

# PRIORITY 1

ACCELERATED RIDS PROCESSING

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ACCESSION NBR: 9505180363    DOC. DATE: 95/05/12    NOTARIZED: NO    DOCKET # 05000397  
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Power  
 AUTH. NAME: PARRISH, J.V.    AUTHOR AFFILIATION: Washington Public Power Supply System  
 RECIP. NAME:    RECIPIENT AFFILIATION: Document Control Branch (Document Control Desk)

SUBJECT: Forwards info supporting review of First & Second 10-yr ISI program plans. *See Reports*

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352-0968 • (509) 372-5000

May 12, 1995  
GO2-95-094

Docket No. 50-397

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

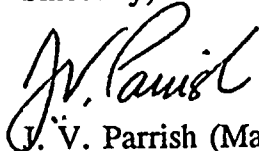
Subject: **WNP-2, OPERATING LICENSE NPF-21  
SECOND 10-YEAR INSERVICE INSPECTION PROGRAM PLAN**

Reference: Letter GO2-94-286, dated December 27, 1994, JV Parrish (SS) to NRC, "Second 10-Year Inservice Inspection Program Plan"

Based on discussions with members of the staff, the attached information is provided in support of your review of the First and Second 10-year Inservice Inspection Program Plans for WNP-2. If additional information is required to clarify the Plan, this information will be provided on request.

Should you have any questions or desire additional information regarding this matter, please call me or D. A. Swank at (509) 377-4563.

Sincerely,



J. V. Parrish (Mail Drop 1023)  
Vice-President, Nuclear Operations

DAS/ml  
Attachments

cc: LJ Callan - NRC RIV w/o  
KE Perkins, Jr. - NRC RIV, Walnut Creek Field Office w/o  
NS Reynolds - Winston & Strawn w/o  
JW Clifford - NRC  
DL Williams - BPA/399 w/o  
NRC Sr. Resident Inspector - 927N w/o

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## FIRST INTERVAL ISI PROGRAM PLAN

ISI-2-003

### Questions:

1. This request for relief is titled as revised, which revision is this?
2. It is stated that "The component supports and welded attachments are completely or partially inaccessible to all examination techniques. No alternate examinations are proposed."

Is it being asked to only do partial examinations or no examinations?

- a) component supports
- b) integral attachments

### Responses:

1. This is revision 1 to the relief request that adds the items starting with FPC-64.
2. We are requesting both partial examinations and no examination depending on the item. The attached table summarizes the specific relief requested for each item. The drawings of the affected items are included.

ISI-2-011

### Questions:

1. It is stated that "Relief is required from ASME Section XI examination requirements for the two C3.40 welds on the basis of inaccessibility of the welds due to their location in a pipe chase where access will place a hardship on the plant to gain access." What are the hardships?
2. The basis states that the welds are inaccessible. Under impact on plant quality and safety it is stated that a partial code exam will be done. What examinations will be performed on the C3.40 welds?
3. If they are partial, what is the % completion?
4. As part of the impact on quality and safety discussion, items 1, 3, and 4 are used. Is there an item 2?



Responses:

1. To gain access to the attachment welds RHR-77(W) and RHR-410(W) to perform a surface examination will require disassembly of the shielding wall for the pipe chase which creates a hardship on the plant.
2. The welds are inaccessible. In the first sentence of after item 4 on page 2 of ISI-2-001, the phrase "doing only a partial Code" should read "not performing".
3. No examination is requested.
4. There is no item 2. The paragraphs should be renumbered 1, 2, and 3.

## SECOND INTERVAL ISI PROGRAM PLAN

2ISI-06

The Reactor Plant Cold Startup procedure 3.1.2 revision 26 identifies a Drywell inspection at 400 psig and 920 psig. This inspection is not required by the procedure for all plant startups, but the inspection is performed following refueling outages. Corrective measures taken for leakage at the CRD flanged connections during the pressure test depend on the degree that the flange leaks. The corrective measures include retorguing the bolting, scrambling the drive, removing and re-installing the drive, and evaluating the leak to determine if it is decreasing.

### Program Plan

page 5-52

The examination of B1.12 items complies with Section XI. Under column "Complies with Section XI" should be "Yes" and under column "Request for Relief No." should be blank for item B1.12.

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List of Component Supports  
and Class 3 Welded Attachments  
For Which Relief is Requested

Identification Number	Degree of Examination Coverage	Relief Requested
SW-69	No coverage	To perform no examinations on component support attachment welds.
SW-67	No coverage	To perform no examinations on component support attachment welds.
SW-72	No coverage	To perform no examinations on component support attachment welds.
SW-317	No coverage	To perform no examinations on component support attachment welds.
SW-152	No coverage	To perform no examinations on component support attachment welds.
SW-431	No coverage	To perform no examinations on component support attachment welds.
SW-137	No coverage	To perform no examinations on component support attachment welds.
SW-438	No coverage	To perform no examinations on component support attachment welds.
SW-203	No coverage	To perform no examinations on component support attachment welds.



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List of Component Supports  
and Class 3 Welded Attachments  
For Which Relief is Requested

Identification Number	Degree of Examination Coverage	Relief Requested
SW-77	No coverage	To perform no examinations on component support attachment welds.
SW-34	No coverage	To perform no examinations on component support attachment welds.
SW-142	No coverage	To perform no examinations on component support attachment welds.
SW-60	No coverage	To perform no examinations on component support attachment welds.
SW-916N	No coverage	To perform no examinations on component support attachment welds.
SW-75	No coverage	To perform no examinations on component support attachment welds.
RCIC-18	No coverage	To perform no examinations
FPC-64	Less than 10% of hanger can be examined. The rest is buried in fire barrier.	To perform no examinations
FPC-64(W)	No coverage	To perform no examinations
FPC-98	No coverage	To perform no examinations on component support attachment welds.
FPC-114	No coverage	To perform no examinations on component support attachment welds.



List of Component Supports  
and Class 3 Welded Attachments  
For Which Relief is Requested

Identification Number	Degree of Examination Coverage	Relief Requested
FPC-203	Bottom kicker plate covered by fire barrier. Greater than 90% of support is examined.	To examine at least 90% of component support
LPCS-19	No coverage	To perform no examinations
RHR-53	Greater than 90% of the hanger can be examined including the spring.	To examine at least 90% of component support
RHR-99	No coverage	To perform no examinations on component support attachment welds.
RHR-174	No coverage	To perform no examinations
RHR-605	Enclosed in cubicle or pipe chase	To perform no examinations
RHR-606	Enclosed in cubicle or pipe chase	To perform no examinations
SLC-4453-57	Less than 50% examination coverage. Most of the hanger is covered by fire barrier.	To perform no examinations
SW-90	No coverage	To perform no examinations
SW-90(W)	No coverage	To perform no examinations
SW-123	75% of hanger can be examined, the rest is buried in fire barrier.	To examine at least 75% of component support



List of Component Supports  
and Class 3 Welded Attachments  
For Which Relief is Requested

Identification Number	Degree of Examination Coverage	Relief Requested
SW-123(W)	75% of the lugs can be examined. The rest is buried in fire barrier.	To examine 75% of the lugs.
SW-439	No coverage	To perform no examinations
SW-439(W)	No coverage	To perform no examinations
SW-946N	No coverage	To perform no examinations
SW-946N(W)	No coverage	To perform no examinations
SW-951N	No coverage	To perform no examinations
SW-951N(W)	No coverage	To perform no examinations
SW-950N	No coverage	To perform no examinations

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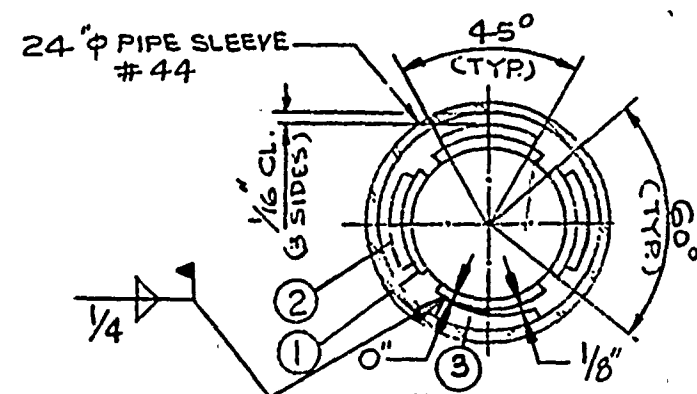
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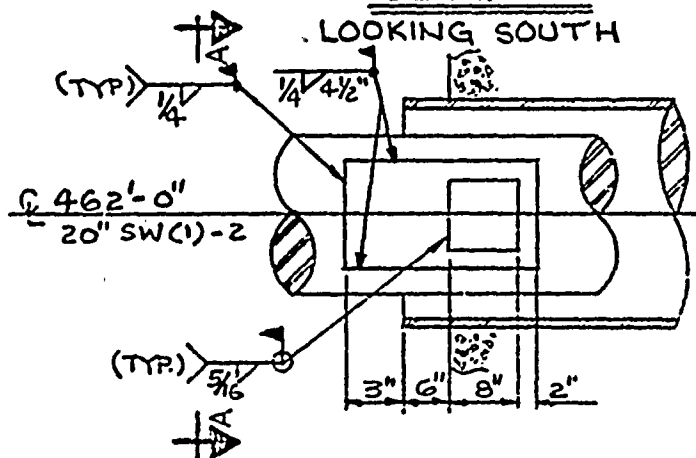




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SECT. A-A

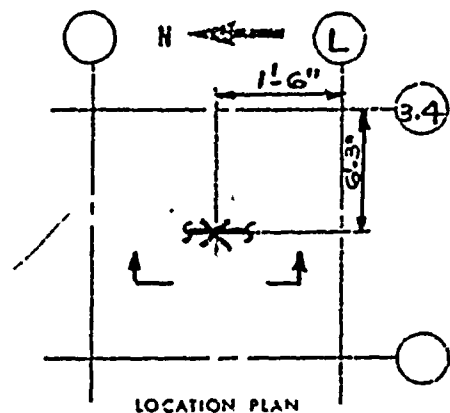
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ZONE: R-22

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### LOCATION PLAN

SEIS. LOADS: VERT = $\pm 11553$ N-S = —		E-W = $\pm 17039$		OP. LD. = 5807*	HYDRO. LD. = 5807*	THERM. LD. = —
CODE / CLASS: III/3	GROUP: 2	Q.A. LEVEL: I	PAINT: I	PIPE CALC. D1459	STEEL CALC. 8.16.2346	
OWNER WASHINGTON PUBLIC POWER SUPPLY SYSTEM			PIPING SYSTEM <u>STANDBY SERVICE WATER</u>			
PROJECT HANFORD NO. 2		ENGINEER BURNS & ROE INC.	150: SW - 250 - 31.40 H			
CUSTOMER BOVEE & CRAIL / GERI			REFERENCE DWG. <u>PIPING: M712</u>			
			MARK NO. <u>SW-67</u>		AM-186	



nps industries. inc.

