

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9207090120 DOC. DATE: 92/06/26 NOTARIZED: NO DOCKET #  
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244  
 AUTH. NAME AUTHOR AFFILIATION  
 MECREDY, R.C. Rochester Gas & Electric Corp.  
 RECIP. NAME RECIPIENT AFFILIATION

JOHNSON, A. Project Directorate I-3

SUBJECT: Revised EOPs, including Rev 1 to ATT, Rev 2 to ATT, Rev 8 to  
 AP-SW.1, Rev 6 to FR-S.1 & Rev 11 to FR-H.1.W/920626 ltr.

DISTRIBUTION CODE: A003D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 79  
 TITLE: OR/Licensing Submittal: Suppl 1 to NUREG-0737 (Generic Ltr 82-33)

NOTES: License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

*Revised 7/16/92*

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
PD1-3 LA	1 1	PD1-3 PD	1 1
JOHNSON, A	2 2		
INTERNAL: NRR/DLPQ/LHFB11	1 1	OC/LFMB	1 0
<u>REG FILE 01</u>	1 1	RES/DSIR/EIB	1 1
EXTERNAL: NRC PDR	1 1	NSIC	1 1

*Cont No*

*P74897848*

NOTE TO ALL "RIDS" RECIPIENTS:

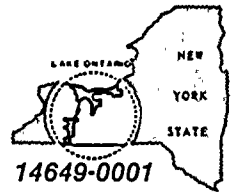
PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,  
 ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION  
 LISTS FOR DOCUMENTS YOU DON'T NEED!

*MA*

TOTAL NUMBER OF COPIES REQUIRED: LTTR 10 ENCL 9



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER N.Y. 14649-0001



ROBERT C. MECREDY  
Vice President  
Ginna Nuclear Production

TELEPHONE  
AREA CODE 716 546-2700

June 26, 1992

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Attn: Mr. Allen Johnson  
Project Directorate I-3  
Washington, D.C. 20555

Subject: Emergency Operating Procedures  
R. E. Ginna Nuclear Power Plant  
Docket No. 50-244

Gentlemen:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

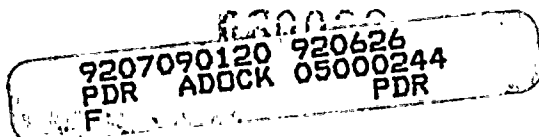
Very truly yours,

*Robert C. Mecredy*  
Robert C. Mecredy

Enclosures

c: Mr. Lee Bettenhausen, USNRC, Region 1  
Resident Inspector, Ginna Station

emergency.pro



*Ent No*  
*9748978418*  
*A0013*  
*11*



12/12/12

12/12/12

12/12/12

ATT  
ATT  
ATT

AP-SW.1  
AP-TURB.1  
FR-S.1  
FR-H.1

Abnormal Procedures Index - issued 6/23  
EOP/AP Attachment Index - 6/23/92  
EOP/AP Attachments - Rev. 1 - 5/13/92 - SW Isolation  
EOP/AP Attachments - SW Loads in CNMT  
EOP/AP Attachments - Rev. 2 - 5/14/92 -  
Flow Through MFW Pump B  
Service Water Leak - Rev. 8  
Turbine Trip Without RX Trip Required, Rev. 5  
Response to Reactor Restart/ATWS - Rev. 6  
Response to Loss of Secondary Heat Sink - Rev. 11

ISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	7	90/02/28
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	8	89/12/19
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	7	89/12/19
AP-CR.1	CONTROL ROOM INACCESSIBILITY	11	91/05/10
AP-CVCS.1	CVCS LEAK	5	89/12/19
AP-CVCS.2	IMMEDIATE BORATION	7	90/02/23
AP-CW.1	LOSS OF A CIRC WATER PUMP	4	90/02/23
AP-ELEC.1	LOSS OF #12A OR 12B TRANSFORMER	8	91/06/17
AP-ELEC.2	SAFEGUARD BUSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	5	90/02/23



