



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 5, 2018

Mr. Thomas D. Ray
Vice President
McGuire Nuclear Station
Duke Energy Carolinas, LLC
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: MCGUIRE NUCLEAR STATION, UNITS 1 AND 2 – ISSUANCE OF
AMENDMENTS ADOPTING TECHNICAL SPECIFICATIONS TASK FORCE
TRAVELER TSTF-197-A, REVISION 2 (CAC NOS. MF9110 AND MF9111;
EPID L-2017-LLA-0161)

Dear Mr. Ray:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 305 to Renewed Facility Operating License No. NPF-35 and Amendment No. 284 to Renewed Facility Operating License No. NPF-52 for the McGuire Nuclear Station, Units 1 and 2, respectively. The amendments are in response to your application dated January 11, 2017 supplemented by letter dated June 8, 2017.

The amendments modify TS limiting condition for operation (LCO) 3.9.5, "Residual Heat Removal (RHR) and Coolant Circulation – High Water Level," and TS LCO 3.9.6, "Residual Heat Removal (RHR) and Coolant Circulation – Low Water Level." Condition A of TS LCO 3.9.5 applies when RHR requirements are not met and includes four Required Actions. Required Action A.4 currently requires that within 4 hours, the closure of all containment penetrations providing direct access from containment atmosphere to outside atmosphere. The proposed changes revise Required Action A.4 and add new Required Actions A.5, A.6.1, and A.6.2, to clarify that the intent of the Required Actions is to establish containment closure. Each of these Required Actions will have a completion time (CT) of 4 hours. Condition B of TS LCO 3.9.6 applies when no RHR loop is in operation, and includes three Required Actions. Current Required Action B.3 requires the closure of all containment penetrations providing direct access from containment atmosphere to outside atmosphere. The proposed changes to TS LCO 3.9.6 are the same as the proposed changes to TS LCO 3.9.5, consisting of a revision to Required Action B.3 and the addition of new Required Actions B.4, B.5.1, and B.5.2. The licensee stated that the proposed changes are consistent with Technical Specifications Task Force (TSTF) Traveler TSTF-197-A, Revision 2, "Require containment closure when shutdown cooling requirements are not met."

T. Ray

- 2 -

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read 'Michael Mahoney', written over a horizontal line.

Michael Mahoney, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

Enclosures:

1. Amendment No. 305 to NPF-9
2. Amendment No. 284 to NPF-17
3. Safety Evaluation

cc w/enclosures: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-369

MCGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 305
Renewed License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Renewed Facility Operating License No. NPF-9, filed by Duke Energy Carolinas, LLC (the licensee), dated January 11, 2017, as supplemented by letter dated June 8, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 305, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed License No. NPF-9
and Technical Specifications

Date of Issuance: January 5, 2018



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-370

MCGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 284
Renewed License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Renewed Facility Operating License No. NPF-17, filed by the Duke Energy Carolinas, LLC (the licensee), dated January 11, 2017, as supplemented by letter dated June 8, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

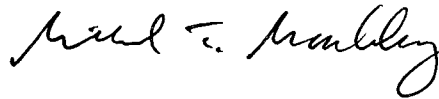
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 284, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed License No. NPF-17
and Technical Specifications

Date of Issuance: January 5, 2018

ATTACHMENT TO
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
LICENSE AMENDMENT NO. 305
RENEWED FACILITY OPERATING LICENSE NO. NPF-9
DOCKET NO. 50-369
AND
LICENSE AMENDMENT NO. 284
RENEWED FACILITY OPERATING LICENSE NO. NPF-17
DOCKET NO. 50-370

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

NPF-9, page 4
NPF-17, page 4

Insert

NPF-9, page 4
NPF-17, page 4

Replace the following pages of the Appendix A Technical Specifications (TS) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

TS 3.9.5-2
TS 3.9.6-2

Insert

TS 3.9.5-2
TS 3.9.6-2

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2, and;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such byproduct material as may be produced by the Duke Training and Technology Center.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The licensee is authorized to operate the facility at a reactor core full steady state power level of 3469 megawatts thermal (100%).
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 305, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than June 12, 2021, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts, 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2; and,
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such by product material as may be produced by the Duke Training and Technology Center.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or thereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The licensee is authorized to operate the facility at a reactor core full steady state power level of 3469 megawatts thermal (100%).
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 284, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than March 3, 2023, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59, and otherwise complies with the requirements in that section.