CONTRACT 215

a subsidiary of  
nuclear power services, inc.  
25 broadway, new york, n.y. 10004

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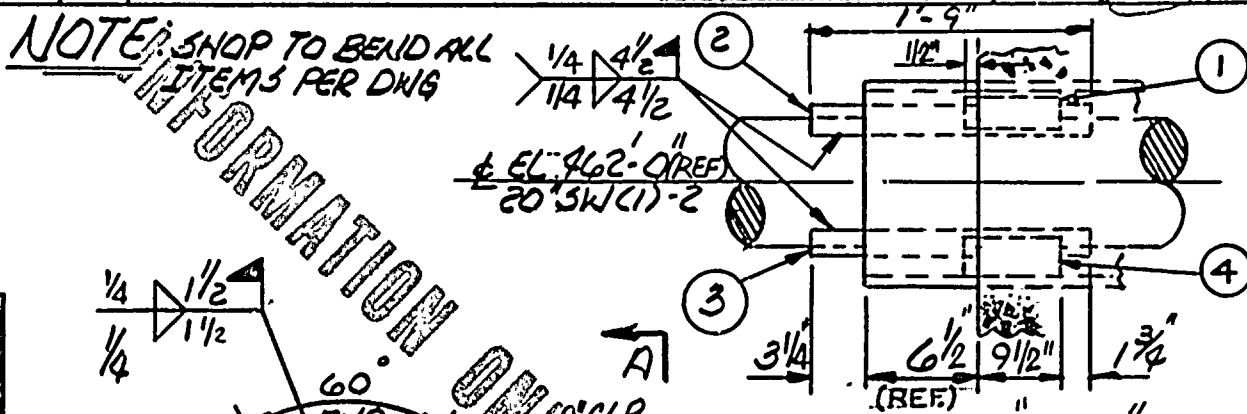




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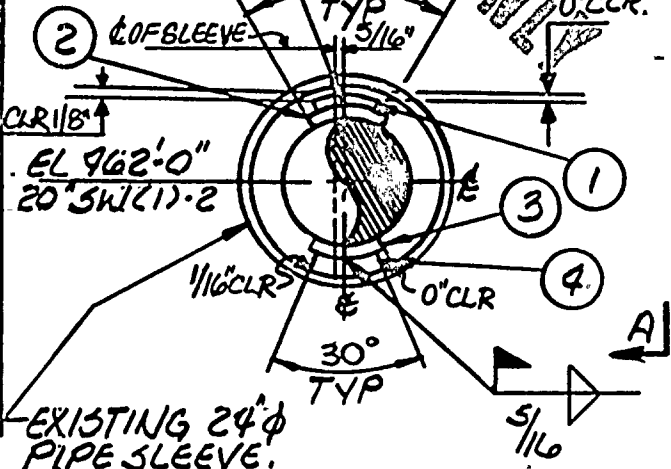
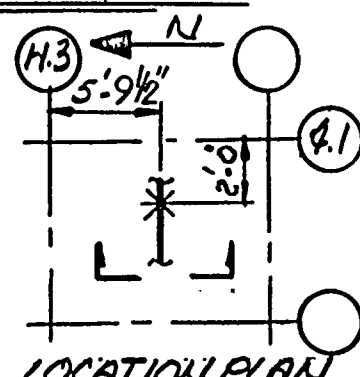
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2	1	R 5/8" X 1 1/8" X 1'-9" (BEND TO RAD. OF PIPE)	A-36	50
3	1	R 3/4" X 1 1/8" X 1'-9" (BEND TO RAD. OF PIPE)	A-36	66
4	1	R 1" X 6 1/8" X 0'-10" (COPE TO MATCH PIPE SLEEVE)	A-36	17
TOTAL WT.				150#

NOTE: SHOP TO BEND ALL ITEMS PER DWG



### SECTION A-A

LOCATION PLAN FOR REFERENCE ONLY! USE FAB. ISO. FOR CONSTRUCTION



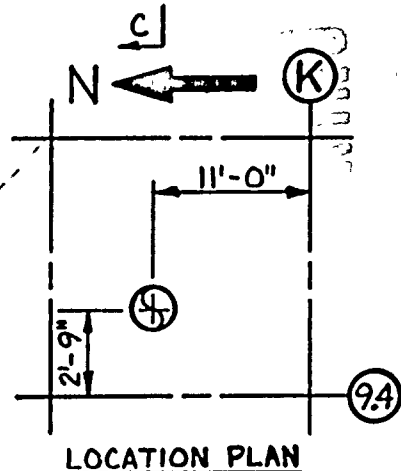
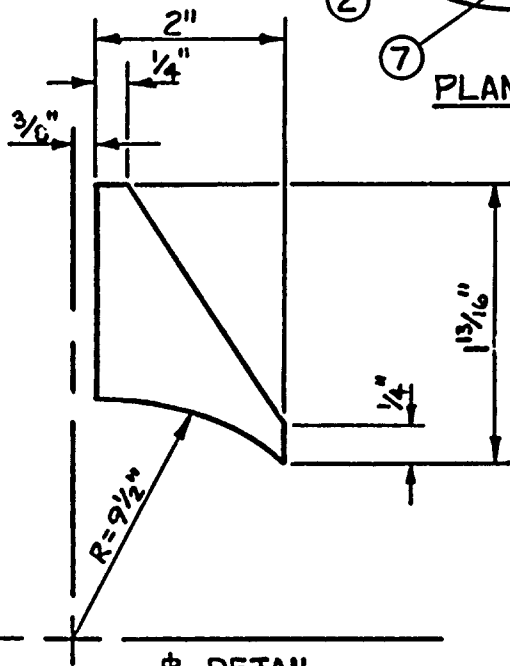
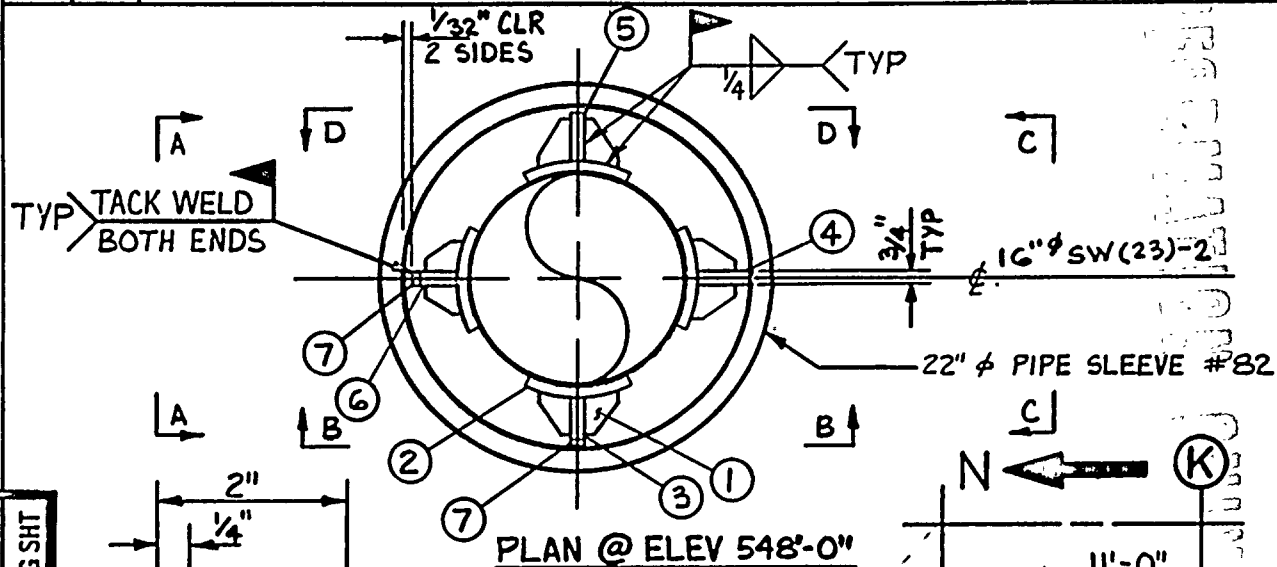
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1	REDRAWN BY BFR TO INCORPORATE PED-H-3748	9/16/80	VP	MA	5%
REV NO	REVISION	DATE	CHKD	APVD	
OWN	PARKER	CHKD	MAW	P.N.	SCALE NTS
SL/LDE	MAW	DATE	9-24-80		
ENGINEERING REVIEW					
MECH		CIVIL	M. Ho	9/16/80	
ELEC					
REVD	H.R. Figue	APVD	MAW	DATE	9/30/80
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SEISMIC LD.: VERT: 1.5210#	N-S: —	E-W: —
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PAINT: I	PIPE CALC: 8.42.023	STL CALC: 8.16.1327
PIPING STAND-BY SYSTEM SERVICE WATER	REF. DWG ISO: SK-230-31.40	PIPING: M 712
WASHINGTON PUBLIC POWER SUPPLY SYSTEM		
HANFORD NO.2 58.0 PS		
MARK NO. SK-317		
BURNS AND ROE, INC.		
Engineers and Constructors		
New Jersey • New York • Connecticut • California		
W.O.3808	DWG. SK-317	REV 3



ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	WT.	
1	24	PL $\frac{3}{8}$ " x $1\frac{1}{16}$ " x 0'-2" (SEE PL DETAIL)	A36	9	
2	4	PL $\frac{1}{2}$ " x $9\frac{7}{16}$ " x 1'-9" (PAD) (BEND TO PIPE RAD)	A36	112	
3	1	PL $\frac{3}{4}$ " x $2\frac{1}{16}$ " x 0'-9" (SHEAR LUG)	A36	4	
4	1	PL $\frac{3}{4}$ " x $1\frac{1}{8}$ " x 0'-9" (SHEAR LUG)	A36	2	
5	1	PL $\frac{3}{4}$ " x $1\frac{3}{4}$ " x 0'-9" (SHEAR LUG)	A36	3	
6	1	PL $\frac{3}{4}$ " x $2\frac{1}{8}$ " x 0'-9" (SHEAR LUG)	A36	5	
7	2	PL $\frac{1}{8}$ " x 1" x 1'-7" (SHIM)	A36	1	
TOTAL WT				136#	



ZONE: R-SI	OR.LD.: —	HYDRO.LD.: —
THERMAL LD.: VERT.: —	N-S: —	E-W: —
SEISMIC LD.: VERT.: —	N-S: $\pm 18,673\#$	E-W: $\pm 2,790\#$
CODE/CLASS: III/3	GROUP: 2	Q.A.LEVEL: I
PAINT: I	PIPE CALC.: P-1465	STL.CALC.: 8.16.2399
PIPING STANDBY	REF.DWG. 150: SW-296-1.5H	
SYSTEM SERVICE WATER	PIPING: M-712	

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2

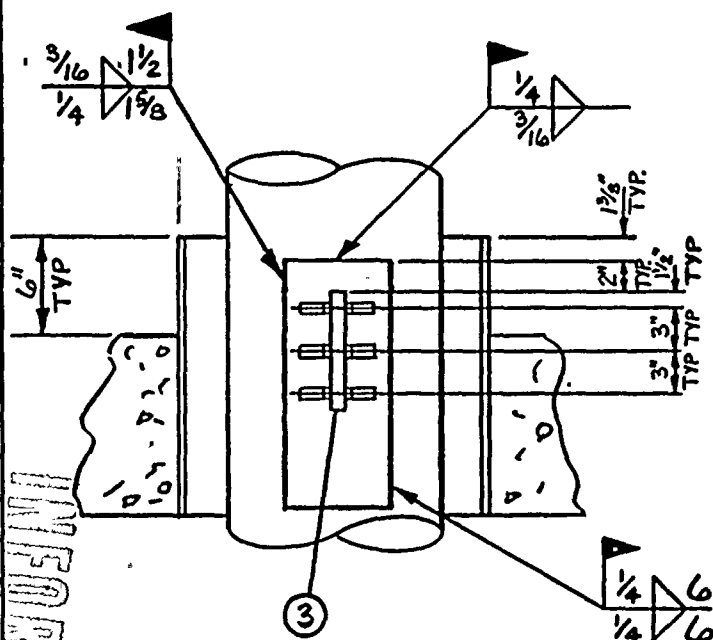
MARK NO. SW-152  
BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O.3808 DWG. SW-152 SHT 1 OF 2  
REV 1

