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 STN-50-530 Palo Verde Nuclear Station, Unit 3, Arizona Publi 05000530

AUTH.NAME AUTHOR AFFILIATION  
 CONWAY,W.F. Arizona Public Service Co. (formerly Arizona Nuclear Power  
 RECIP.NAME RECIPIENT AFFILIATION  
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SUBJECT: Forwards addl info re Generic Ltr 89-06,Revising  
 Implementation schedule.Informing that Safety Parameters  
 Display Sys 921031 date be revised.

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 TITLE: Safety Parameter Display System (Item 1.D.2) Generic Ltr 89-06 Respon

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MAY



# Arizona Public Service Company

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WILLIAM F. CONWAY  
EXECUTIVE VICE PRESIDENT  
NUCLEAR

161-03846-WFC/MEP/KLMC

March 29, 1991

Docket Nos. STN 50-528/529/530

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Mail Station P1-37  
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- References: 1) Letter to the NRC from W. F. Conway, APS, dated August 30, 1989. Subject: Response to Generic Letter 89-06 (161-02242).
- 2) Letter to the NRC from W. F. Conway, APS, dated December 28, 1989. Subject: Generic Letter 89-06 - Implementation Schedule (161-02726).
- 3) Letter from the NRC to W. F. Conway, APS, dated April 25, 1990. Subject: Response to NRC Generic Letter 89-06 on the Safety Parameter Display System for Palo Verde Nuclear Generating Station.

Dear Sirs:

Subject: Palo Verde Nuclear Generating Station (PVNGS)  
Units 1, 2 and 3  
Generic Letter 89-06 - Revised Implementation Schedule  
File: 91-010-026; 91-056-026

Generic Letter 89-06, Task Action Plan Item I.D.2 - Safety Parameter Display System (SPDS), dated April 12, 1989, requested the implementation status and the certification of the PVNGS SPDS with respect to the requirements of NUREG-0737, Supplement 1, taking into account the information provided in NUREG-1342. Reference 1 provided the APS certification of the PVNGS SPDS. Reference 2 provided the descriptions and implementation schedules for the items to be completed for Generic Letter 89-06.

As documented in Reference 3, the NRC concluded that PVNGS will satisfactorily meet all requirements for an SPDS, provided that commitments for implementation are completed on schedule. Reference 3 also requested notification if the proposed date of SPDS full implementation, of October 31, 1992, is revised. This letter is provided to inform the NRC that the October 31, 1992, commitment date will be revised as follows:

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U. S. Nuclear Regulatory Commission  
Generic Letter 89-06  
Page 2

- Unit 1      Prior to startup from Unit 1 fourth refueling outage -  
                 currently scheduled in November 1993
- Unit 2      Prior to startup from Unit 2 fourth refueling outage -  
                 currently scheduled in April 1993
- Unit 3      Prior to startup from Unit 3 fourth refueling outage -  
                 currently scheduled in April 1994

In addition, the completion dates of Items A.11, A.12, A.13, A.14, and A.15, from Reference 2, will be changed from March 31, 1991, to the completion dates stated above. APS has determined that there is no significant impact to the SPDS due to the schedule revisions. All the remaining actions contained in Items A.1 through A.10, from Reference 2, with the completion date of March 31, 1991, have been completed. Details of the schedule revisions are provided in the attachment to this letter.

APS will notify the NRC of any additional revisions necessary to the Generic Letter 89-06 implementation schedule and also upon full implementation of the SPDS at PVNGS.

If you have any questions concerning this issue, contact Michael E. Powell at (602) 340-4981.

