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 RECIP. NAME RECIPIENT AFFILIATION
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SUBJECT: Forwards revised pages of reload analysis rept, correcting
 inaccurate info re upper end fitting hold down plate design.
 Proposed design change in ref rept will be implemented after
 Unit 1 Cycle 2 reload.

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September 4, 1987
161-00487-JGH/LJM

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

ATTN: Document Control Desk

Reference: J. G. Haynes letter (161-00321) to DCD (NRC), 6/29/87,
Subject: Submittal of the Reload Analysis Report for
Unit 1 Cycle 2

Dear Sir:

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Unit 1
Docket No. STN 50-528 (License NPF-41)
Submittal of the Reload Analysis Report
for Unit 1 Cycle 2 - Revision
File: 87-B-056-026

Attached please find revised pages of the Reload Analysis Report (RAR), which contained inaccurate information regarding the upper end fitting hold down plate design. The proposed design change described in the referenced Reload Analysis Report will not be implemented until after Unit 1 Cycle 2 reload, at which time all fuel fabricated for PVNGS will be modified as described in the referenced RAR.

If you have any questions, please contact W. F. Quinn at (602) 371-4087.

Very truly yours,

J. G. Haynes
Vice President
Nuclear Production

JGH/LJM/mes

Attachments

cc: O. M. De Michele
E. E. Van Brunt, Jr.
G. W. Knighton
J. B. Martin
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A. C. Gehr
J. R. Ball

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4.0 FUEL SYSTEM DESIGN

4.1 MECHANICAL DESIGN

4.1.1 Fuel Design

With the exception of the design features listed below the mechanical design of the Batch D reload fuel assemblies is identical to the design of the Core 1 Batch C fuel assemblies that were modified to increase the shoulder gap from 1.682 inches to 2.382 inches, Reference 4-1. No changes in mechanical design bases have occurred since the original fuel design. The design features incorporated into Batch D were made to improve the fabricability and quality of the fuel and to improve the burnup capability of the poison rods. The specific changes are discussed in the following paragraphs.

1. Deleted

2. The inspection envelope for the fuel bundle assembly has been changed, from a square of 8.230 inches per side for all of the bundle with exception of the vicinity around the upper most grid where a square of 8.250 inches per side is permissible to a square of 8.290 inches per side for the entire length of the fuel bundle. This change will not affect any of the mechanical design considerations from the standpoint of structural behavior and integrity, fuel assembly bow, handling, interface or other considerations. Furthermore, a reduction in the

12.0 References

12.1 Section 1.0 References

(1-1) "Palo Verde Nuclear Generating Station Unit No. 1, Final Safety Analysis Report," Arizona Public Service Company, Docket No. 50-528.

(1-2) "Combustion Engineering Standard Safety Analysis Report (CESSAR)", Docket #STN-50-470F.

12.2 Section 2.0 References

None

12.3 Section 3.0 References

None

12.4 Section 4.0 References

(4-1) "Palo Verde Unit 1 Fuel Design Report" NPSD-207-P, November 1982.

(4-2) Deleted

(4-3) Letter, A. E. Lundvall, Jr. to J. R. Miller (Chief Operating Reactors Branch #3), "Calvert Cliffs Nuclear Power Plant Unit Nos. 1 and 2, Docket Nos. 50-317 and 50-318, Request for Amendment", December 31, 1984.

(4-4) EPRI NP-3966-CCM, "CEPAN Method of Analyzing Creep Collapse of Oval Cladding Volume 5: Evaluation of Interpellet Gap Formation and Clad Collapse in Modern PWR Fuel Rods," April, 1985.