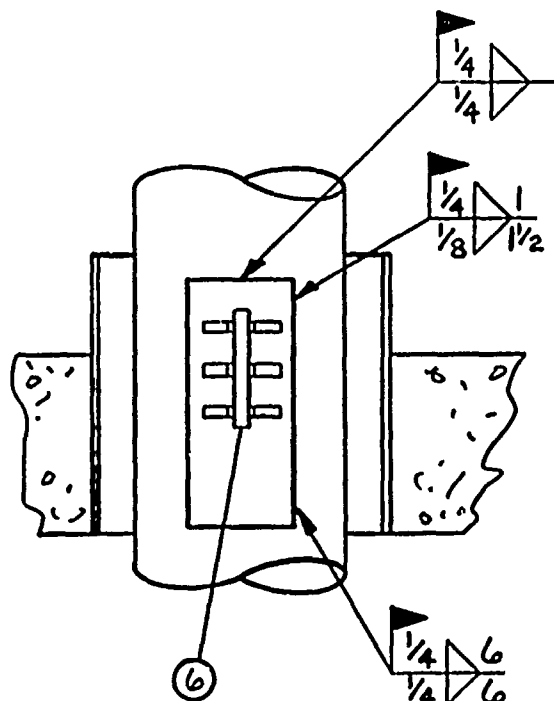
REV NO	REVISION	DATE	APVD
1	REVISED IN ACCORDANCE W/WBG STATUS AS-BUILT	4/29/81	ACH
DWN A HARGRAVES	CHKD (P)	SCALE NTS	
SL/LDE	DATE 5/4/81		
MECH	CIVIL		
ELEC			
REV'D	DATE		
H.R. Tamm	5/4/81		
CHIEF DRAFTSMAN	CHIEF	MECH. ENGINEER	



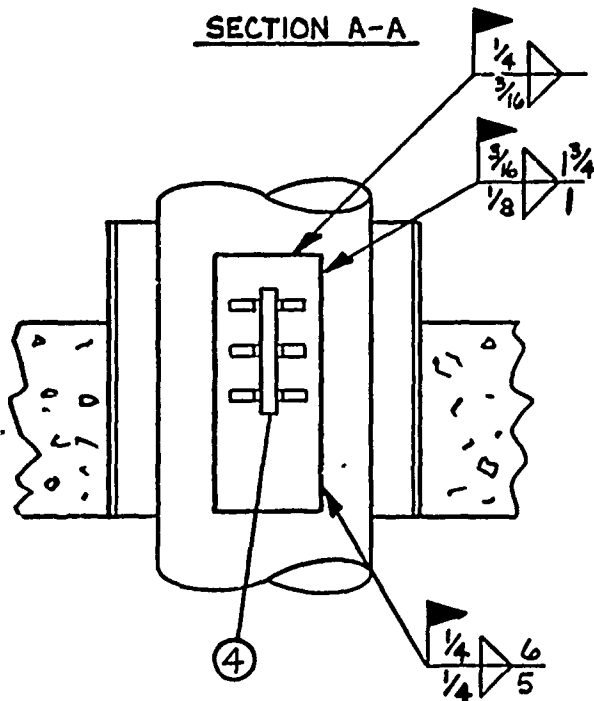




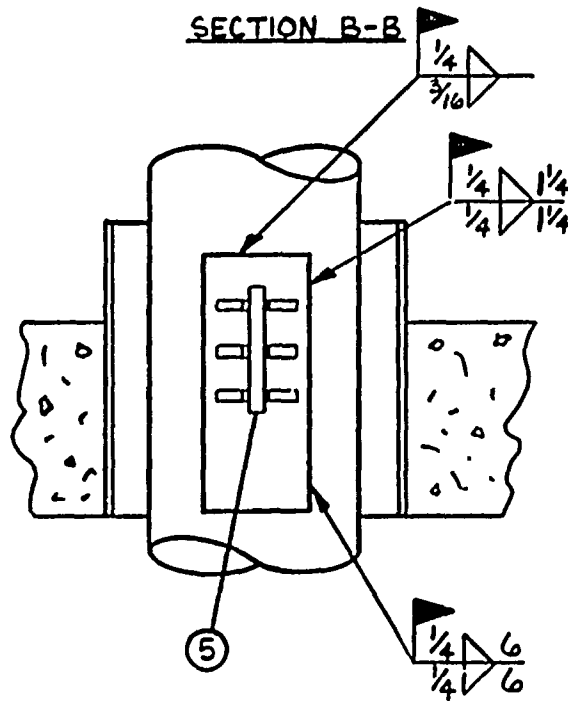
**SECTION A-A**



SECTION B-B



SECTION C-C

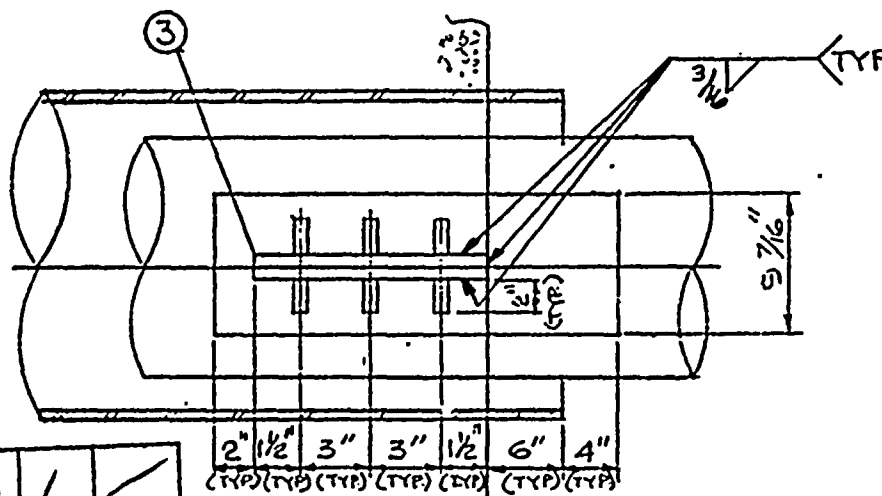
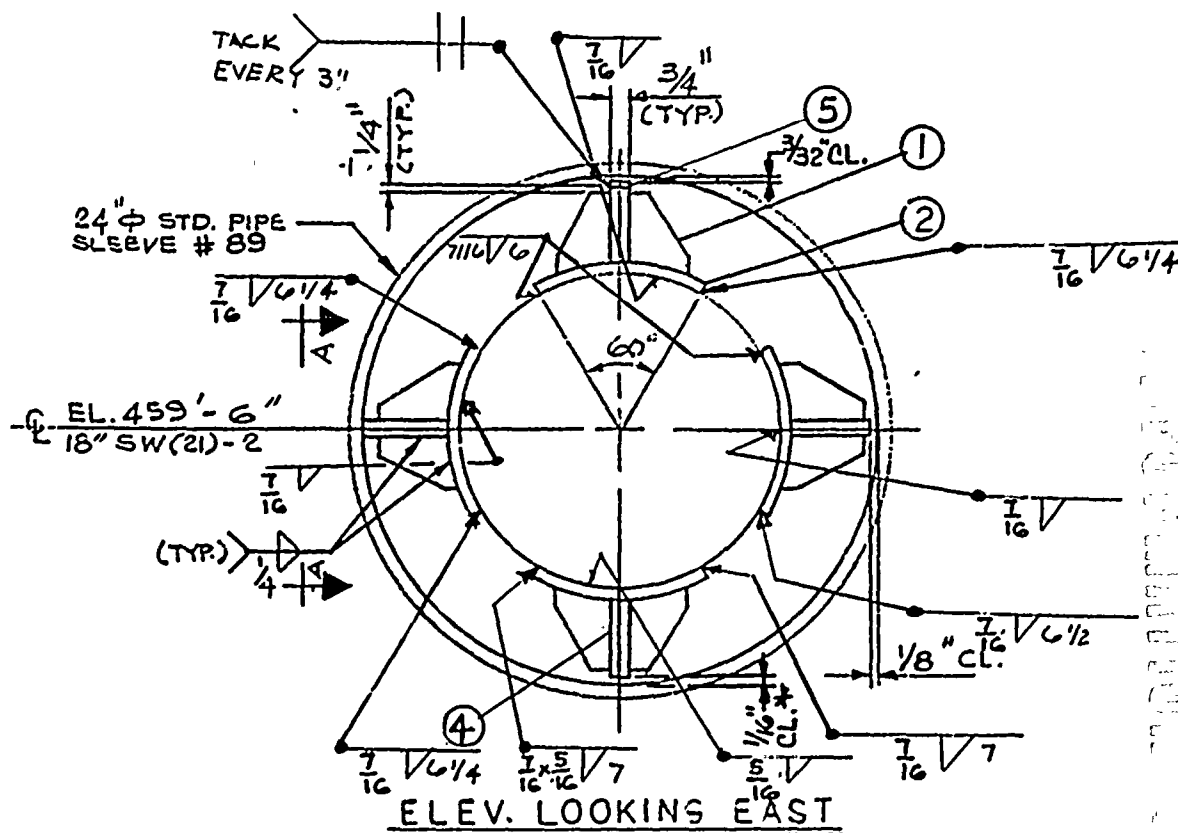


SECTION D-D

1 REVISED IN ACCORDANCE W/WBG STATUS AS-BUILT				4/29/81 ACH 0 A/C/S 5H				WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2					
REV NO		REVISION		DATE		DWN CHKD		APVD					
DWN A HARGRAVES				CHKD 10		SCALE NTS		MARK NO. SW-152					
BL/LDE		1/11		DATE 5/4/81									
ENGINEERING REVIEW										BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California			
MECH				CIVIL									
ELEC													
REVD				APVD		DATE		W.O.3808		DWG. SW-152		REV	
H.R. Tupper				S.M. Mackay		4/6/81						SHT 2 OF 2	
CHIEF DRAFTSMAN				CHIEF MECH		ENGINEER						1	



DATE \_\_\_\_\_



02- 215-08	11036	1	/
CVI NO	CVI SHT	REV	DWG SHT

SECTION A-A

\* DESIGN CLEARANCE = 0"

OWNER	
WASHINGTON PUBLIC POWER SUPPLY SYSTEM	
PROJECT	ENGINEER
HANFORD NO.2	BURNS & ROE INC.
CUSTOMER	
BOVEE & CRAIL / GENI	

PIPING SYSTEM		SERVICE WATER
ISO.: SW-296-7.16 H		
REFERENCE DWG.		PIPING: M 712
MARK NO.	SW-431	AM-184



**nps industries. inc.**

**CONTRACT 215**

a subsidiary of  
nuclear power services, inc.  
26 broadway, new york, n.y. 10004

DRAWN	DATE	CHK'D.	DATE	APPV'D.	DATE
AB	9.11.77	HS	9.13.77	JG	11-6-77
JOB NO.					
DWG NO. SW-431 (1 OF 2)				REV.	1







DK.