ABNORMAL PROCEDURES INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	8	91/05/10
AP-IA.1	LOSS OF INSTRUMENT AIR	6	92/03/19
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	5	91/11/01
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	4	90/02/23
AP-RCC.2	RCC/RPI MALFUNCTION	4	90/04/09
AP-RCP.1	RCP SEAL MALFUNCTION	6	89/11/17
AP-RCS.1	REACTOR COOLANT LEAK	7	91/11/01
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	5	90/02/23
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	5	90/11/20





ABNORMAL PROCEDURES INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
AP-RHR.1	LOSS OF RHR	8	90/06/01
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	4	92/03/27
AP-SW.1	SERVICE WATER LEAK	7	90/02/23
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	4	90/02/23
AP-TURB.2	AUTOMATIC TURBINE RUNBACK	10	91/10/11
AP-TURB.3	TURBINE VIBRATIONS	5	90/02/23
AP-TURB.4	LOSS OF CONDENSER VACUUM	6	90/02/23



23

EMERGENCY PROCEDURES INDEX

ISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
E-0	REACTOR TRIP OR SAFETY INJECTION	16	91/05/03
E-1	LOSS OF REACTOR OR SECONDARY COOLANT	10	91/05/03
E-2	FAULTED STEAM GENERATOR ISOLATION	4	90/04/09
E-3	STEAM GENERATOR TUBE RUPTURE	12	91/07/26



EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ECA-0.0	LOSS OF ALL AC POWER	12	91/08/05
ECA-0.1	LOSS OF ALL AC POWER RECOVERY WITHOUT SI REQUIRED	5	91/05/03
ECA-0.2	LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED	6	90/04/09
ECA-1.1	LOSS OF EMERGENCY COOLANT RECIRCULATION	6	91/05/03
ECA-1.2	LOCA OUTSIDE CONTAINMENT	2	90/04/09
ECA-2.1	UNCONTROLLED DEPRESSURIZATION OF ALL STEAM GENERATORS	6	91/05/03
ECA-3.1	SGTR WITH LOSS OF REACTOR COOLANT-SUBCOOLED RECOVERY DESIRED	7	91/05/03
ECA-3.2	SGTR WITH LOSS OF REACTOR COOLANT-SATURATED RECOVERY DESIRED	9	91/05/03



EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ECA-3.3	SGTR WITHOUT PRESSURIZER PRESSURE CONTROL	6	91/05/03





EQUIPMENT SUB-PROCEDURES INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ES-0.0	REDIAGNOSIS	7	90/04/09
ES-0.1	REACTOR TRIP RESPONSE	7	90/12/19
ES-0.2	NATURAL CIRCULATION COOLDOWN	2	91/05/03
ES-0.3	NATURAL CIRCULATION COOLDOWN WITH STEAM VOID IN VESSEL (WITH RVLIS)	1	90/04/30
ES-1.1	SI TERMINATION	7	91/05/03
ES-1.2	POST LOCA COOLDOWN AND DEPRESSURIZATION	8	91/06/24
ES-1.3	TRANSFER TO COLD LEG RECIRCULATION	13	91/10/28
ES-3.1	POST-SGTR COOLDOWN USING BACKFILL	4	91/05/03
ES-3.2	POST-SGTR COOLDOWN USING BLOWDOWN	5	91/05/03

8-1



EQUIPMENT SUB-PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE

TITLE

REV

EFFECTIVE  
DATE

ES-3.3

POST-SGTR COOLDOWN USING  
STEAM DUMP

5

91/05/03



CRITICAL SAFETY FUNCTION STATUS TREES INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
F-0.1	SUBCRITICALITY CSFST	1	89/07/21
F-0.2	CORE COOLING CSFST	3	89/07/21
F-0.3	HEAT SINK CSFST	2	90/01/12
F-0.4	INTEGRITY CSFST	1	89/07/21
F-0.5	CONTAINMENT CSFST	2	90/01/12
F-0.6	INVENTORY CSFST	3	90/01/12



FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
FR-C.1	RESPONSE TO INADEQUATE CORE COOLING	6	90/04/09
FR-C.2	RESPONSE TO DEGRADED CORE COOLING	5	90/04/09
FR-C.3	RESPONSE TO SATURATED CORE COOLING	4	90/04/09
FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	10	91/06/14
FR-H.2	RESPONSE TO STEAM GENERATOR OVERPRESSURE	2	90/04/09
FR-H.3	RESPONSE TO STEAM GENERATOR HIGH LEVEL	2	90/04/09
FR-H.4	RESPONSE TO LOSS OF NORMAL STEAM RELEASE CAPABILITIES	1	90/04/09
FR-H.5	RESPONSE TO STEAM GENERATOR LOW LEVEL	2	90/04/09
FR-I.1	RESPONSE TO HIGH PRESSURIZER		





FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

ISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
	LEVEL	3	91/05/03
FR-I.2	RESPONSE TO LOW PRESSURIZER LEVEL	4	91/05/03
FR-I.3	RESPONSE TO VOIDS IN REACTOR VESSEL	5	91/05/03
FR-P.1	RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK CONDITION	6	91/05/03
FR-P.2	RESPONSE TO ANTICIPATED PRESSURIZED THERMAL SHOCK CONDITION	2	90/04/09
FR-S.1	RESPONSE TO NUCLEAR POWER GENERATION/ATWS	4	90/04/09
FR-S.2	RESPONSE TO LOSS OF CORE SHUTDOWN	2	90/04/09
FR-Z.1	RESPONSE TO HIGH CONTAINMENT PRESSURE	2	90/04/09
FR-Z.2	RESPONSE TO CONTAINMENT FLOODING	2	90/04/09



FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE

TITLE

REV

EFFECTIVE  
DATE

FR-Z.3

RESPONSE TO HIGH CONTAINMENT  
RADIATION LEVEL

2

90/04/09



EOP/AP ATTACHMENT INDEX

23

ISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ATT	AUX BLDG SW	0	90/04/09
ATT	CI/CVI	1	90/04/09
ATT	CNMT RECIRC FANS	1	90/04/30
ATT	COND TO S/G	1	90/04/09
ATT	DC LOADS	2	91/05/10
ATT	D/G STOP	1	90/04/09
ATT	EXCESS L/D	1	90/04/30
ATT	FAULTED S/G	2	90/09/17
ATT	GEN DEGAS	3	91/05/10
ATT	LETDOWN	3	91/01/17



EOP/AP ATTACHMENT INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ATT	N2 PORVS	1	90/04/09
ATT	NC	1	90/04/30
ATT	NONVITAL	1	90/04/09
ATT	RCP START	1	90/04/09
ATT	RCS ISOLATION	0	90/04/09
ATT	RHR COOL	1	90/04/09
ATT	RHR NPSH	0	91/05/10
ATT	RHR SYSTEM	1	90/04/30
ATT	RUPTURED S/G	3	91/09/05
ATT	SAFW	2	91/10/11





EOP/AP ATTACHMENT INDEXISSUED: 03/28/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ATT	SD-1	2	90/06/27
ATT	SD-2	2	90/06/27
ATT	SEAL COOLING	1	90/04/09
ATT	SFP-RWST	1	90/04/30
ATT	SI FLUSH	2	91/09/05
ATT	SI/UV	1	90/04/09
ATT	VENT TIME	1	90/04/30
ATT	NORMAL RHR COOLING	0	92/03/27



EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ECA-0.0	LOSS OF ALL AC POWER	12	91/08/05
ECA-0.1	LOSS OF ALL AC POWER RECOVERY WITHOUT SI REQUIRED	5	91/05/03
ECA-0.2	LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED	6	90/04/09
ECA-1.1	LOSS OF EMERGENCY COOLANT RECIRCULATION	6	91/05/03
ECA-1.2	LOCA OUTSIDE CONTAINMENT	2	90/04/09
ECA-2.1	UNCONTROLLED DEPRESSURIZATION OF ALL STEAM GENERATORS	6	91/05/03
ECA-3.1	SGTR WITH LOSS OF REACTOR COOLANT-SUBCOOLED RECOVERY DESIRED	7	91/05/03
ECA-3.2	SGTR WITH LOSS OF REACTOR COOLANT-SATURATED RECOVERY DESIRED	9	91/05/03



EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 01/08/92

PROCEDURE

TITLE

REV

EFFECTIVE  
DATE

ECA-3.3

SGTR WITHOUT PRESSURIZER  
PRESSURE CONTROL

6

91/05/03



CRITICAL SAFETY FUNCTION STATUS TREES INDEX

ISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
F-0.1	SUBCRITICALITY CSFST	1	89/07/21
F-0.2	CORE COOLING CSFST	3	89/07/21
F-0.3	HEAT SINK CSFST	2	90/01/12
F-0.4	INTEGRITY CSFST	1	89/07/21
F-0.5	CONTAINMENT CSFST	2	90/01/12
F-0.6	INVENTORY CSFST	3	90/01/12





FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

ISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
FR-C.1	RESPONSE TO INADEQUATE CORE COOLING	6	90/04/09
FR-C.2	RESPONSE TO DEGRADED CORE COOLING	5	90/04/09
FR-C.3	RESPONSE TO SATURATED CORE COOLING	4	90/04/09
FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	10	91/06/14
FR-H.2	RESPONSE TO STEAM GENERATOR OVERPRESSURE	2	90/04/09
FR-H.3	RESPONSE TO STEAM GENERATOR HIGH LEVEL	2	90/04/09
FR-H.4	RESPONSE TO LOSS OF NORMAL STEAM RELEASE CAPABILITIES	1	90/04/09
FR-H.5	RESPONSE TO STEAM GENERATOR LOW LEVEL	2	90/04/09
FR-I.1	RESPONSE TO HIGH PRESSURIZER		



FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEXISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
	LEVEL	3	91/05/03
FR-I.2	RESPONSE TO LOW PRESSURIZER LEVEL	4	91/05/03
FR-I.3	RESPONSE TO VOIDS IN REACTOR VESSEL	5	91/05/03
FR-P.1	RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK CONDITION	6	91/05/03
FR-P.2	RESPONSE TO ANTICIPATED PRESSURIZED THERMAL SHOCK CONDITION	2	90/04/09
FR-S.1	RESPONSE TO NUCLEAR POWER GENERATION/ATWS	4	90/04/09
FR-S.2	RESPONSE TO LOSS OF CORE SHUTDOWN	2	90/04/09
FR-Z.1	RESPONSE TO HIGH CONTAINMENT PRESSURE	2	90/04/09
FR-Z.2	RESPONSE TO CONTAINMENT FLOODING	2	90/04/09



FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

ISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
FR-Z.3	RESPONSE TO HIGH CONTAINMENT RADIATION LEVEL	2	90/04/09



EOP/AP ATTACHMENT INDEX

ISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ATT	AUX BLDG SW	0	90/04/09
ATT	CI/CVI	1	90/04/09
ATT	CNMT RECIRC FANS	1	90/04/30
ATT	COND TO S/G	1	90/04/09
ATT	DC LOADS	2	91/05/10
ATT	D/G STOP	1	90/04/09
ATT	EXCESS L/D	1	90/04/30
ATT	FAULTED S/G	2	90/09/17
ATT	GEN DEGAS	3	91/05/10
ATT	LETDOWN	3	91/01/17





