

# CATEGORY 1

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 MECREDY, R.C.      Rochester Gas & Electric Corp.  
 RECIP. NAME      RECIPIENT AFFILIATION  
 VISSING, G.

*See Reports*

SUBJECT: Provides addl info to assist in exam of 970318 ltr to NRC  
 re "Reactor Coolant Pump Flywheel Insp Interval Change."  
 Also forwards Section XI, rev 0 & revised Section XI, rev 1  
 which contains updated RCP requirements & exams performed.

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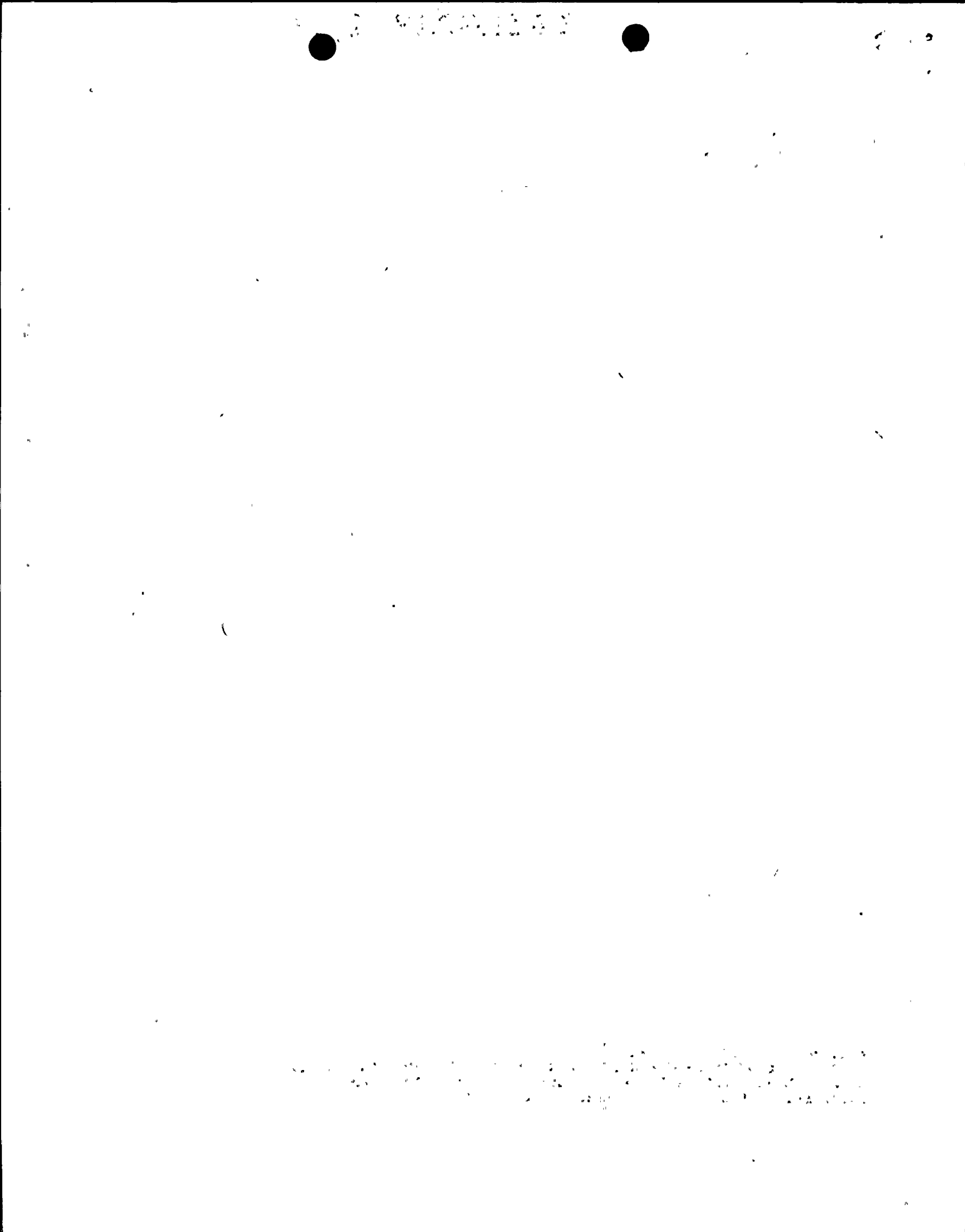
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ROBERT C. MECREDY  
Vice President  
Nuclear Operations

July 8, 1997

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Attn: Guy Vissing  
Project Directorate I-1  
Washington, D.C. 20555

Subject: Reactor Coolant Pump Flywheel Inspection Interval Change  
Additional Information  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

Ref.(a): Letter from Robert C. Mecredy, (RG&E), to Guy S. Vissing,  
(NRC), "Reactor Coolant Pump Flywheel Inspection Interval  
Change," dated March 18, 1997.