DESCRIPTION

DATE

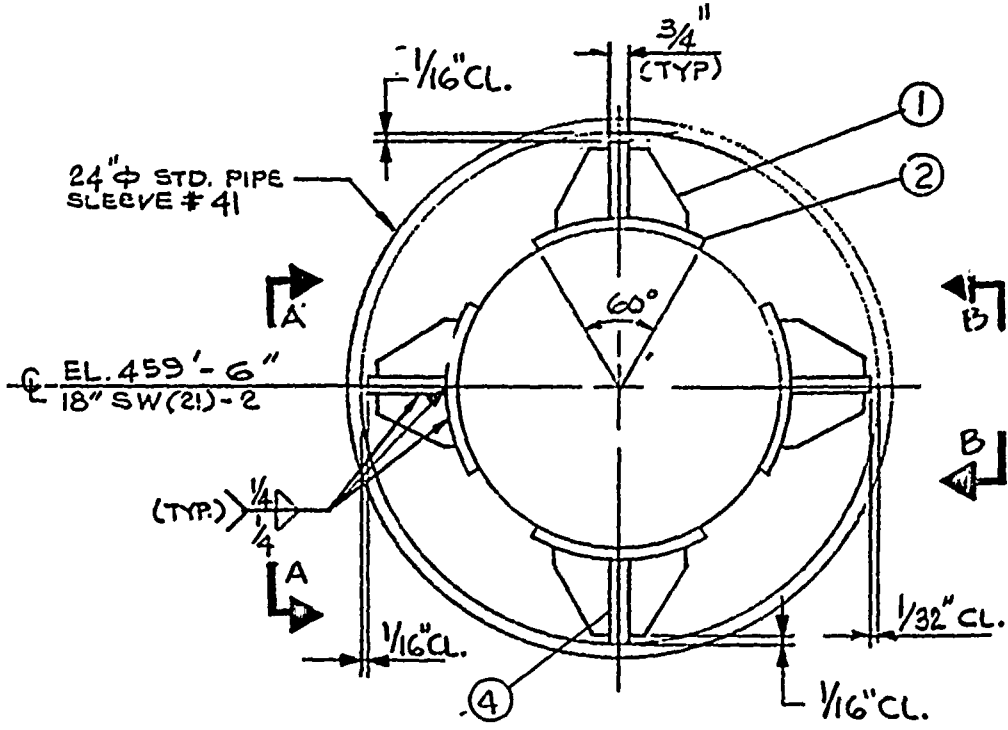
DR. APPR. REV.

DESCRIPTION

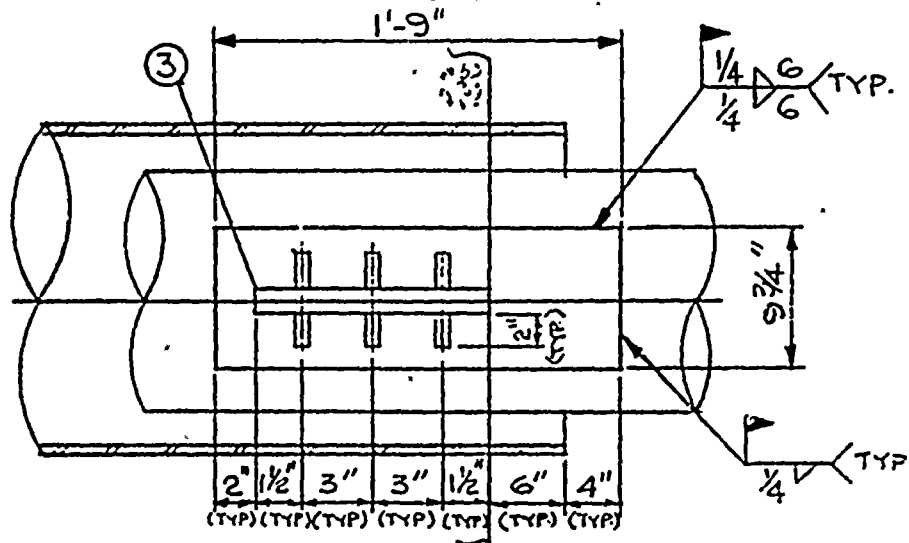
REV. DATE

REV. IN ACCORDANCE W/ WEG STATUS AS-BUILT

CVI	NO	CVI	SHT	REV	DWG	SHT
02	15-05	10000	1	1	1	1



ELEV. LOOKING SOUTH



SECTION A-A

OWNER <b>WASHINGTON PUBLIC POWER SUPPLY SYSTEM</b>		PIPING SYSTEM <b>SERVICE WATER</b>	
PROJECT <b>HANFORD NO.2</b>	ENGINEER <b>BURNS &amp; ROE INC.</b>	ISO. : SW-256-17.26 H	
CUSTOMER <b>BOVEE &amp; CRAIL / GERI</b>		REFERENCE DWG. <b>PIPING: M 712</b>	
		MARK NO. <b>SW-137</b>	<b>AM-184</b>



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**nuclear power services, inc.**  
26 broadway-new york, n.y. 10004

DRAWN	DATE	CHK'D.	DATE	APPV'D.	DATE
ME	9-26-76	JJM	10-1-76	JG	11-4-77
JOB NO.					
DWG NO. SW-137 (1 OF 2)					
REV. 1					

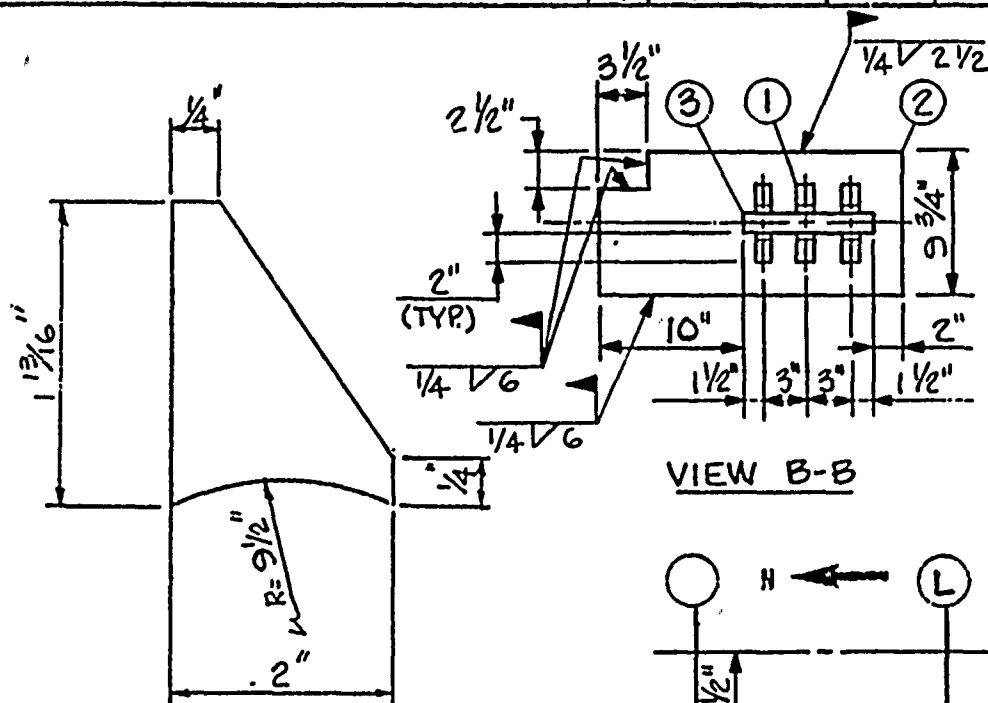
NOT TO SCALE

BCS-715-14935

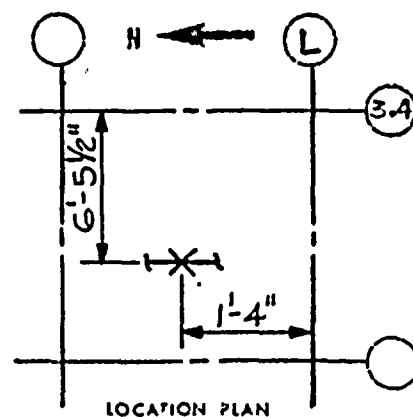




	DR.	APP.	ITEM NO	NO. REQ'D.	SIZE	DESCRIPTION	ASTM	WT.
			1	24	3/8"	X 1' 3/16" X 2" LG. PLATE (SEE PLATE DET.)	A36	24
			2	4	1/2"	X 9 7/16" X 1'-9" LG. PLATE	A36	116
			3	3	3/4"	X 2' 1/16" X 9" LG. SHEAR LUG	A36	12
			4	1	3/4"	X 2' 1/8" X 9" LG. SHEAR LUG	A36	4
						TOTAL WT.		156 #



DETAIL-1



### LOCATION PLAN

ZONE: R-22

SEIS. LOADS: VERT =  $\pm 8127$  # N-S =  $\pm 13507$  # E-W = 13507 # OP. LD. = 4560 # HYDRO. LD. = 4560 # THERM. LD. =  $\pm 1000$  #

CODE / CLASS: III/3	GROUP: 2	Q.A. LEVEL: I	PAINT: I	PIPE CALC. P-1465	STEEL CALC. S-2425
---------------------	----------	---------------	----------	-------------------	--------------------

**OWNER**

# WASHINGTON PUBLIC POWER SUPPLY SYSTEM

# PROJECT

**HANFORD NO.2**

**ENGINEER**

**BURNS & ROE INC.**

**CUSTOMER**

BOVEE & CRAIL / GERI

PIPING SYSTEM SERVICE WATER

ISO.: SW-296-17.26H

REFERENCE DWG. PIANG: M 712

MARK NO. SW-137

AN-184



**nps industries. inc.**

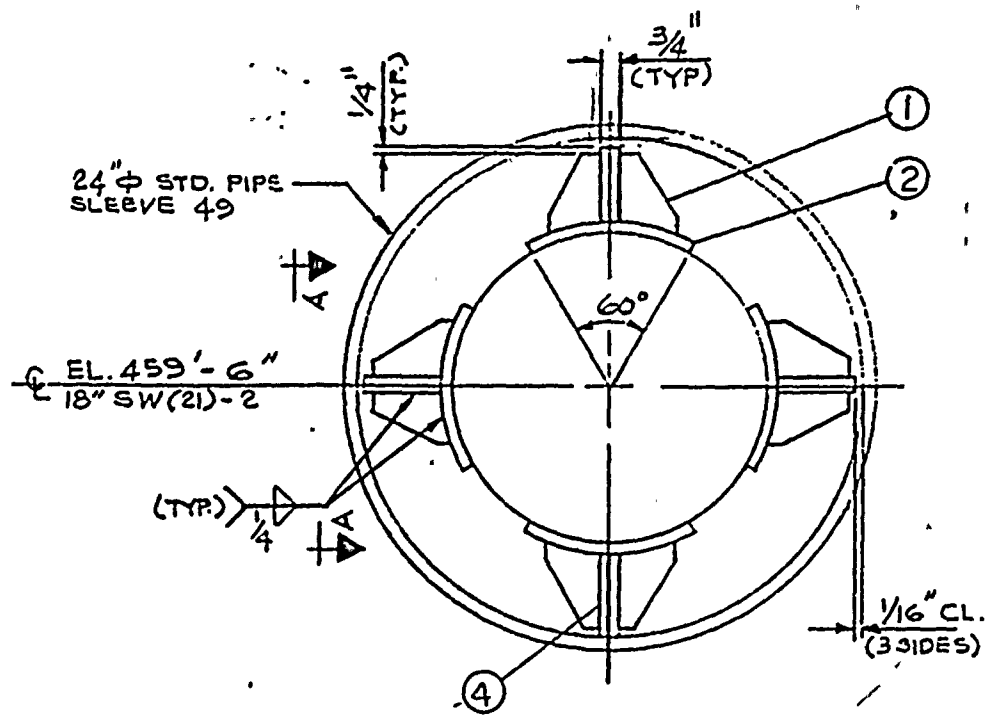
**CONTRACT 215**

a subsidiary of  
nuclear power services, inc.  
26 broadway - new york, n.y. 10004

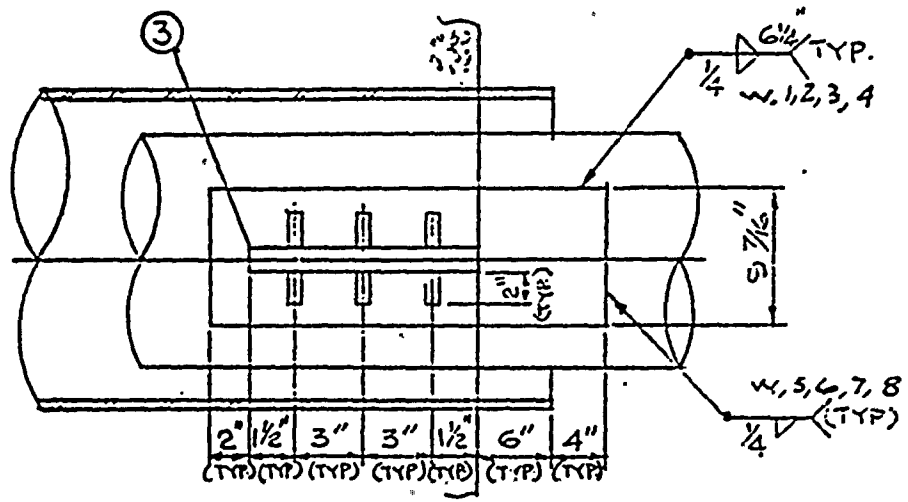
DRAWN	DATE	CHK'D	DATE	APPV'D	DATE
MF	9.26.76	JJM	10.1.76	J(-)	11.1.77
JOB NO.					
DWG. NO SW-137 (2453)				PLV	



REV.	DATE	DESCRIPTION	DR.	APP.
1	4/30/81	REV. IN ACCORDANCE W/ WEG STATUS AS-BUILT		



ELEV. LOOKING SOUTH



SECTION A-A

ZONE: R22 SU. 58.0

OWNER WASHINGTON PUBLIC POWER SUPPLY SYSTEM	ENGINEER BURNS & ROE INC.
PROJECT HANFORD NO. 2	CUSTOMER BOVEE & CRAIL / GENI

PIPING SYSTEM SERVICE WATER	ISO.: SN-296-17.26 H
REFERENCE DWG. PIPING: V 712	MARK NO. SW-438
	AM-184



nps industries, inc.