EOP/AP ATTACHMENT INDEXISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ATT	N2 PORVS	1	90/04/09
ATT	NC	1	90/04/30
ATT	NONVITAL	1	90/04/09
ATT	RCP START	1	90/04/09
ATT	RCS ISOLATION	0	90/04/09
ATT	RHR COOL	1	90/04/09
ATT	RHR NPSH	0	91/05/10
ATT	RHR SYSTEM	1	90/04/30
ATT	RUPTURED S/G	3	91/09/05
ATT	SAFW	2	91/10/11



EOP/AP ATTACHMENT INDEX

ISSUED: 01/08/92

<u>PROCEDURE</u>	<u>TITLE</u>	<u>REV</u>	<u>EFFECTIVE DATE</u>
ATT	SD-1	2	90/06/27
ATT	SD-2	2	90/06/27
ATT	SEAL COOLING	1	90/04/09
ATT	SFP-RWST	1	90/04/30
ATT	SI FLUSH	2	91/09/05
ATT	SI/UV	1	90/04/09
ATT	VENT TIME	1	90/04/30



EOP:  ATT	TITLE:  EOP/AP ATTACHMENTS	REV: 1  PAGE 1 of 2
-----------------	----------------------------------	---------------------------

ATTACHMENT SD-1Supt. Joseph Widay Date 4-4-90

Perform the following local actions to complete normal secondary system shutdown:

- o Close Reheater 4th pass temperature control valves:
  - o V-2432 (SW corner 1A MSR)
  - o V-2433 (SW corner 1B MSR)
  - o V-2434 (SW corner 2A MSR)
  - o V-2435 (SW corner 2B MSR)
- o Close reheater steam chain valves:
  - o V-3550
  - o V-3551
  - o V-3552
  - o V-3553
- o Open Reheater steamline vents (SW corner of condenser, middle floor)
  - o V-8500
  - o V-8501
  - o V-8502
  - o V-8504
  - o V-8505
- o Close flange heating isolation valves:
  - o MOV-3601A
  - o MOV-3602A
- o Open the following valves to align for condensate feed system cooldown RECIRC:
  - o V-3982B (at #5 heater outlet header)
  - o V-3983B (at #5 heater outlet header)
  - o V-4363 (at #5 heater outlet header)
  - o V-4365 (by MFW regulating valves)
  - o V-4361 (southwest corner of condenser, middle floor)
  - o V-3976A A MFP discharge valve bypass valve
  - o V-3977 B MFP discharge valve bypass valve
- o Secure all 5 secondary chemical addition pumps on TURB BLDG middle floor by #5 heaters.
- o Secure all 3 ammonia pumps, TUB BLDG basement by MCC A.
- o Isolate SW from the following coolers:
  - o Main Feed Pump Oil Coolers:
    - o V-4703
    - o V-4704
  - o Exciter Air Cooler:
    - o V-4679 (chain valve next to condensate transfer pump)
  - o Bus Duct Air Cooler
    - o V-4674
    - o V-4674C (mini bypass around V-4674)



EOP: ATT	TITLE: EOP/AP ATTACHMENTS	REV: 1 PAGE 2 of 2
-------------	------------------------------	-----------------------

ATTACHMENT SD-1 (Cont'd)

- o Throttle SW as necessary from following coolers:
  - o Generator Seal Oil Unit Coolers (H2 side and air side):
    - o V-4676A (mini bypass disch valve inside seal oil enclosure Bldg. NW corner)
    - o V-4677A (mini bypass disch valve inside seal oil enclosure Bldg. NW corner)
  - o Main Lube Oil Coolers (SW corner of Turb Oil Reservoir)
    - o V-4691
    - o V-4692
- o Notify Control Room when turbine shaft stops. Control Room personnel will determine if adequate power available to start turning gear.
- o Transfer house heating steam to house heating boiler if necessary (refer to T-35H, NUCLEAR HOUSE HEATING STEAM TO BOILER STEAM SUPPLY CHANGE).
- o Perform T-14G, STEAM GENERATOR BLOWDOWN HEAT RECOVERY SYSTEM SHUTDOWN.
- o Restore MAKEUP to CSTs as directed by Control Room.