RHR and Coolant Circulation – High Water Level
3.9.5

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.4 Close the containment equipment hatch and secure with four bolts.	4 hours
	<u>AND</u>	
	A.5 Close one door in each air lock.	4 hours
	<u>AND</u>	
	A.6.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent.	4 hours
	<u>OR</u>	
	A.6.2 Verify each penetration is capable of being closed on a high containment radiation signal.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.5.1 Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of ≥ 1000 gpm and RCS temperature is $\leq 140^{\circ}\text{F}$.	In accordance with the Surveillance Frequency Control Program
SR 3.9.5.2 Verify required RHR loop locations susceptible to gas accumulation are sufficiently filled with water.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No RHR loop in operation.	B.1 Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
	<u>AND</u>	
	B.2 Initiate action to restore one RHR loop to operation.	Immediately
	<u>AND</u>	
	B.3 Close the containment equipment hatch and secure with four bolts.	4 hours
	<u>AND</u>	
	B.4 Close one door in each air lock.	4 hours
	<u>AND</u>	
	B.5.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent.	4 hours
	<u>OR</u>	
	B.5.2 Verify each penetration is capable of being closed on a high containment radiation signal.	4 hours



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 305 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-9

AND

AMENDMENT NO. 284 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-17

DUKE ENERGY CAROLINAS, LLC

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

By letter dated January 11, 2017, as supplemented by letter dated June 8, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML17025A069 and ML17159A688, respectively), Duke Energy Carolinas, LLC (Duke Energy, the licensee) submitted an application to change the technical specifications (TSs) for the McGuire Nuclear Station, Units 1 and 2 (McGuire). The supplement dated June 8, 2017, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC or Commission) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on May 23, 2017 (82 FR 23619).

The amendments modify TS limiting condition for operation (LCO) 3.9.5, "Residual Heat Removal (RHR) and Coolant Circulation – High Water Level," and TS LCO 3.9.6, "Residual Heat Removal (RHR) and Coolant Circulation – Low Water Level." Condition A of TS LCO 3.9.5 applies when RHR requirements are not met and includes four Required Actions. Required Action A.4 currently requires that within 4 hours, the closure of all containment penetrations providing direct access from containment atmosphere to outside atmosphere. The proposed changes revise Required Action A.4 and add new Required Actions A.5, A.6.1, and A.6.2, to clarify that the intent of the Required Actions is to establish containment closure. Each of these Required Actions will have a completion time (CT) of 4 hours. Condition B of TS LCO 3.9.6 applies when no RHR loop is in operation, and includes three Required Actions. Current Required Action B.3 requires the closure of all containment penetrations providing direct access from containment atmosphere to outside atmosphere. The proposed changes to TS LCO 3.9.6 are the same as the proposed changes to TS LCO 3.9.5, consisting of a revision to Required Action B.3 and the addition of new Required Actions B.4, B.5.1, and B.5.2. The licensee stated that the proposed changes are consistent with Technical Specifications Task Force (TSTF)

Traveler TSTF-197-A, Revision 2, "Require containment closure when shutdown cooling requirements are not met," (ADAMS Accession No. ML040560357).

The amendments adopt TSTF-197-A, Revision 2, in order to bring the McGuire TSs into closer alignment with NUREG-1431, Revision 4, "Standard Technical Specifications Westinghouse Plants" (ADAMS Package No. ML12100A222), which contains the Westinghouse Owners Group (WOG) Standard Technical Specifications (STS), hereafter referred to as "WOG STS" or "STS".