CONTRACT 215

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nuclear power services, inc.  
26 broadway-new york, n.y. 10004

DRAWN	DATE	CHK'D	DATE	APPV'D	DATE
AB	9.6.77	HE	9.13.77	JG	11.4.77
JOB NO.					
DWG NO. SW-438 (1 OF 2)				REV.	

INFORMATION

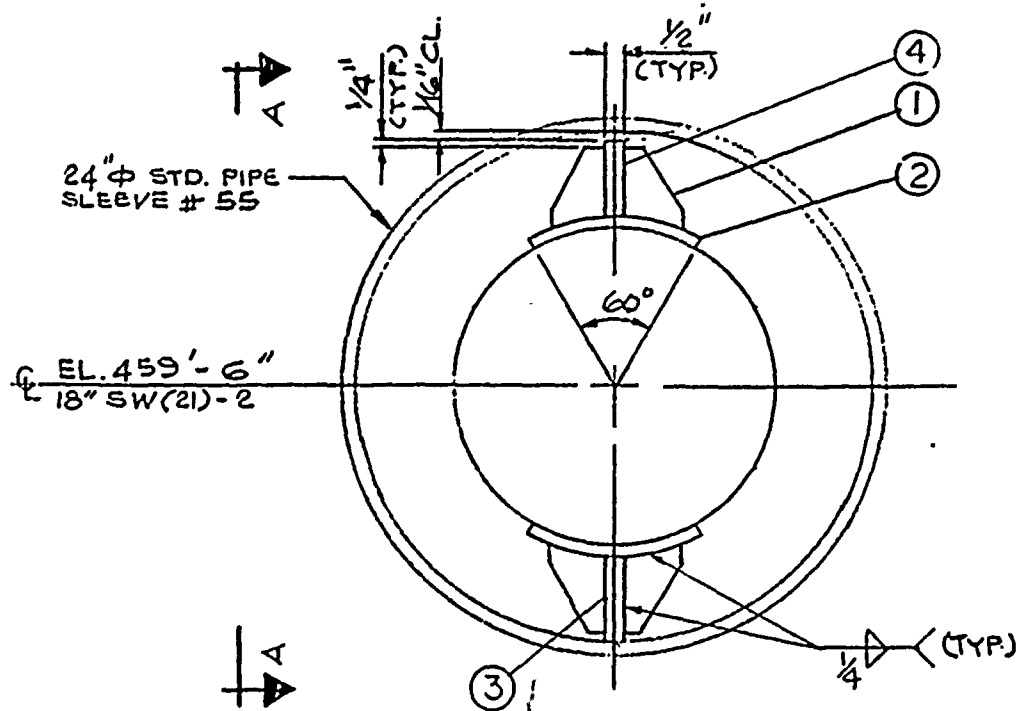
WEG-215-1493



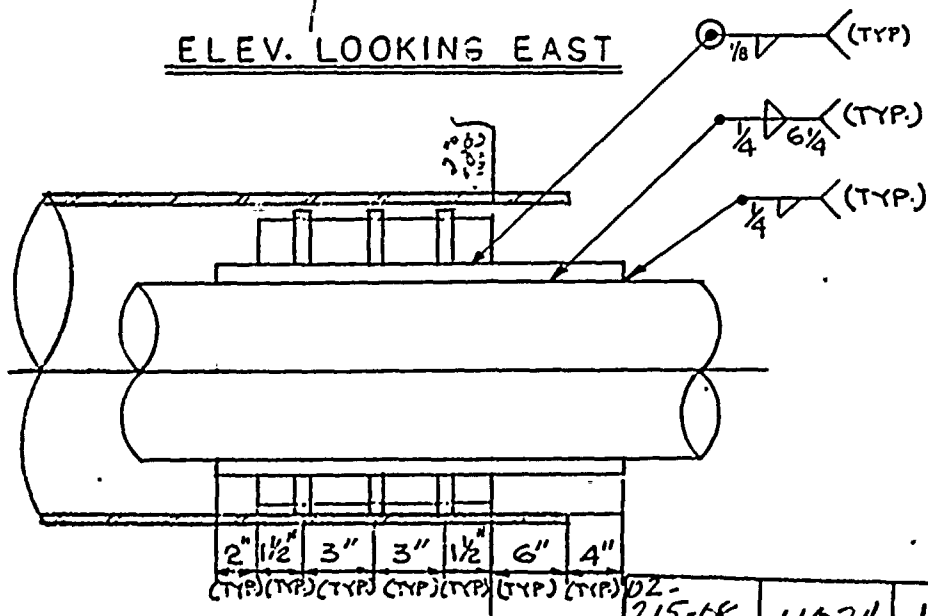




INFORMATION



ELEV. LOOKING EAST



SECTION A-A

CVI NO	CVI SHT	REV	DWG SHT
215-68	11024	1	

OWNER  
WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
PROJECT  
HANFORD NO. 2  
ENGINEER  
BURNS & ROE INC.  
CUSTOMER  
BOVEE & CRAIL / GERI

PIPING SYSTEM  
STAND BY SERVICE WATER  
ISO.: SW-296-17.26  
REFERENCE DWG. PIPING: M 712  
MARK NO. SW-203  
AM-184



nps industries. inc.

CONTRACT 215

a subsidiary of  
nuclear power services, inc.  
26 broadway-new york, n.y. 10004

