

# CATEGORY 1

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ACCESSION NBR: 9607290226      DOC. DATE: 96/07/19      NOTARIZED: YES      DOCKET #  
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Powe      05000397  
 AUTH. NAME      AUTHOR AFFILIATION  
 PARRISH, J.V.      Washington Public Power Supply System  
 RECIP. NAME      RECIPIENT AFFILIATION  
                          Document Control Branch (Document Control Desk)

SUBJECT: Forwards response to RAI re NRC Bulletin 96-002, "Movement  
 of Heavy Loads Over Spent Fuel, Over Fuel in Reactor Core, Or  
 Over Safety-Related Equipment."

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

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July 19, 1996  
GO2-96-141

Docket No. 50-397

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

Subject: **WNP-2, OPERATING LICENSE NPF-21, RESPONSE TO NRC BULLETIN 96-02, "MOVEMENT OF HEAVY LOADS OVER SPENT FUEL, OVER FUEL IN THE REACTOR CORE, OR OVER SAFETY-RELATED EQUIPMENT" - ADDITIONAL INFORMATION**

- References:
- 1) NRC Bulletin 96-02, dated April 11, 1996, "Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment"
  - 2) Letter, GO2-96-100, dated May 10, 1996, JV Parrish (SS) to NRC, "Response to NRC Bulletin 96-02, Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment"

Per the above references, the Supply System has conducted an extensive review of our plans and capabilities for handling heavy loads while the reactor is at power. The review included a walkdown to assess general plant condition in the crane and hoist areas and assess handling system load paths. The review concluded that present activities are in accordance with existing guidelines and our licensing basis, with the exceptions listed in the enclosed attachment.

The review identified procedure enhancements that will be made to ensure consistency and provide commitment clarification. These will be tracked by the Plant Tracking Log (PTL).

In addition, the Supply System will be pursuing future use of a spent fuel dry cask storage facility. As requested by Reference 1, a license amendment request will be submitted 6 to 9 months in advance for planned movement of loads involving the handling of any new

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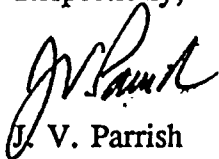
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**RESPONSE TO NRC BULLETIN 96-02**

heavy loads, including dry storage casks, that create a potential load drop accident not previously evaluated in the FSAR, or that require changes to Technical Specifications. If necessary, the capability of performing actions required for safe shutdown in the presence of a source term resulting from a dropped storage cask will also be addressed in the license amendment request.

Should you have any questions or desire additional information regarding this matter, please call me or Ms. L. C. Fernandez (509) 377-4147.

Respectfully,



J. V. Parrish  
Chief Executive Officer  
Mail Drop 1023

REB

Attachment

cc: LJ Callan - NRC RIV  
KE Perkins, Jr. - NRC RIV, WCFO  
NS Reynolds - Winston & Strawn

TG Colburn - NRR  
DL Williams - BPA/399  
NRC Sr. Resident Inspector - 927N

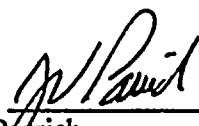


STATE OF WASHINGTON )  
 )  
COUNTY OF BENTON )

Subject: Response to NRC Bulletin 96-02 "Move-  
ment of Heavy Loads Over Spent Fuel,  
Over Fuel in the Reactor Core, or Over  
Safety-Related Equipment" - Additional  
Information

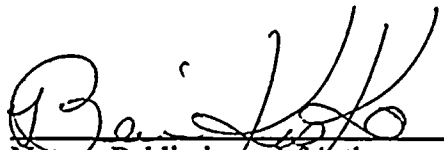
I, J. V. PARRISH, being duly sworn, subscribe to and say that I am the Chief Executive Officer for the WASHINGTON PUBLIC POWER SUPPLY SYSTEM, the applicant herein; that I have the full authority to execute this oath; that I have reviewed the foregoing; and that to the best of my knowledge, information, and belief the statements made in it are true.

DATE 19 July, 1996

  
\_\_\_\_\_  
J. V. Parrish  
Chief Executive Officer

On this date personally appeared before me J. V. PARRISH, to me known to be the individual who executed the foregoing instrument, and acknowledged that he signed the same as his free act and deed for the uses and purposes herein mentioned.

GIVEN under my hand and seal this 19 day of July 1996.

  
\_\_\_\_\_  
Notary Public in and for the  
STATE OF WASHINGTON

Residing at Kennewick  
My Commission Expires 4/28/98

## RESPONSE TO NRC BULLETIN 96-02

Attachment A

Page 1 of 2

1) Plant activities that deviate from descriptions provided in the WNP-2 FSAR:

- The FSAR states that new fuel is loaded into the spent fuel pool using the auxiliary hook on the Reactor Building crane, whereas pool side jib cranes have been used since initial plant startup. The jib cranes are specifically designed for handling new fuel, and for related refueling activities.
- The FSAR states that crane hook periodic inspection will be done using the Dye Penetrant (PT) method, whereas present practice employs the equivalent Magnetic Particle Testing method.

FSAR changes will be processed and will be tracked by the plant corrective action program and PTL.

2) The impact testing recommendation of ANSI N14.6-1978, as referenced in NUREG-0612, to prevent brittle fracture has not been performed for special lifting devices.

A review of the design adequacy of existing special lifting devices was performed. The review considered device material, the stresses due to the specified load limits, member thickness, and temperatures to which the devices could be exposed. This review concluded that impact testing was not required for one of two reasons:

- . the stress levels were lower than the threshold stress for performing impact testing; or,
- . the threshold temperatures for performing impact testing are less than minimum plant design temperatures.

Two enhancements are planned as precautionary measures. Administrative controls will be incorporated, where appropriate, to ensure that devices are not used below their threshold temperatures. In addition plant procedures will be revised accordingly to ensure the ANSI N14.6-1978 impact testing recommendation is addressed in the future design, fabrication, and procurement of special lifting devices.

One exception was noted for the lifting device used for the Reactor Recirculation Pump motors which was supplied by General Electric. The thickness of members used is such that impact testing may be required. It will be necessary to determine the device material to ascertain whether impact testing is required prior to release for use.

**RESPONSE TO NRC BULLETIN 96-02**

**Attachment A**

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This device would be used only in the shutdown refueling or defueled mode to resolve a significant equipment problem. Therefore, there are no plans to use this device in the near future. Administrative controls will be imposed to prevent use of this device until this evaluation is complete.

The identified administrative controls and procedure enhancements will be tracked by the plant corrective action program and PTL.