

CATEGORY 1

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 AUTH.NAME AUTHOR AFFILIATION
 BEMIS, P.R. Washington Public Power Supply System
 RECIP.NAME RECIPIENT AFFILIATION
 MERSCHOFF, E.W. Region 4 (Post 820201)

SUBJECT: Notification of unexpected change in feedwater temperature & flow indication used to calculate plant heat balance during routine trending of plant computer feedwater temperatures on 980112. Corrective actions reduced reactor power to 99%.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • Richland, Washington 99352-0968

January 14, 1998
GO2-98-006

Docket No. 50-397

Mr. EW Merschoff
NRC Regional Administrator
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

Dear Mr. Merschoff:

Subject: **WNP-2 OPERATING LICENSE NPF-21
FACILITY OPERATION AT REACTOR CORE POWER LEVELS IN
EXCESS OF FULL POWER**

This is a notification to the NRC Regional Administrator in accordance with Facility Operating License Condition 2.F.

On January 12, 1998, routine trending of the plant computer feedwater temperatures by plant personnel revealed an unexpected change in the feedwater temperature and flow indication used to calculate plant heat balance starting on January 9, 1998. Due to the possibility of exceeding 100 percent rated thermal power (3486 Mwt) because of the feedwater indication error, immediate corrective actions were taken to reduce reactor power 1 percent, to 99 percent rated thermal power as indicated by the plant computer, and control room personnel were directed not to increase reactor power above 99 percent until further notice.

On January 13, 1998, the Supply System confirmed by detailed analysis of plant computer heat balance data that, due to the feedwater indication error, reactor thermal power exceeded Condition 2.C(1) of the Facility Operating License, which limits reactor power to less than or equal to 3486 Mwt. The analysis revealed that the maximum reactor power during the three day period from January 9 through January 12 may have been as high as 100.7 percent rated thermal power.

Although the Facility Operating License Condition 2.C(1) for maximum power level was exceeded, there appears to be no immediate reactor safety issue at this time.

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PDR ADDCK 05000397
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**FACILITY OPERATION AT REACTOR CORE POWER LEVELS IN EXCESS OF
FULL POWER**

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Should you have any questions regarding this matter, please call me or Mr. PJ Inserra at
(509) 377-4147.

Respectfully,



PR Bemis

Vice President, Nuclear Operations

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