DRAWN	DATE	CHK'D.	DATE	APPV'D.	DATE
MF	9.27.76	RW	10.1.76	JG	11-471
JOB NO.					
DWG NO. SW-203 (1 OF 2)					
REV. 1					

BCR-215-1493



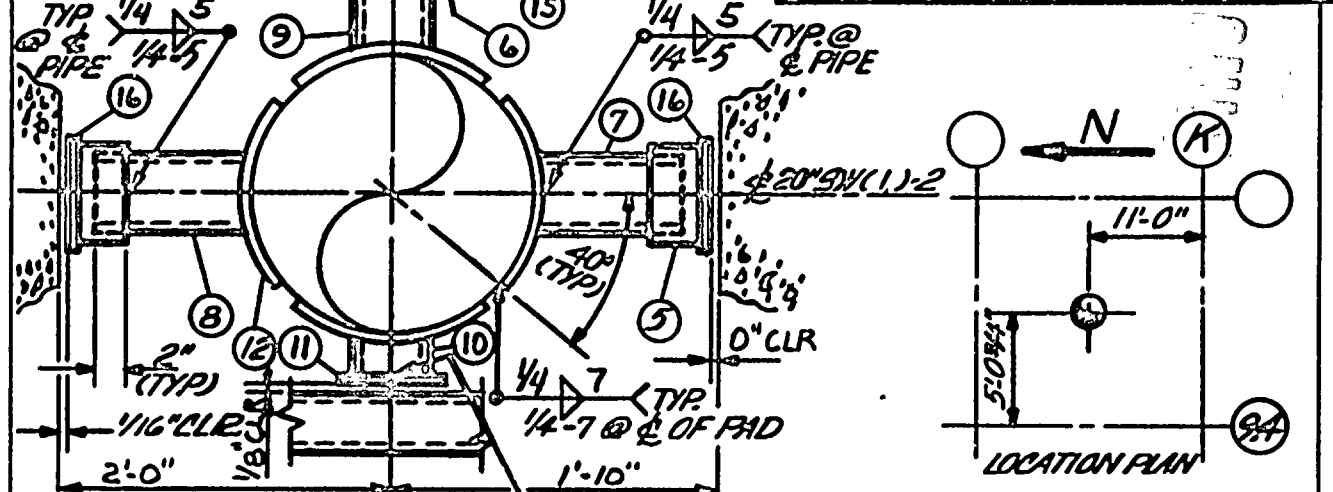






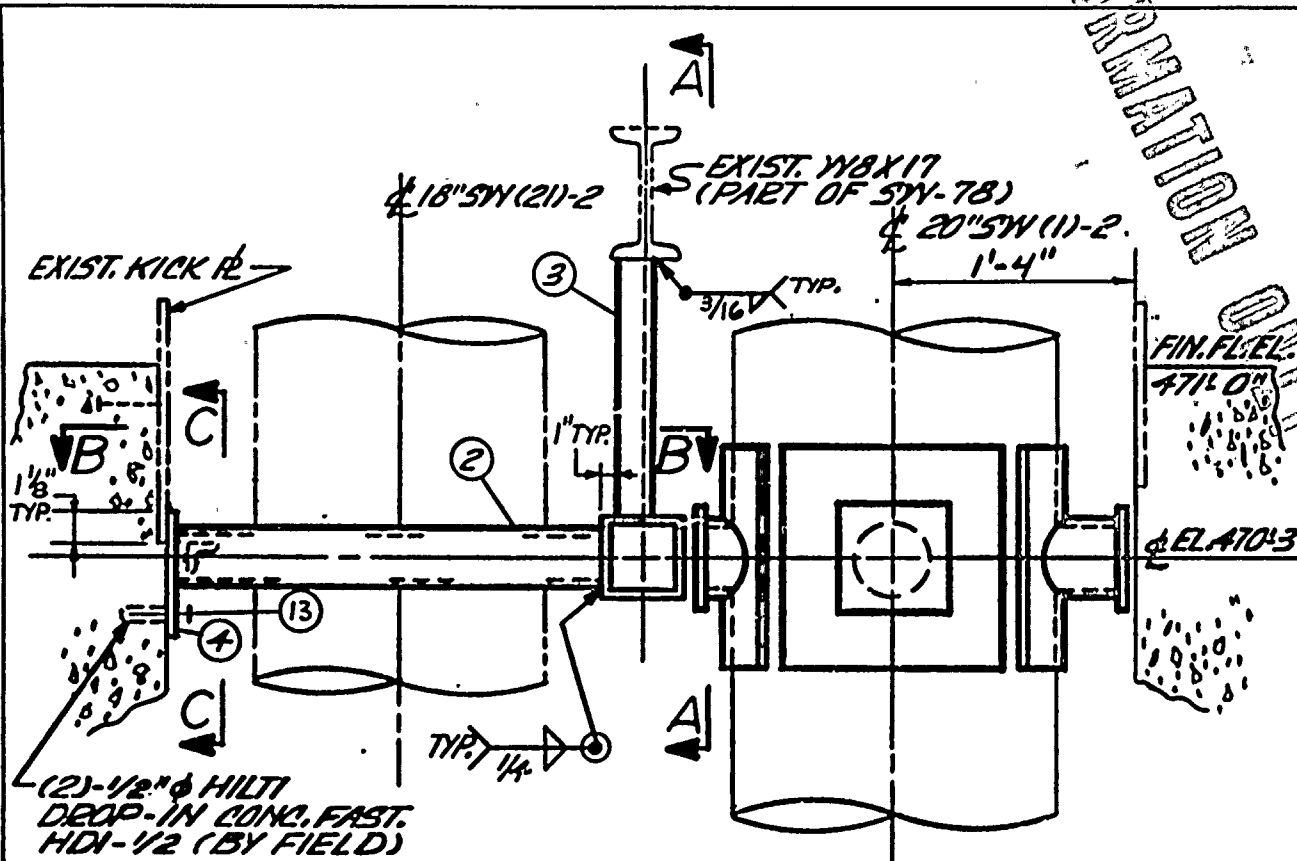
ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	1	TS 5X5X.375X2'-6" LG.	A36	55
2	2	TS 4X4X.375X2'-2 1/2" LG.	A36	74
3	2	L 2"X2"X 1/4"X1'-37 1/2" LG.	A36	8
4	2	PL 1/2"X6"X0'-8"	A36	14
5	2	PIPE 6" SCH. 80 X0'-3" LG.	A106	14
6	1	PL 1/2"X7"X0'-7"	A36	7
7	1	TEELION 5" SCH. 80 PIPE X0'-10 1/2" LG.	A106	18
8	1	5" SCH. 80 PIPE X0'-11 1/2" LG. (TEELION)		20
9	1	5" SCH. 80 PIPE X0'-4 1/2" LG. (TO SLUT)		8
10	1	TEELION 5" SCH. 80 PIPE X0'-17 1/2" LG.	A106	3
11	1	PL 1/2"X6 1/2"X0'-6 1/2"	A36	12
12	4	TEELION PAD-PL 1/2"X14"X1'-2 1/4"	A36	120
13	2	1/2" X 1 1/4" LG. HYX. HEX. NUTS W/L.W.		1
14	1	L 2"X2"X 1/4"X1'-8" LG.	A36	5
15	1	PL 1/8"X7 1/8"X7 1/2"	A36	2
16	2	PL 1/2"X8"X8"	A36	18
TOTAL:				379*

02-	215-68	11589	2	✓
CVI NO	CVI SHT	REV	DWG SHT	

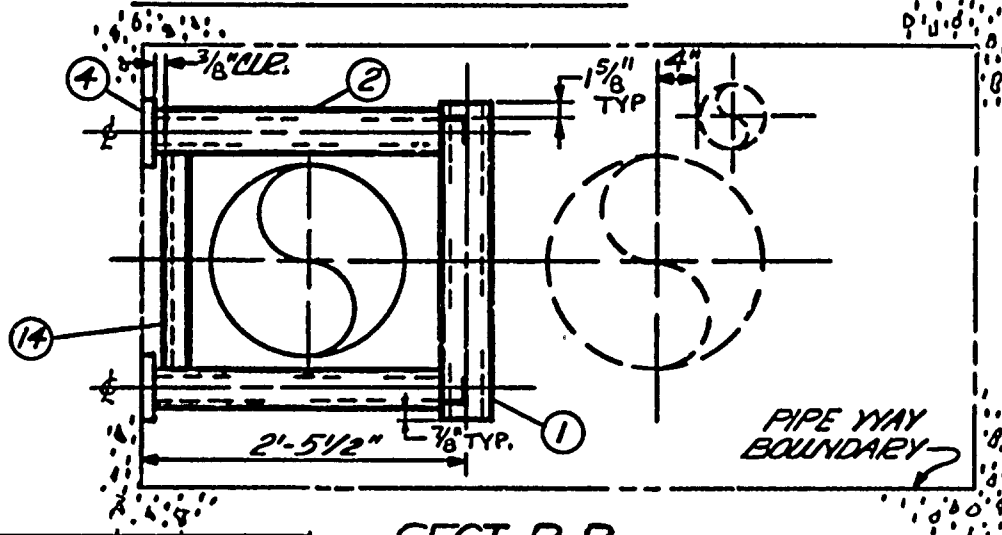


<div>PLAN @ ELEV. 470'-3"</div> <div>1/4" = 1'-0"</div> <div>PIPE 1/4" = 1'-0"</div>						ZONE: <u>E21</u>		OR LD.: ~		HYDRO. LD.: ~	
						THERMAL LD.: VERT.: ~		N-S: ~		E-W: ~	
						SEISMIC LD.: VERT.: ~		N-S: <u>18791#</u>		E-W: <u>18022#</u>	
						CODE/CLASS: <u>III/3</u>		GROUP: <u>2</u>		Q.A. LEVEL: <u>I</u>	
						PAINT: <u>I</u>		P&E CALC.: <u>P459</u>		STL. CALC.: <u>B&amp;G 604</u>	
						PIPING SYSTEM: <u>SERVICE WATER</u>		REF. DWG. 130: <u>SW-250-41.504</u>		PIPING: <u>M712</u>	
2 REVISED IN ACCORDANCE W/WB6 STATUS AS-BUILT						WASHINGTON PUBLIC POWER SUPPLY SYSTEM					
1 REDESIGNED BY <u>B&amp;R</u>						HANFORD NO.2					
REV NO		REVISION		DATE		OWN		CHKD		APVD	
OWN		B. SANDERS		CHKD		W. B. M.		SCALE		NONE	
SL/DGE		PORTER		DATE		3-7-80					
ENGINEERING REVIEW											
MECH				CIVIL							
ELEC											
REVD				DATE							
H.R. Tague				3-13-80							
CHIEF DRAFTSMAN				CHIEF EXCH. ENGINEER							
W.O.3808						DWG. SW 77 SHT. 1 OF 3					
						REV 2					





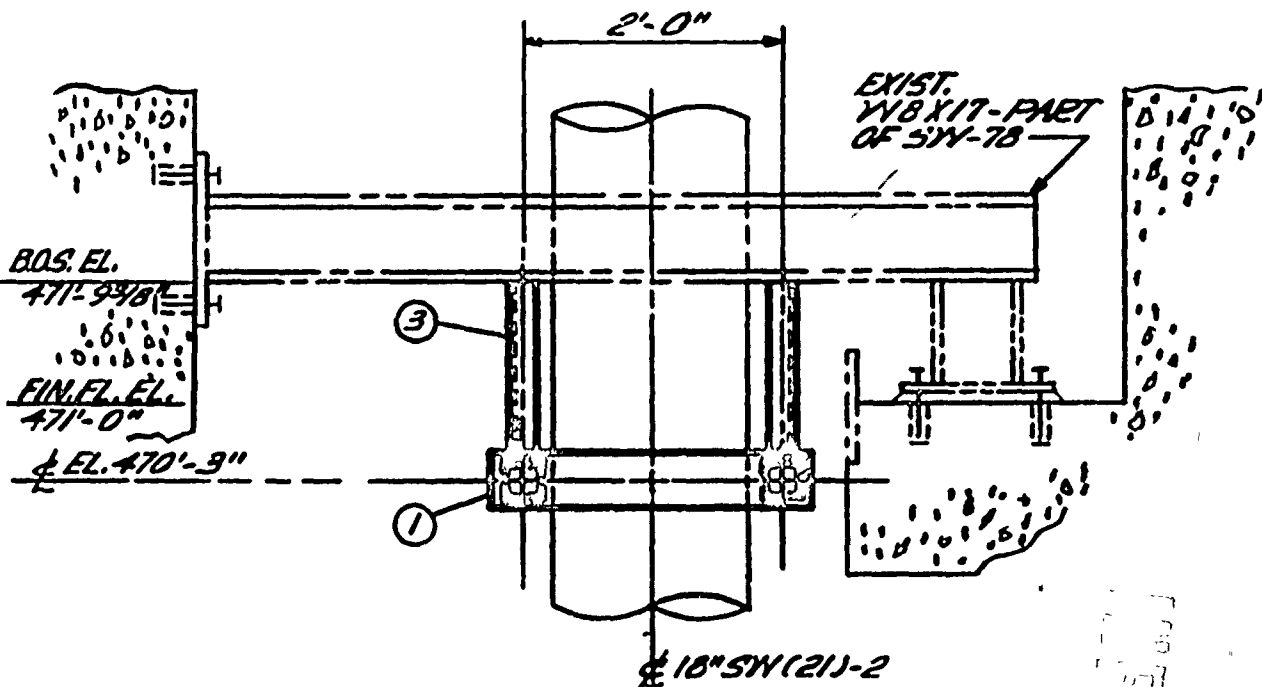
ELEV. LKG. NORTH



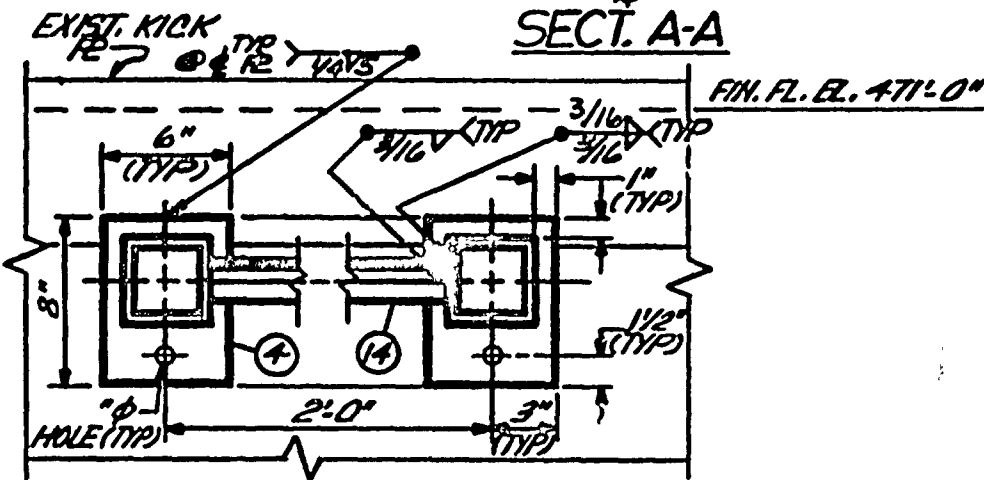
SECT. B-B

2	REVISED IN ACCORDANCE W/WBG STATUS AS-BUILT	6-2-80	A	MC	SH
1	REDESIGNED BY REP	3-7-80	ES	RP	KP
REV NO	REVISION	DATE	DWN	CHKD	APVD
DWN B. SANDERS	CHKD JH	DATE 3-7-80	SCALE NONE	MARK NO. SW-77	
SL/LDE PORTER	DATE 3-7-80				
ENGINEERING REVIEW			BURNS AND ROE, INC.		
MECH	CIVIL		Engineers and Constructors		
ELEC			New Jersey • New York • Connecticut • California		
REVD	DATE		W.O.3808	DWG. SW-77 SHT. 2 OF 3	REV 2
CHKD DRAFTSMAN	CHIEF	ENGINEER			





SECT. A-A



SECT. C-C

INFORMATION ONLY

2	REVISED IN ACCORDANCE W/ WBG STATUS AS-BUILT	5-2-81	AP	MCD	JH	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2	
1	REDESIGNED BY B&R	3-8-80	AP	KP			
REV NO	REVISION	DATE	CHKD	APVD		MARK NO. SW-77	
DWN B. SANDERS	CHKD JH	BTM	SCALE	NONE		BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California	
SL/LOE PORTER	YTW	DATE 3-7-80					
ENGINEERING REVIEW						W.O.3808 DWG. SW-77 SH. 3 OF 3	
MECH	CIVIL						
ELEC						REV 2	
REVD							
L.R. 9/22/80	CHKD	DATE 3-13-80	TECH.	ENGINEER			
CHIEF DRAFTSMAN	CHIEF						