Dear Mr. Vissing:

The purpose of this letter is to provide additional information to  
assist you in the examination of the referenced transmittal.

The reactor coolant pump (RCP) flywheel inspection was not part of  
the previous R.E. Ginna Technical Specifications. As such, it has  
not been carried into the Improved Technical Specifications. The  
previous Inservice Inspection (ISI) program document related to RCP  
flywheel inspections is Section XI Revision 0. The revised ISI  
program document is Section XI Revision 1 which contains the  
updated RCP requirements.

Copies of these two documents are enclosed. In addition, a copy of  
the examinations performed during the second period of the third  
interval are enclosed.

Should you have any questions, please call George Wrobel at (716)  
724-8070.

Very truly yours,

*Thomas A. Marlow* For

Robert C. Mecredy

Enclosures  
REJ\470

9707170007 970708  
PDR ADOCK: 05000244  
Q PDR



xc: Mr. Guy Vissing (Mail Stop 14B2)  
Project Directorate I-1  
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
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Ginna Senior Resident Inspector



## ADDITIONAL PROGRAM

**SECTION 11**

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### 1.0 General:

- 1.1 The purpose of this section is to provide information and clarification on additional inspection programs being performed at R. E. Ginna Nuclear Power Plant. These inspection programs may be due in part to ASME Section XI program requirements or to other commitments made by Rochester Gas and Electric.

The following list identifies additional inspection programs being performed at R. E. Ginna Nuclear Power Plant:

- \* Steam Generator Tube Inspection Program
- \* Reactor Coolant Pump Flywheel Program
- \* Reactor Vessel Augmented Program, Category B-A

### 2.0 Steam Generator Tube Inspection Program:

#### 2.1 General:

- 2.1.1 The Steam Generator Tube Inspection Program incorporates the requirements of ASME Section XI Code, under Category B-Q, Item Number B16.20. The Code requires that Steam Generator tubing in U-Tube Design be volumetrically (Eddy Current) examined to the extent and frequency governed by the plant Technical Specifications. In accordance with this Code requirement and R. E. Ginna Station Technical Specifications, eddy current examinations shall be performed. Steam Generator Tubing shall be examined their full length, at least once every five years.



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2.1.2 The Steam Generator Tube Inspection Program also incorporates the requirements of USNRC Regulatory Guide 1.83, Revision 1, dated July, 1975, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes" and the recommendations of the Electric Power Research. (EPRI) PWR Steam Generator Inspection Guidelines, latest approved revision.

### 2.2 Inspection Program:

2.2.1 The program for each inspection cycle of the five years shall include, as a minimum, the following requirements:

1. A rotating random sampling of 20% of all operational tubes for their full length. (Assuring that 100% of tubes shall be completed within 5 years.)
2. A rotating random sample of 20% of each type of sleeved inlet tube for their full length, including the sleeve from the upper end through the expanded transition of the lower end.
3. All operational tubes that had a previously identified service induced degradation of greater than 20% through wall to the extent of previously identified degradation. However, if after two (2) consecutive inspections these tubes have not had greater than 10% further penetration, the inspection frequency on these tubes may be extended to 40 months.



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2.2.2 Other tubes may be added to the program each inspection cycle as necessary to meet other concerns and are classified as "owner elected". "Owner Elected" examinations are not mandatory and may be performed as determined by the owner.

2.2.3 The Ginna Steam Generator Reliability Committee may change the aforementioned plan to meet outage schedules, provided that the changes meet the requirement of Regulatory Guide 1.83 and Supplement 1 the Inservice Inspection Program Plan.

### 2.3 Examination Method:

2.3.1 Eddy Current (Volumetric) Examination techniques shall be employed to perform the required examinations on Steam Generator tubes.

### 2.4 Frequency of Examinations:

2.4.1 Examinations shall be performed every inspection cycle to the extent required and specified within 2.2, as a minimum.

### 2.5 Examination Evaluation:

2.5.1 Eddy Current evaluation shall be performed in accordance with approved procedures.



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### 2.6 Repair, Replacement and Testing Requirements

2.6.1 Repair criteria for steam generator tubes is based on the requirements of Regulatory Guide 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes".

2.6.1.1 Steam generator tubes that have imperfections greater than 40 percent through-wall as indicated by eddy current, shall be repaired by plugging or sleeving.

2.6.1.2 Steam generator sleeves that have imperfections greater than 30 percent through wall as indicated by eddy current shall be repaired by plugging.