2.0 REGULATORY EVALUATION

2.1 Applicable Regulations and Guidance

As described in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36(c)(2)(i) states, "Limiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility."

The regulation in 10 CFR 50.36(c)(2)(i) states, "When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the technical specifications until the condition can be met."

The amendments adopt previously NRC-approved TSTF-197-A, Revision 2, to bring McGuire's TSs into alignment with NUREG-1431, Revision 4, which contains the STS for Westinghouse plants.

2.2 Licensee's Proposed Changes

Current TS LCO 3.9.5, Condition A, "RHR loop requirements not met," Required Action A.4, states:

Close all containment penetrations providing direct access from containment atmosphere to outside atmosphere.

The current CT of TS LCO 3.9.5, Condition A, Required Action A.4 is 4 hours.

Revised TS LCO 3.9.5, Condition A, Required Action A.4, will state:

A.4 Close the containment equipment hatch and secure with four bolts.

The following new Required Actions will be added to TS LCO 3.9.5, Condition A:

AND

A.5 Close one door in each air lock.

AND

A.6.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent.

OR

- A.6.2 Verify each penetration is capable of being closed on a high containment radiation signal.

The revised Required Action A.4 and new Required Actions A.5, A.6.1, and A.6.2, will each have CTs of 4 hours.

Current TS LCO 3.9.6, Condition B, "No RHR loop in operation," Required Action B.3, states:

Close all containment penetrations providing direct access from containment atmosphere to outside atmosphere.

The current CT of TS LCO 3.9.6, Condition B, Required Action A.4 is 4 hours.

Revised TS LCO 3.9.6, Condition B, Required Action B.3, will state:

- B.3 Close the containment equipment hatch and secure with four bolts.

The following new Required Actions will be added to TS LCO 3.9.6, Condition B:

AND

- B.4 Close one door in each air lock.

AND

- B.5.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent.

OR

- B.5.2 Verify each penetration is capable of being closed on a high containment radiation signal.

The revised Required Action B.3 and New Required Actions B.4, B.5.1, and B.5.2, will each have CTs of 4 hours.

3.0 TECHNICAL EVALUATION

3.1 NRC Staff's Technical Evaluation

The NRC staff compared the licensee's proposed changes against NRC-approved TSTF-197-A, Revision 2, and NUREG-1431, Revision 4, which contains the WOG STS.

The licensee is not proposing to change the 4-hour CT for the revised Required Actions in TS LCO 3.9.5 and TS LCO 3.9.6, which is consistent with TSTF-197-A, Revision 2. The proposed new actions will also have 4-hour CTs.

Consistent with TSTF-197, Revision 2, and NUREG-1431, Revision 4, the proposed changes to TS LCO 3.9.5 and TS LCO 3.9.6 replace a Required Action (TS LCO 3.9.5, A.4 and TS LCO 3.9.6, B.3) with a more specific Required Action with the addition of three new Required Actions

(TS LCO 3.9.5, A.5, A.6.1, and A.6.2 and TS LCO 3.9.6, B.4, B.5.1, and B.5.2). The revised and new Required Actions will provide more specific instructions for containment closure compared to the original TS LCO 3.9.5, Required Action A.4 and TS LCO 3.9.6, Required Action B.4. The more specific instructions increase the assurance that the containment will be in an appropriate state and lessening the low safety value burdens on the plant, while retaining confidence that the containment will be placed in an acceptable configuration should an event occur.

3.1.1 Deviations from TSTF-197-A, Revision 2

The licensee stated that the following deviations exist between the proposed changes of the licensee's amendment requests and the NRC-approved TSTF-197-A, Revision 2.