EOP:	TITLE:	REV: 1
ATT	EOP/AP ATTACHMENTS	PAGE 1 of 1

ATTACHMENT LETDOWN

Supt. Joseph A. Widay Date 1-4-90

- A) The following conditions must be met to place either normal or excess letdown in service:
- o IA to CNMT - ESTABLISHED
  - o CCW - IN SERVICE
  - o PRZR level - GREATER THAN 13%.
- B) Establish Normal Letdown:
1. Establish charging line flow to REGEN Hx - GREATER THAN 20 gpm.
  2. Place the following switches to CLOSE:
    - o Letdown orifice valves (AOV-200A, AOV-200B, and AOV-202)
    - o AOV-427, loop B cold leg to REGEN Hx
  3. Place letdown controllers TCV-130 and PCV-135 in MANUAL at 25% open.
  4. Verify AOV-371, letdown isolation valve - OPEN.
  5. Open loop B cold leg to REGEN Hx, AOV-427.
  6. Open one 40 gpm letdown orifice valve (AOV-200A or AOV-200B).
  7. Place TCV-130 in AUTO at 105°F.
  8. Place PCV-135 in AUTO at 250 psig.
  9. Adjust charging pump speed and HCV-142 as necessary.
- C) IF normal letdown can NOT be established, THEN establish excess letdown:
1. Place AOV-312 to NORMAL.
  2. Ensure CCW from excess letdown Hx, (AOV-745) - OPEN
  3. Open excess letdown isolation valve AOV-310.
  4. Slowly open HCV-123 to maintain excess letdown temperature less than 195°F and pressure less than 100 psig.
  5. Adjust charging pump speed as necessary.



EOP: ATT	TITLE: ATTACHMENTS	REV: 1 PAGE 22 of 32
-------------	-----------------------	-------------------------

ATTACHMENT RUPTURED S/G (CONT)

\*\*\*\*\*

CAUTION

PART B OF THIS ATTACHMENT SHOULD ONLY BE PERFORMED IF RUPTURED S/G MSIV CANNOT BE CLOSED.

\*\*\*\*\*

PART B. Dispatch AO to locally perform the following when a ruptured S/G MSIV cannot be closed, if areas are accessible:

- 1) Close Air Ejector/Gland steam root valve, V-3540 (Main steam header TURB BLDG).
- 2) Close flange heating isolation valves, MOV-3601A and MOV-3602A.
- 3) Verify MSIV bypass valves shut.
- 4) Notify Control Room that main flowpaths are isolated, THEN complete isolation by closing the following valves:
  - o Support heating steam valves, V-3669 and V-3668, (INT BLDG steam header area)
  - o Steam to sampling system valves, V-3413A and V-3412A, (INT BLDG steam header area)
  - o Upstream traps isolation valves, V-3520 and V-3521, (TURB BLDG east of MFW regulating valves)
  - o MFW regulating valve and bypass valve manual isolation valves for both S/Gs:
    - S/G A, V-3985 and V-3989
    - S/G B, V-3984 and V-3988
  - o Steam to trap header isolation valves
    - o V-8513 (Main steam header TURB BLDG)
    - o V-8529 (south side EH skid)
    - o V-8510 (south side EH skid)
  - o Steam dump header isolation and bypass valves (Main steam header TURB BLDG on platform overhead)
    - o V-3532 and V-3658
    - o V-3533 and V-3659
  - o Reheat steam chain valves and reheat steamline warmup valves (warmup vlvs located east end of 1A and 2A MSRs TURB BLDG middle floor)
    - o 1A MSR, V-3551 and V-3645
    - o 1B MSR, V-3550 and V-3646
    - o 2A MSR, V-3553 and V-3647
    - o 2B MSR, V-3552 and V-3648