DR. APP.	DESCRIPTION	DATE	REV.	NO.	REQ'D	SIZE	DESCRIPTION	ASTM	WT.
1	4	1" x 9" x 1'-4" LG	PAD	A-36	164				
2	1	5/8" x 2" x 6" LG	PAD	A-36	2				
3	3	9/16" x 2" x 6" LG	PAD	A-36	6				
TOTAL									172 #

**SECTION A-A**

**SECTION A-A**

ZONE: R-13

ELEV LOOKING EAST

PS 6 24" STD SCH

WTB# 32

SECT A-A

ZONE: R-13

**LOCATION PLAN**

LOCATION PLAN

BCSR-215-1532-7286

83

SEIS. LOADS: VERT ± 7195 N-S ~ E-W ± 16037		OP. ID. # 3325		HYDRO. ID. # ~		THERM. ID. # VERT ± 2000	
CODE / CLASS: III-3		GROUP: 2		Q.A. LEVEL: I		PAINT: I	
OWNER WASHINGTON PUBLIC POWER SUPPLY SYSTEM		ENGINEER BURNS & ROE INC.		PIPING SYSTEM STANDBY SERVICE WATER		ISO: SW-251-23.29 H	
PROJECT HANFORD NO. 2		CUSTOMER BOVEE & CRAIL / GERI		REFERENCE DWG. PIPE: M-711		STEEL CALC 8.16.248	
DATE 4-26-81		REV. 1		MARK NO. SW-34		AM-149	

**nps industries, inc.**

CONTRACT 215

a subsidiary of

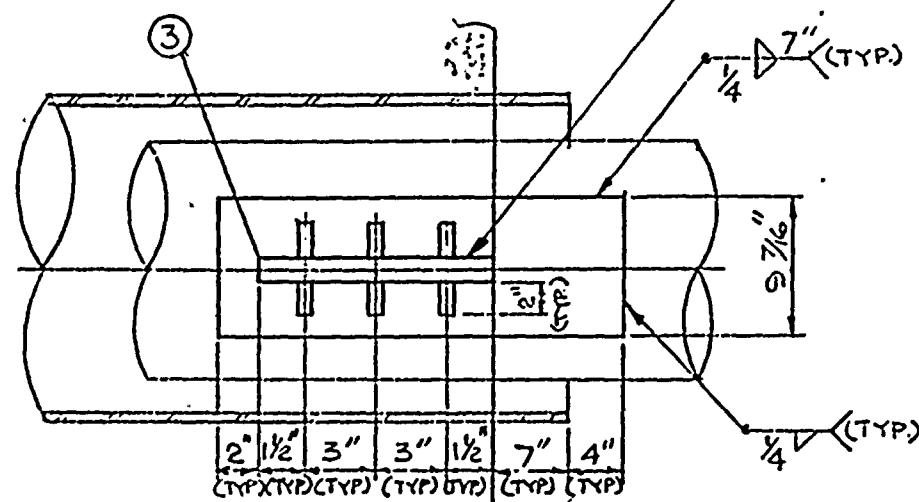
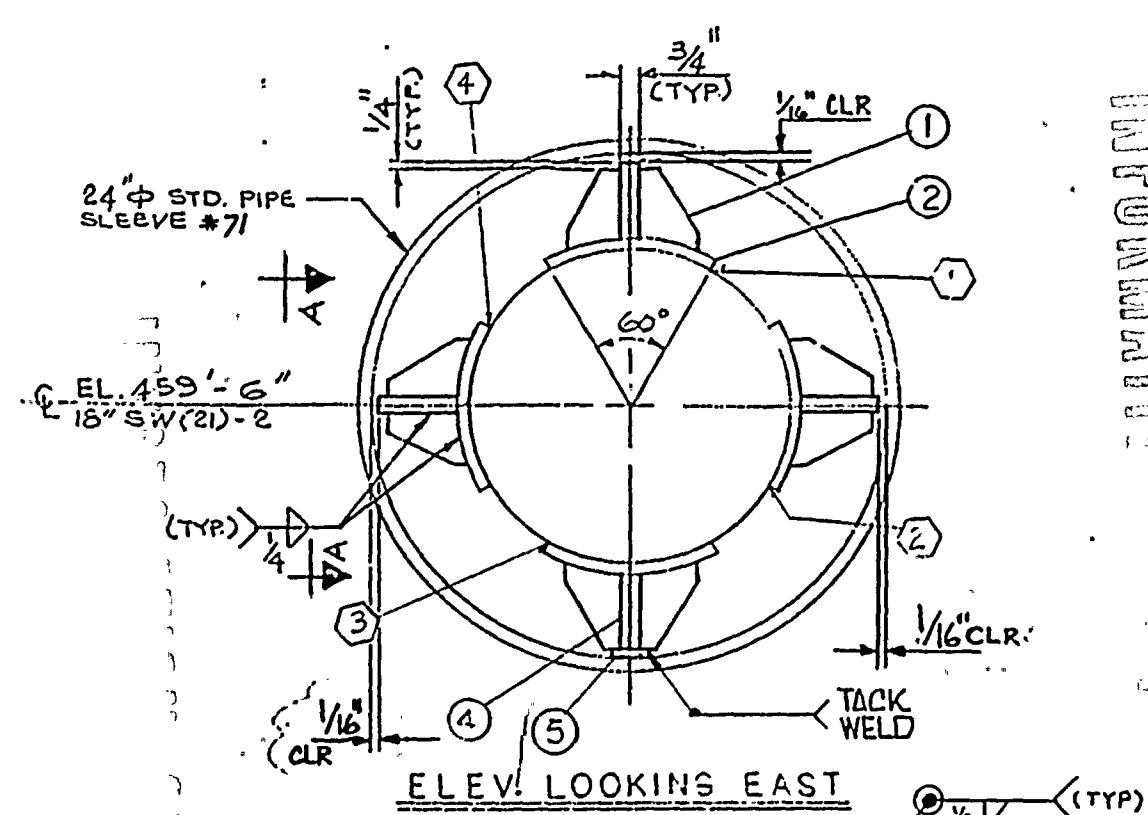
**nuclear power services, inc.**

26 broadway-new york, n.y. 10004

DRAWN	DATE	CHK'D	DATE	APPV'D	DATE
JS	8/21/76	JS	9/25/76	JG	4-28-77
JOB NO.					
DWG. NO. SW-34					
REV. 1					



INFORMATION



215-03	CVI	NO
11004	CVI	SHT
2	REV	SHT
1	DWG	SHT

OWNER <b>WASHINGTON PUBLIC POWER SUPPLY SYSTEM</b>		PIPING SYSTEM <b>SERVICE WATER</b>	
PROJECT <b>HANFORD NO.2</b>		ISO.: SW-296-17.26 H	
ENGINEER <b>BURNS &amp; ROE INC.</b>		REFERENCE DWG. <b>PIPING: M 712</b>	
CUSTOMER <b>BOVEE &amp; CHAIL/GERI</b>		58.0 PS	
MARK NO. <b>SV-142</b>		<b>AM-184</b>	



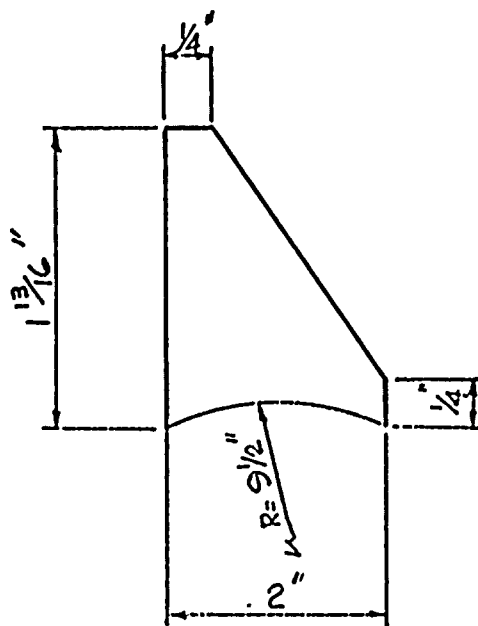
**nps industries, inc.**  
CONTRACT 215

a subsidiary of  
**nuclear power services, inc.**  
26 broadway-new york, n.y. 10004

DRAWN	DATE	CHK'D.	DATE	APPV'D.	DATE
MF	9-28-76	JJM	10-1-76	JG	11-4-77
JOB NO.					
DWG NO. SW-142 (1 OF 2)				REV. 2	

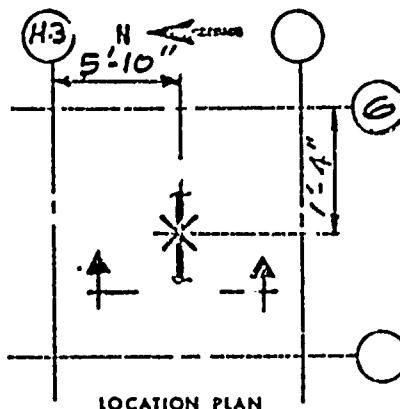
BCBR-215-1493



[illegible]

DETAIL 1

**LOCATION PLAN FOR  
REFERENCE ONLY!  
USE FAB. ISO. FOR  
CONSTRUCTION**



ACAR-219-1293-

ZONE: R-21

SEIS. LOADS: VERT =  $\pm 615$  # N-S =  $\pm 10,247$  # -W = .

OP. ID. # 3395 \*

HYDRO. ID. = —

THERM. I.D. 3

CODE / CLASS: III/3

GROUP: 2

Q.A. LEVEL: I

PAINT: I

PIPE CALC. 8.42.243

STEEL CAL: 8.16, 2393

**OWNER**

# WASHINGTON PUBLIC POWER SUPPLY SYSTEM

PROJECT  
HANFORD NO.2

**ENGINEER  
BURNS & ROE INC.**

**CUSTOMER**

BOVEE &amp; CRAIL / GERI

PIPING SYSTEM SERVICE WATER

ISO.: SW-296-1726 H

REFERENCE DWG. PIANG: M 712  
58.0 P

58.0 P3

MARK NO. SW-142

AM-184



nps industries. inc.