3.1.1.1 Required Action A.4

The TSTF-197-A, Revision 2, Inserts 5 and 6 markups for TS LCO 3.9.5, Required Action A.4 and TS LCO 3.9.6, Required Action B.3, respectively, read: "Close equipment hatch and secure with [four] bolts." The licensee proposes the more precise wording of "Close the containment equipment hatch and secure with four bolts" for TS LCO 3.9.5, Required Action A.4 and TS LCO 3.9.6, Required Action B.3.

The licensee noted that the proposed wording avoids confusion with other types of equipment hatches at McGuire (e.g., divider barrier equipment hatches utilized in the ice condenser containment). The licensee also noted the use of four bolts to secure the containment equipment hatch is consistent with current TS 3.9.4 "Containment Penetrations."

The NRC staff finds that the proposed Required Action wording for TS LCOs 3.9.5 and 3.9.6 is more precise, avoids confusion, and is consistent with the bolting requirements for other containment penetration equipment hatches as contained in TS 3.9.4. Accordingly, this deviation from TSTF-197-A, Revision 2, continues to meet 10 CFR 50.36(c)(2)(i) and is, therefore, acceptable.

3.1.1.2 Required Actions A.6.2 and B.5.2

The TSTF-197-A, Revision 2, Inserts 5 and 6 markups for TS LCO 3.9.5, Required Action A.6.2 and TS LCO 3.9.6, Required Action B.5.2, respectively, read: "Verify each penetration is capable of being closed by an OPERABLE Containment Purge and Exhaust Isolation System." The licensee proposes to use the words "Verify each penetration is capable of being closed on a high containment radiation signal" for TS LCO 3.9.5, Required Action A.6.2 and TS LCO 3.9.6, Required Action B.5.2.

The McGuire units have a Containment Purge Ventilation System and Containment Purge Supply and Exhaust System, as described in Section 9.4.5 of McGuire's Updated Final Safety Analysis Report (UFSAR). Regarding their isolation signals, UFSAR Section 9.4.5 states that the Containment Purge Ventilation System isolates its supply and exhaust paths from the outside atmosphere upon receipt of a high containment radiation signal and the Containment Purge Supply and Exhaust System is isolated on a high radiation signal during all plant Modes. Additionally, UFSAR Section 9.4.5 states that for all penetrations through the containment vessel for both systems, there are two, normally closed, isolation valves at each penetration, that provide containment isolation.

UFSAR Section 6.2.4.1 indicates that containment ventilation isolation occurs automatically on a safety injection (SI) signal or on a high containment "air particle" and/or "radio gas" activity signal. Because the SI systems are not OPERABLE during the mode of applicability for TS 3.9.5 and TS 3.9.6 (MODE 6), the appropriate containment valve closure signal is the high gaseous radioactivity monitor signal.

The operability of the containment high gaseous radioactivity monitor is not addressed in the TSs. The term "OPERABLE" is intended to apply only to TSs. At McGuire, the containment high radiation monitor is addressed in "The McGuire Selected Licensee Commitment [SLC] Manual," Section 16.7.6, "Radiation Monitoring for Plant Operations." Therefore, the functionality of this monitor is maintained in accordance with McGuire's SLC Manual. McGuire's SLC Manual contains commitments to control important plant equipment and operating conditions, not controlled elsewhere, and is equivalent to other utilities Technical Requirements Manual. Changes to McGuire's SLC Manual are controlled by the 10 CFR 50.59 process.

SLC 16.7.6 includes requirements for a collection of radiation monitoring instrumentation used during plant operations, including the High Gaseous Radioactivity Monitor (EMF-39-Low Range). If the channel is nonfunctional, the SLC requires immediate verification that the containment purge exhaust system valves are maintained closed. SLC 16.7.6 include testing requirements (TRs) to ensure the FUNCTIONAL status of "EMF-39-Low Range". TR 16.7.6.2 requires that a Channel Operational Test (COT) be performed every 92 days. The COT demonstrates the functional capabilities of the High Gaseous Radioactivity Monitor. The mode applicability for the SLC 16.7.6 requirements includes Mode 6, which is the applicable mode for TS 3.9.5 and TS 3.9.6.

