATURAL CIRCULATION COOLDOWN WITH STEAM VOID IN VESSEL	006	06/03/96	04/07/94	04/07/99	EF
ES-1.1	SI TERMINATION	012	06/03/96	04/20/95	04/20/00	EF
ES-1.2	POST LOCA COOLDOWN AND DEPRESSURIZATION	014	06/03/96	04/20/95	04/20/00	EF
ES-1.3	TRANSFER TO COLD LEG RECIRCULATION	019	01/10/97	03/26/93	03/26/98	EF
ES-3.1	POST-SGTR COOLDOWN USING BACKFILL	010	06/03/96	04/20/95	04/20/00	EF
ES-3.2	POST-SGTR COOLDOWN USING BLOWDOWN	011	06/03/96	04/20/95	04/20/00	EF
ES-3.3	POST-SGTR COOLDOWN USING STEAM DUMP	011	06/03/96	04/20/95	04/20/00	EF

TOTAL FOR PRES 10

Superseded per PM Per
to SOP dtd 1/23/97
9702030046
50-244
23

REPORT NO. 01
REPORT NAME: DOC-P001
DOC TYPE: PRF

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
CRITICAL SAFETY FUNCTION STATUS PROC

01/25/95 08:54 PAGE 2

23

PARAMETERS: DOC TYPES - PRES PRF PRFR PRPT PRPTT STATUS:

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
F-0.1	SUBCRITICALITY CSFST	001	07/21/89	07/13/94	01/13/97	EF
F-0.2	CORE COOLING CSFST	003	07/21/89	07/13/94	01/13/97	EF
F-0.3	HEAT SINK CSFST	002	01/12/90	07/13/94	01/13/97	EF
F-0.4	INTEGRITY CSFST	001	07/21/89	07/13/94	01/13/97	EF
F-0.5	CONTAINMENT CSFST	002	01/12/90	07/13/94	01/13/97	EF
F-0.6	INVENTORY CSFST	003	01/12/90	07/13/94	01/13/97	EF

TOTAL FOR PRF 6

REPORT NO. 01
REPORT: NPSPO200
DOC TYPE: PRFR

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
FUNCTIONAL RESTORATION GUIDELINE PROC

01/10/97 PAGE: 4

PARAMETERS: DOC TYPES - PRER PRES PRFR STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
FR-C.1	RESPONSE TO INADEQUATE CORE COOLING	012	01/10/97	04/07/94	04/07/99	EF
FR-C.2	RESPONSE TO DEGRADED CORE COOLING	011	01/10/97	04/07/94	04/07/99	EF
FR-C.3	RESPONSE TO SATURATED CORE COOLING	007	01/10/97	03/02/95	03/02/00	EF
FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	015	11/08/96	04/20/95	04/20/00	EF
FR-H.2	RESPONSE TO STEAM GENERATOR OVERPRESSURE	003	05/31/96	02/22/95	02/22/00	EF
FR-H.3	RESPONSE TO STEAM GENERATOR HIGH LEVEL	004	05/31/96	02/22/95	02/22/00	EF
FR-H.4	RESPONSE TO LOSS OF NORMAL STEAM RELEASE CAPABILITIES	003	05/31/96	04/21/93	04/21/98	EF
FR-H.5	RESPONSE TO STEAM GENERATOR LOW LEVEL	003	09/26/96	02/22/95	02/22/00	EF
FR-I.1	RESPONSE TO HIGH PRESSURIZER LEVEL	007	05/31/96	04/20/95	04/20/00	EF
FR-I.2	RESPONSE TO LOW PRESSURIZER LEVEL	004	05/03/91	07/28/93	07/28/98	EF
FR-I.3	RESPONSE TO VOIDS IN REACTOR VESSEL	008	06/05/96	10/19/94	10/19/99	EF
FR-P.1	RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK CONDITION	013	11/08/96	04/20/95	04/20/00	EF
FR-P.2	RESPONSE TO ANTICIPATED PRESSURIZED THERMAL SHOCK CONDITION	005	06/05/96	11/08/94	11/08/99	EF
FR-S.1	RESPONSE TO REACTOR RESTART/ATWS	010	05/31/96	04/20/95	04/20/00	EF
FR-S.2	RESPONSE TO LOSS OF CORE SHUTDOWN	006	05/31/96	04/20/95	04/20/00	EF
FR-Z.1	RESPONSE TO HIGH CONTAINMENT PRESSURE	003	10/16/92	02/22/95	02/22/00	EF
FR-Z.2	RESPONSE TO CONTAINMENT FLOODING	002	04/09/90	02/22/95	02/22/00	EF
FR-Z.3	RESPONSE TO HIGH CONTAINMENT RADIATION LEVEL	003	09/30/92	02/22/95	02/22/00	EF

TOTAL FOR PRFR 18

REPORT NO. 01
REPORT NAME: DOC-P001
DOC TYPE: PRE

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EMERGENCY PROCEDURE

01/25/95 08:50 PAGE 5

23

PARAMETERS: DOC TYPES - PRAP PRATT PRE PRECA PRER STATUS:

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
E-0	REACTOR TRIP OR SAFETY INJECTION	019	04/07/94	04/07/94	10/07/96	EF
E-1	LOSS OF REACTOR OR SECONDARY COOLANT	011	03/26/93	03/26/93	09/26/95	EF
E-2	FAULTED STEAM GENERATOR ISOLATION	006	07/08/93	07/08/93	01/08/96	EF
E-3	STEAM GENERATOR TUBE RUPTURE	015	06/09/94	04/07/94	10/07/96	EF

TOTAL FOR PRE 4

REPORT NO. 01
REPORT NAME: DOC-P001
DOC TYPE: PRECA

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EMERGENCY CONTINGENCY ACTIONS PROC

01/25/95 08:50 PAGE 6

23

PARAMETERS: DOC TYPES - PRAP PRATT PRE PRECA PRER STATUS:

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
ECA-0.0	LOSS OF ALL AC POWER	016	11/08/94			SS
ECA-0.1	LOSS OF ALL AC POWER RECOVERY WITHOUT SI REQUIRED	010	06/09/94	04/07/94	10/07/96	EF
ECA-0.2	LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED	008	04/07/94	04/07/94	10/07/96	EF
ECA-1.1	LOSS OF EMERGENCY COOLANT RECIRCULATION	009	04/07/94	04/07/94	10/07/96	EF
ECA-1.2	LOCA OUTSIDE CONTAINMENT	002	04/09/90	09/23/92	03/23/95	EF
ECA-2.1	UNCONTROLLED DEPRESSURIZATION OF BOTH STEAM GENERATORS	010	06/09/94	04/07/94	10/07/96	EF
ECA-3.1	SGTR WITH LOSS OF REACTOR COOLANT-SUBCOOLED RECOVERY DESIRED	010	04/07/94	04/07/94	10/07/96	EF
ECA-3.2	SGTR WITH LOSS OF REACTOR COOLANT-SATURATED RECOVERY DESIRED	013	04/07/94	04/07/94	04/07/96	EF
ECA-3.3	SGTR WITHOUT PRESSURIZER PRESSURE CONTROL	011	06/09/94	04/07/94	10/07/96	EF

TOTAL FOR PRECA 9