2.6.2 Repairs by welded plugs and sleeves shall be performed in accordance with Section 12.

### 2.7 Scheduling:

2.7.1 Eddy Current examination schedules of Steam Generator Tubes shall be established within Supplement 1, the Inservice Inspection Program Plan.

### 2.8 Reports and Records:

2.8.1 Applicable records shall be maintained as specified within Section 1 and Section 12.

2.8.2 Within 15 days following the completion of the evaluation of each inservice inspection of steam generator tubes, the number of tubes required by Paragraph 2.6.1 above to be plugged or sleeved in each steam generator shall be reported to the NRC in a Special Report.







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2.8.3 The complete results of the steam generator tube inservice inspection shall be submitted to the NRC in a Special Report within 12 months following the completion of the inspection. This Special Report shall include:

- (a) Number of tubes inspected and extent to which inspected.
- (b) Location and percent of wall-thickness penetration for each indication of an imperfection, and
- (c) Identification of tubes plugged or sleeved.

2.8.4 If the number of tubes in a generator falling into categories (a) or (b) below exceeds the criteria, then results of the inspection shall be considered a Reportable Event pursuant to 10 CFR 50.73. Oral notification to the NRC Staff shall be accomplished within 48 hours, but no sooner than the next normal working day after the final review of the eddy current results. A written follow-up report shall provide a description of investigations conducted to determine the cause of the tube degradation and corrective measures taken to preclude recurrence. Categories (a) and (b) are:

- (a) More than 10 percent of the total tubes inspected are degraded (imperfections greater than 20 percent of the nominal wall thickness). However, previously degraded tubes must exhibit at least 10 percent further wall penetration to be included in this calculation.





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- (b) More than 1 percent of the total tubes inspected are degraded (imperfections greater than the repair limit).

3.0 Reactor Coolant Pump Flywheel Program:

3.1 General:

The augmented inservice inspection program for Reactor Coolant Pump Flywheels incorporates the requirements of the USNRC Regulatory Guide 1.14, Revision 1, dated August 1975, entitled "Reactor Coolant Pump Flywheel Integrity".

3.2 Examination Requirements:

- 3.2.1 Examinations shall be performed on all active Reactor Coolant Pump Flywheels and Anti-Rotation Pawls.

3.3 Examination Method:

- 3.3.1 Reactor Coolant Pump Flywheels shall be examined using Ultrasonic and Surface examination techniques.
- 3.3.2 Reactor Coolant Pump Anti-Rotation Pawls shall be examined using Surface examination techniques.
- 3.3.3 Ultrasonic and Surface examinations shall conform to and be performed in accordance with Section 1 of this program.



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### 3.4 Frequency of Examinations:

3.4.1 Examinations shall be performed on all operating Reactor Coolant Pump Flywheel and Anti-Rotation Pawls once each period.

### 3.5 Examination Evaluation:

3.5.1 Examination evaluations shall be performed in accordance with Section 1 of this program.

3.5.2 Unacceptable examinations shall be reported for evaluation and appropriate corrective action.

### 3.6 Repair, Replacement and Testing Requirements

3.6.1 Repairs and Replacements shall be performed in accordance with Section 12, as applicable.

### 3.7 Scheduling:

3.7.1 Examination schedules shall be established within Supplement 1, the Inservice Inspection Program Plan.

### 3.8 Reports and Records:

3.8.1 Applicable records shall be maintained as specified in Section 1 and Section 12.



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### 4.0 Reactor Vessel Augmented Program, Category B-A:

#### 4.1 General:

As specified within the Federal Register, 10 CFR Part 50, Vol. 57, No. 152 dated August 6, 1992, a specific augmented examination program is required for ASME Section XI Category B-A, Shell Welds of "Pressure Retaining Welds in Reactor Vessel".

#### 4.2 Examination Requirements:

4.2.1 Examinations shall be performed on all ASME Section XI Category B-A, Item B1.0, Shell Welds of "Pressure Retaining Welds in Reactor Vessel" that are not required by the Code.

#### 4.3 Examination Method:

4.3.1 Ultrasonic examinations shall conform and be performed in accordance with Section 1 of this program.

#### 4.4 Frequency of Examinations:

4.4.1 Examinations shall be performed once during the Inspection Interval.

#### 4.5 Examination Evaluation:

4.5.1 Examination Evaluations shall be performed in accordance with Section 1 of this program.



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4.5.2 Unacceptable examinations shall be reported for evaluation and appropriate corrective action.

4.6 Repair, Replacement and Testing Requirements

4.6.1 Repairs and Replacements shall be performed in accordance with Section 12, as applicable.

4.7 Scheduling:

4.7.1 Examination schedules shall be established within Supplement 1, the Inservice Inspection Program Plan.

4.8 Reports and Records:

4.8.1 Applicable records shall be maintained as specified within Section 1 and Section 12.