The NRC staff has confirmed that the relevant containment isolation valve closure signal in Mode 6 is the high containment activity signal associated with the high containment radiation monitors governed by SLC 16.7.6. Therefore, the NRC staff concludes that for TS LCO 3.9.5, Required Action A.6.2 and TS LCO 3.9.6, Required Action B.5.2, the use of the words "Verify each penetration is capable of being closed on a high containment radiation signal," in place of TSTF-197-A, Revision 2, words, "Verify each penetration is capable of being closed by an OPERABLE Containment Purge and Exhaust Isolation System," continues to meet 10 CFR 50.36(c)(2)(i) and is, therefore, acceptable.

3.2 NRC Staff Conclusion

The NRC staff has reviewed the licensee's proposed license amendment requests and finds that the proposed changes to TS LCO 3.9.5, Required Action A.4 (and new Required Actions A.5, A.6.1, and A.6.2), and TS LCO 3.9.6, Required Action B.3 (and new Required Actions B.4, B.5.1, and B.5.2), are consistent with TSTF-197, Revision 2 and NUREG-1431, Revision 4, and will ensure that the containment will be in the appropriate closure state should unplanned events occur.

Accordingly, the NRC staff finds that the requirements of 10 CFR 50.36(c)(2) continue to be met because the minimum performance level of equipment needed for safe operation of the facility is contained in the revised TS LCOs 3.9.5 and 3.9.6. Therefore, the NRC staff concludes that the proposed changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, NRC staff notified the South Carolina State official of the proposed issuance of the amendments on October 23, 2017. The State official confirmed on October 20, 2017, that the State of South Carolina had no comments.

5.0 PUBLIC COMMENTS

On May 23, 2017, the NRC staff published a "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Combined Licenses and Proposed No Significant Hazards Consideration Determination," in the *Federal Register* associated with the proposed amendment request (82 FR 23618). In accordance with the requirements in 10 CFR 50.91, the notice provided a 30-day period for public comment on the proposed no significant hazards consideration determination. One comment from a member of the public was received, however it was not related to the proposed no significant hazards consideration determination or to the proposed amendment request. The comment can be found at www.regulations.gov, reference NRC-2017-0120-0002.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on this finding (82 FR 23618: May 23, 2017). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: S. Darbali, NRR
R. Grover, NRR
D. Nold, NRR

Date of issuance: January 5, 2018

SUBJECT: MCGUIRE NUCLEAR STATION, UNITS 1 AND 2 – ISSUANCE OF
AMENDMENTS ADOPTING TECHNICAL SPECIFICATIONS TASK FORCE
TRAVELER TSTF-197-A, REVISION 2 (CAC NOS. MF9110 AND MF9111;
EPID L-2017-LLA-0161) DATED JANUARY 5, 2018

DISTRIBUTION:

PUBLIC

PM Reading File

RidsACRS_MailCTR Resource

RidsNrrLpl2-1 Resource

RidsNrrDssStsb Resource

RidsNrrDssSbpb Resource

RidsNrrDeEicb Resource

RidsNrrPMMcGuire Resource

RidsRgn2MailCenter Resource

RidsNrrLALKGoldstein Resource

DNold, NRR

ADAMS Accession No.: ML17297A917

*by memorandum

OFFICE	DORL/LPL2-1/PM	DORL/LPL2-1/LA	DSS/SBPB/BC	DE/EICB/BC
NAME	MMahoney	Goldstein (SRohrer for)	RDennig	MWaters
DATE	11/06/17	11/06/17	07/27/17*	11/08/17
OFFICE	DSS/STSB/BC	OGC	DORL/LPL2-1/BC	DORL/LPL2-1/PM
NAME	VCusumano	STurk	MMarkley	MMahoney
DATE	11/30/17	12/18/17	01/05/18	01/05/18

OFFICIAL RECORD COPY