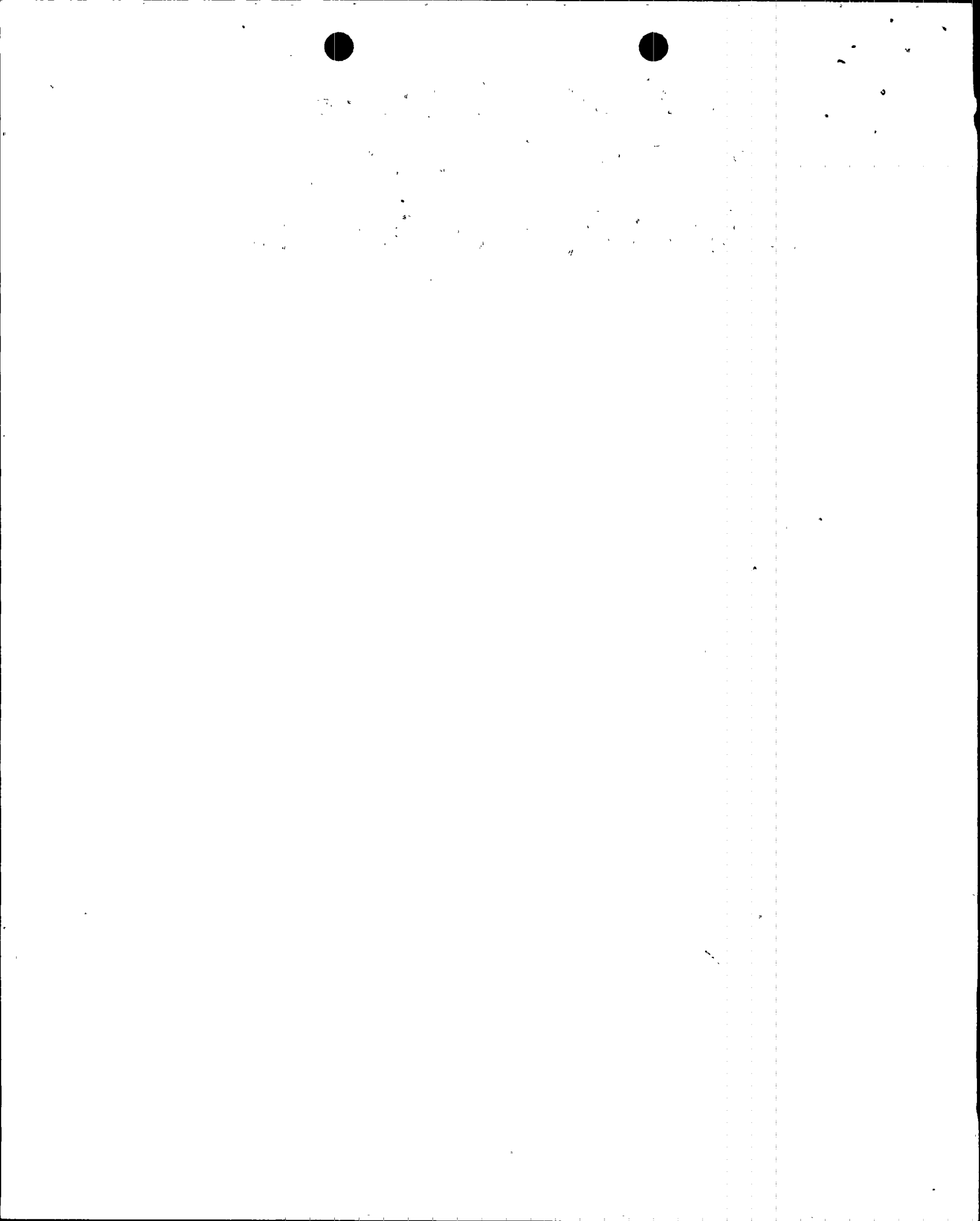
Sincerely,



WFC/MEP/KLMC

Attachment

cc: J. B. Martin  
D. H. Coe  
A. C. Gehr  
A. H. Gutterman



ATTACHMENT

Generic Letter 89-06  
Implementation Schedule Revision

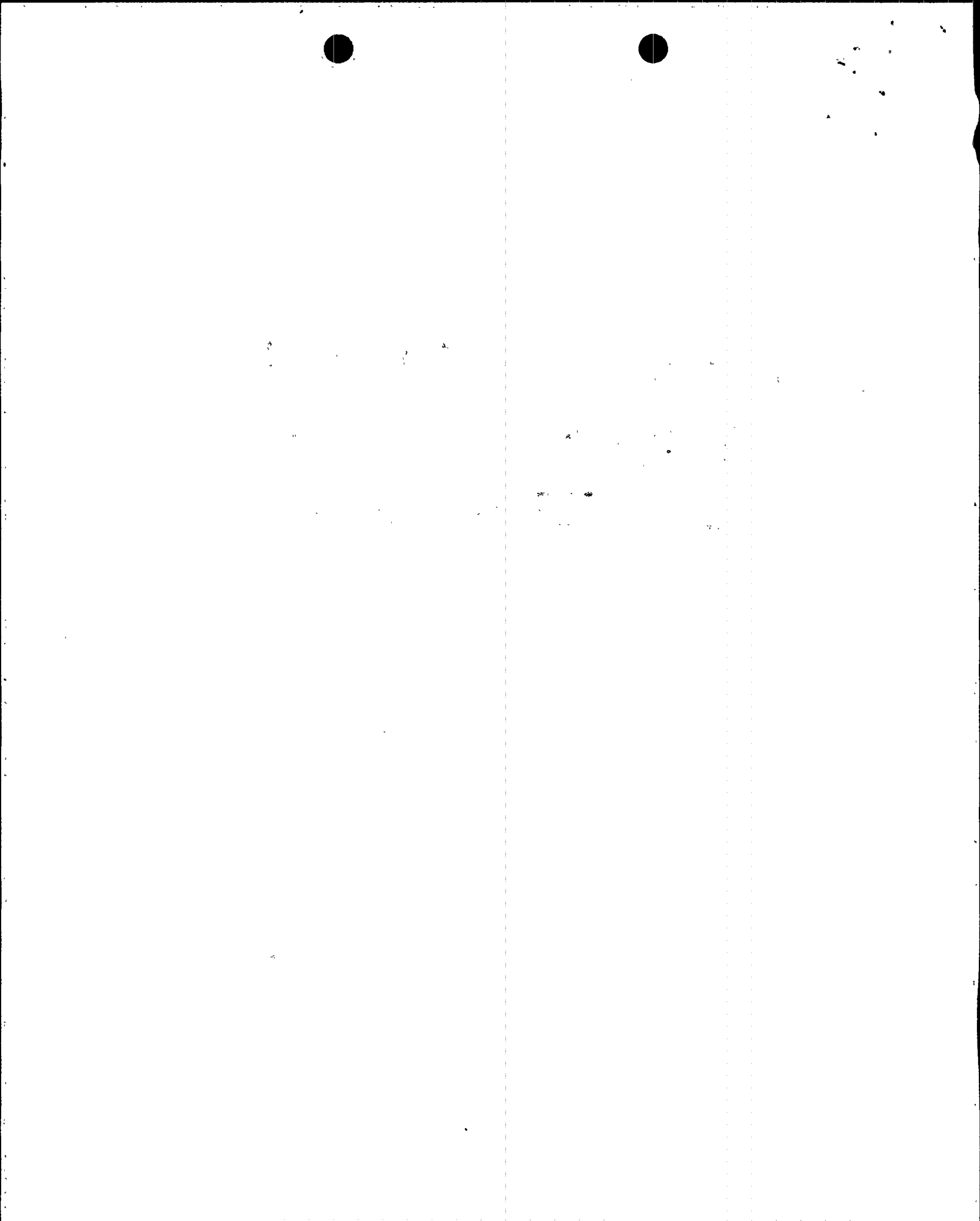
The Safety Parameter Display System (SPDS) implementation schedule revision is necessary because the full implementation of the SPDS will require a major Emergency Response Facilities Data Acquisition (ERFDADS) upgrade. Initially, only minor modifications were anticipated to be required to fulfill the SPDS commitments. However, as the effort to implement the long-term items (from Reference 2) progressed, APS realized that the SPDS changes could not be implemented without performing significant hardware modifications. The current system has insufficient capacity for any additional load, nor will it allow for any improvement in response time. Once the necessity of a hardware upgrade was realized, additional studies were performed to determine what other enhancements would result in the most effective system for plant operators.

