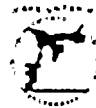


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ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

July 9, 1990

 TELEPHONE
 AREA CODE 716 546-2700

U.S. Nuclear Regulatory Commission
 Document Control Desk
 Attn: Allen R. Johnson
 Project Directorate I-3
 Washington, D.C. 20555

Subject: Response to Generic Letter 90-04
 Status of Generic Safety Issues
 R.E. Ginna Nuclear Power Plant
 Docket No. 50-244

Dear Mr. Johnson:

In Generic Letter 90-04 from James G. Partlow, dated April 25, 1990, you requested information regarding the status of generic safety issues. The attachment to the April 25 letter has been retyped, filled in to show status, and is enclosed with this letter.

Very truly yours,

Robert C. McCreedy
 Division Manager
 Nuclear Production

JRJ/110
 Attachment

xc: Mr. Allen R. Johnson (Mail Stop 14D1)
 Project Directorate I-3
 Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
 Region I
 475 Allendale Road
 King of Prussia, PA 19406

Ginna Senior Resident Inspector

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10-10-10

FACILITY NAME: Ginna Station
 DOCKET NO: 50-244
 LICENSEE: Rochester Gas and Electric Corporation

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	N/A	
43 (B107)	Reliability Of Air Systems	All Plants	C	RG&E responses to NRC Generic Letter 88-14, dated February 3, 1989, and July 9, 1990.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	RG&E response to NRC Generic Letter 88-17, dated January 29, 1990. Work to be completed by end of 91 Refueling outage.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I	RG&E Responding to February 20, 1990 NRC Request. Expected Submittal in July 1990. Completion subject to NRC review.
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	SER by NRC October 25, 1985
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	I	RG&E Letter to NRC dated November 4, 1983



10-10-10

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	Part 1 C Part 2 C	NRC Letter dated August 8, 1986 NRC SER sent in letter dated February 6, 1989
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NRC SER dated November 18, 1987
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	I	RG&E letter to NRC dated November 4, 1983. Generic Letter 90-03 dated May 20, 1990. Supplement 1 to Generic Letter 90-03 dated May 14, 1990. RG&E response due September 25, 1990.
75 (B078)	Items 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C	NRC SER dated April 10, 1986
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NRC SER dated October 1, 1985
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	NRC SER dated April 10, 1986
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NRC SER dated October 1, 1985
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	NRC SER dated April 10, 1986

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	NRC SER dated May 14, 1986
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	NRC SER dated January 9, 1987
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	C	Tech Spec Approved April 4, 1989
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	N/A	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	NRC SER dated April 10, 1986
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	4.5.2 C 4.5.3 I	NRC SER dated February 9, 1987 NRC Letter dated July 26, 1989. Response scheduled for October 1, 1990
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	N/A	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	RG&E Letter dated May 24, 1988 from Bruce Snow to the NRC

GSI/(MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
99(L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C	NRC SER for SEP Topic V-11.B, 9/29/81. RG&E letter 4/17/90.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	Tech Spec amendment 37 dated May 30, 1989
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	Tech Spec amendment 37 dated May 30, 1989
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C	NRC SER with Technical Specification amendment 38 to provisional license dated March 26, 1981. NRC SER of SEP Topic VII-3 dated April 2, 1981. License amendment application dated 7/2/90 provides further enhancement.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	N/A	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	Tech. Spec. amendment 40 by order of NRC, April 20, 1981

