



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

JOHN E. MAIER
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May 25, 1982

Director of Nuclear Reactor Regulation
Attention: Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Supplemental Information
Steam Generator Evaluation
Steam Generator Tube Rupture Incident
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Crutchfield:

Our Steam Generator Evaluation, submitted on April 26, 1982, provided an evaluation of the Ginna steam generator including the results of inspections, a description of analysis and tests used to establish the tube failure mechanism, and a summary of the repairs. As discussed in the report, additional confirmatory testing was still in progress at the time the report was prepared. Some results were presented to the NRC Staff during an April 30, 1982 meeting. The additional testing has been completed and enclosed reports present the results. The reports are:

1. Addendum 1, Laboratory Collapse Tests
These tests were discussed in Sections 4.7.1 and 7.2.1 of our April 26, 1982 submitted.
2. Addendum 2, Laboratory Fatigue Testing
These tests were discussed in Sections 4.7.2 and 7.2.1. of our April 26, 1982 submittal.
3. Addendum 3, Phase II Metallurgical Examinations
These tests were discussed in Section 4.4 of our April 26, 1982 submittal.
4. Addendum 4, External Tube Loading Test
This test was discussed in Section 7.2.2 and in Appendix C of our April 26, 1982 submittal.

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TO Mr. Dennis M. Crutchfield

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The results of the testing do not alter the conclusions presented in our April 26, 1982 report and, therefore, should not alter the conclusions presented in NUREG-0916.

Ten copies of this letter and the enclosed reports are provided.

Very truly yours,


John E. Maier

Enclosures