These evaluations also indicate that a total system replacement would be more cost effective than attempting to implement the enhancements to the current system. To upgrade the current system there is a large effort associated with a software upgrade of dated technology and a capital investment in dated hardware. Also, a total system replacement would produce a more valuable tool for plant operations because of better interface with other plant systems and improved man-machine interface.

Items A.11, A.12, A.13, and A.14, from Reference 2, are being changed from implementation actions to evaluation actions because changes planned for the SPDS full implementation may make these changes inappropriate or otherwise incompatible with the redesign of the SPDS. Item A.15 also involves changes to the SPDS displays which are being redesigned as part of the SPDS full implementation.

APS considers that short-term items A.11, A.12, A.13, A.14, A.15, and the long-term items, are enhancements to the current system. The current SPDS is acceptable for use by the operators for accident evaluation. Therefore, there is no adverse effect to the SPDS due to the schedule revision. The short-term items from Reference 2 are as follows:

- Item A.11    Modify the deviation bar alarm propagation logic to achieve consistency between the alarm states of the top level bars and the corresponding SPDS CSF boxes.
- Item A.12    Add numerical values and associated engineering units to the second level deviation bar displays.
- Item A.13    Add Shutdown Cooling (RHR flow) to the SPDS Heat Removal display.
- Item A.14    Add Containment Radiation Level to the SPDS Indirect Radiation Release display.





ATTACHMENT  
(continued)

Generic Letter 89-06  
Implementation Schedule Revision

- Item A.15    Add the Critical Safety Function (CSF) overlay to the plant status displays commonly used in the control room so that these displays can be viewed while still meeting the continuous display requirement. In addition, administratively limit the Control Room SPDS terminal to displays containing CSF boxes. In addition, re-install the SPDS audible alarm in the designated user's SPDS workstation.

The ERFDADS replacement requires considerable design, development and testing time. In addition, the installation must be done during a scheduled outage. The schedule for full implementation for each of the Units is prior to startup from the fourth refueling outage.



Docket Nos. 50-528, 50-529  
and 50-530

MAR 21 1991

Mr. William F. Conway  
Executive Vice President, Nuclear  
Arizona Public Service Company  
P. O. Box 53999  
Phoenix, Arizona 85072-3999

Dear Mr. Conway:

SUBJECT: OPERATIONAL EVENTS WHILE SHUTDOWN

The NRC has just issued Information Notice 91-22, "Four Plant Outage Events Involving Loss of AC Power or Coolant Spills," which addresses recent events that occurred during shutdown operations.

The chief purpose of this information notice is to notify each licensee that the high rate of precursor events to loss of decay heat removal during shutdown is a source of concern to the NRC. All of the events discussed in this information notice occurred during a one-week period in March 1991. Because of the potential for loss of a critical safety function in these and similar events, I believe a high level of management attention is required in the planning, coordination, and execution of shutdown operations.

While this information notice does not require specific licensee action or response, I urge you to give this important matter your personal attention.

Sincerely,  
Original signed by  
Thomas E. Murley

Thomas E. Murley, Director  
Office of Nuclear Reactor Regulation

cc: See next page

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3. The Commission has also been informed that the Government of India has been requested to provide information on the progress of the implementation of the recommendations of the Commission's report on the subject.

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1. The first step in the process is to identify the problem or issue that needs to be addressed. This involves gathering information and understanding the context of the problem.

2. Once the problem is identified, the next step is to define the objectives and goals of the project. This helps to clarify what needs to be achieved and provides a clear direction for the team.

3. The third step is to develop a plan or strategy to address the problem. This involves breaking down the problem into smaller, manageable tasks and determining the resources needed to complete each task.

4. The fourth step is to implement the plan. This involves putting the strategy into action and monitoring progress to ensure that the project is on track.

5. The final step is to evaluate the results of the project. This involves assessing the outcomes against the objectives and goals and identifying any areas for improvement.

[illegible]

Mr. William F. Conway  
Arizona Public Service Company

Palo Verde

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