**CONTRACT 215**

a subsidiary of  
nuclear power services, inc.  
25 broadway new york, n.y. 10004

DRAWN	DATE	CHK'D	DATE	APPV'D	DATE
MF PL	9.28.76	JJM	10.1.76	JG	11.1.76
JOB NO.					
DWG. NO. SW-142 (2 OF 2) FLV 2					





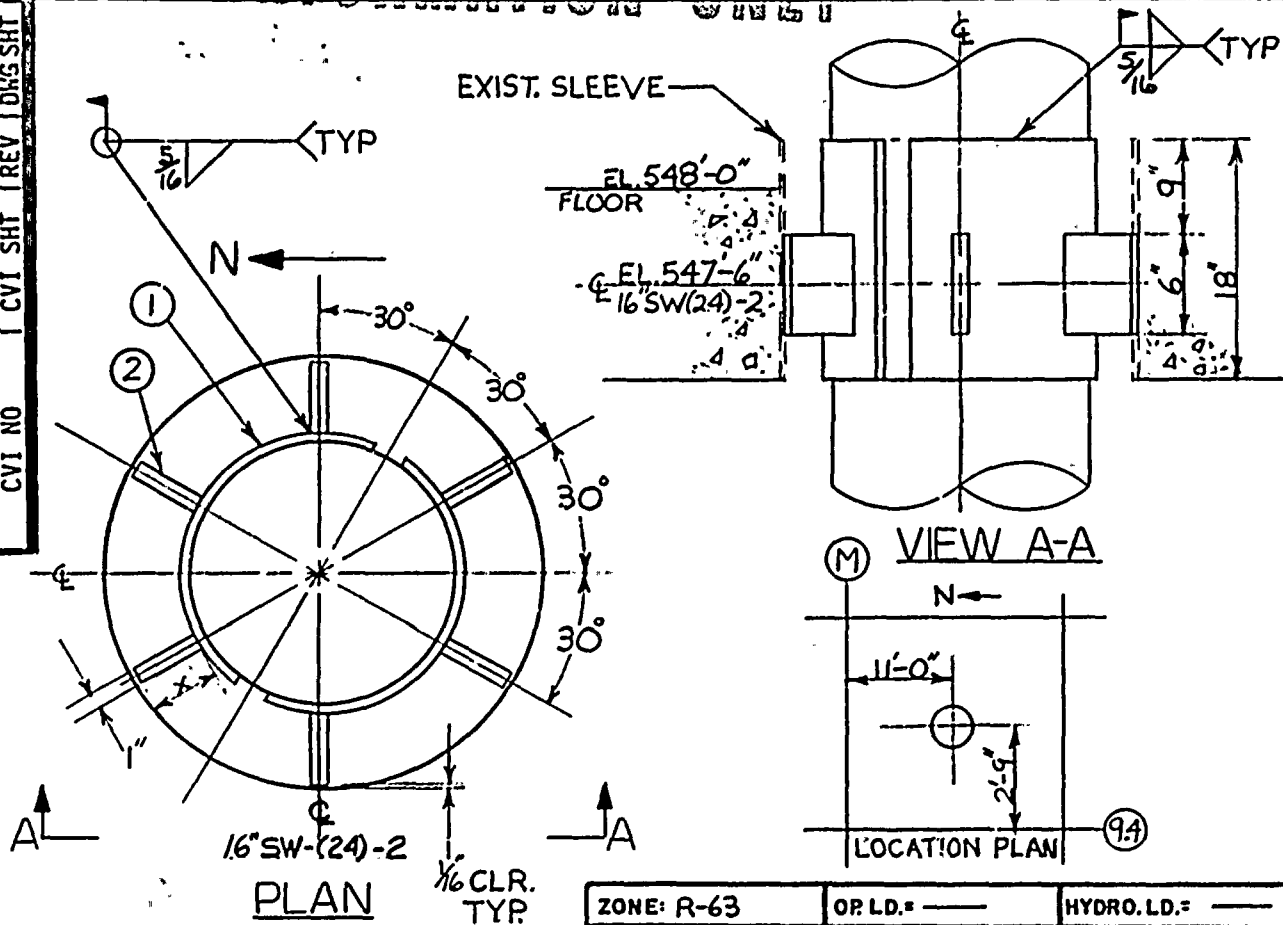




ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.	
1	2	R 3/4" X 18" X 2'-2" BEND TO 8 3/4" RADIUS	A-36	198.9	
2	6	R 1" X 2 1/2" X 0'-6" FIELD CUT X DIMENSION TO SUIT.	A-36	25.5	
TOTAL				224.4#	

INFORMATION ONLY  
INFORMATION ONLY

02-215-08	CVI NO	CVI SHT	REV	DWG SHT
13/53				



ZONE: R-63	OP. LD. =	HYDRO. LD. =
THERMAL LD.: VERT. =	N-S =	E-W =
SEISMIC LD.: VERT. =	N-S = ±17876"	E-W = ±11367"
CODE/CLASS: III/3	GROUP: II	QA LEVEL: I
PAINT: I	PIPE CALC.: P145B	STL. CALC.: 8/6/232
PIPING STAND-BY SYSTEM SERVICE H <sub>2</sub> O	REF. DWG. ISO.: SW-295-4.6 H	PIPING: M-715
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO. 2		
MARK NO. SW-916N		
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
W.O. 3808	DWG. SW-916N	REV 1

1	REVISED IN ACCORDANCE W/WBG STATUS AS-BUILT	A/12/81	IM	APD
REV NO	REVISION	DATE	OWN	APVD
OWN D. MEHEEN	CHKD F. S. / 10	SCALE N.T.S.		
SL/LDE - 10/1/81	DATE 7/22/80			
ENGINEERING REVIEW				
MECH	CIVIL			
ELEC				
REVD	APVD	DATE		
H.R. T. / 11/1/81	B. B. / 7/22/80			
CHIEF DRAFTSMAN	CHIEF MECH. ENGINEER			







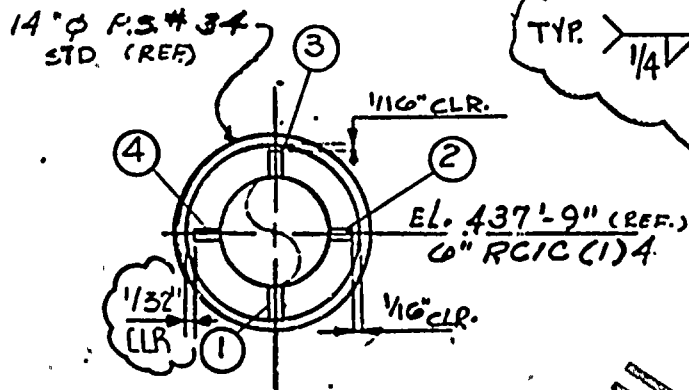
1  
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101

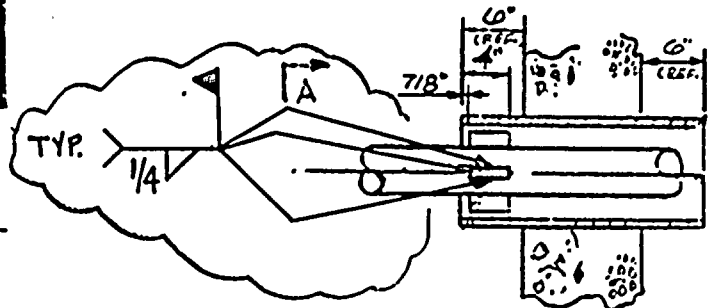
SUPERIOR REFRACTORIES, INC.

ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	1	R 1/2" X 4" X 0'-2 5/8" (LUG)	A-36	1
2	1	R 1/2" X 4" X 0'-3 1/8" (LUG)	A-36	2
3	1	R 1/2" X 4" X 0'-3 7/8" (LUG)	A-36	2
4	1	R 1/2" X 4" X 0'-3 3/16" (LUG)	A-36	2
TOTAL WT.				7#

02-	215 - 08	5847	4
CVI NO	CVI SHT	REV	DWG SHT

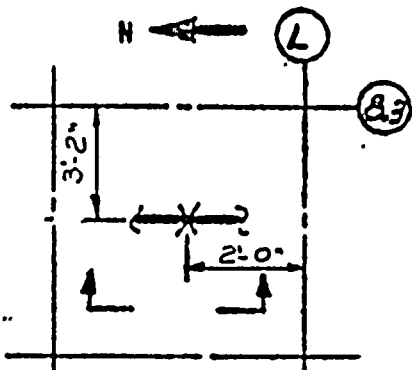


SECTION A-A



ELEVATION LKG. EAST

LOCATION PLAN FOR  
REF ONLY! SEE ISO FOR  
HANGER ALONG PIPE.



LOCATION PLAN

B & R ISO M-200-115  
DATA PT. # 44 PS 16(4) & PS 17(2)

INFORMATION ONLY

THERM MVTs	
DX	0.303" S
DY	.000"
DZ	.000"

ZONE: R- II	OR LD. = $FY = -741\#$ $FZ = 12\#$	HYDRO. LD. = -692#
THERMAL LD.: VERT = 118# N-S = — E-W = 203#		
SEISMIC LD.: VERT = ±198# N-S = — E-W = ±243#		
CODE/CLASS: III/2	GROUP: I	Q.A. LEVEL: I
PAINT: I	PIPE CALQ. 8.14.966	STL. CALC. 8-15-2142
PIPING REACTOR CORE SYSTEM INSULATION COOLING	REF. DWG. ISO. RCIC-659-7-10 PIPING M-711	

4	1/2/85	CONSTR. COMPLETE PER PRR REV. 1 & PED 215-H-6883.	RA B	ERB	JPR
REV	DATE	REMARKS	D/END	DS	ENG.
DWNAL: Sanjivani			CHKD: JH (check)		
BL/LOE			DATE 6-21-82		
ENGINEERING REVIEW					
MECH			CIVIL		
ELEC					
REV			DATE 7/21/82		
J. R. Tejedor			J. R. Tejedor		
CHIEF DRAFTSMAN			CHIEF MECH. ENGINEER		

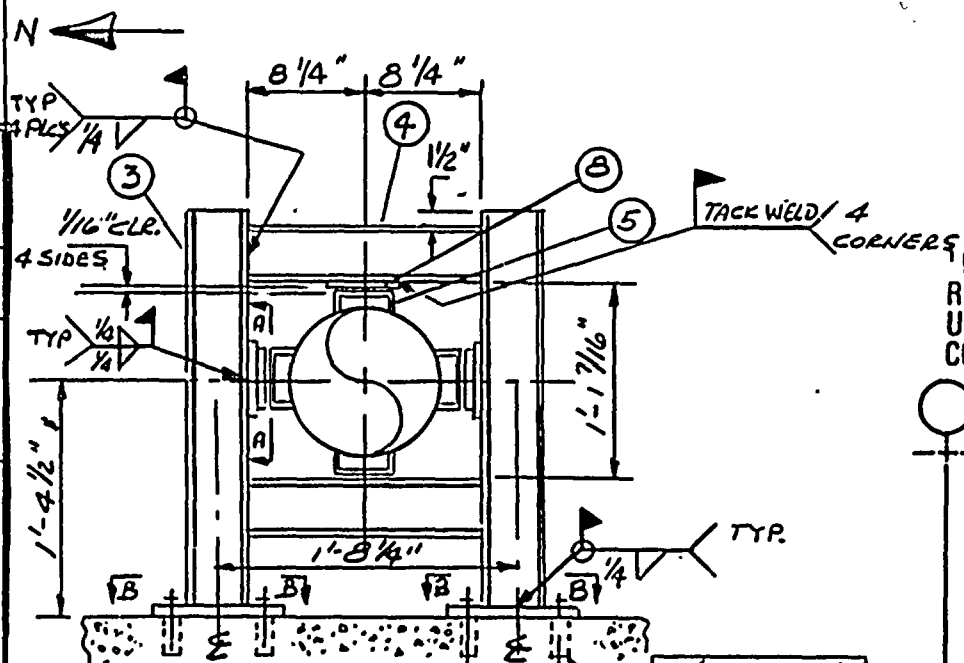
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO. 2 006.0-P2		
MARK NO. RCIC-18		
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
W.O.3808	DWG. RCIC-18	REV 4



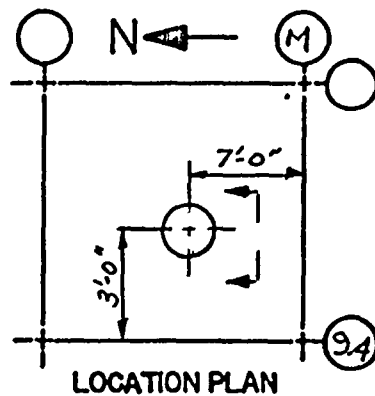
SUPERIOR REPROGRAPHICS 18-354

ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	WT.
1	2	PL 3/4" X 9" X 0'-3" (SEE SECT. B-B)	A-36	34
2	8	5/8" Ø X 1 1/2" LG. H.V. HEX. MACH. BOLT W/L.W.	A-307 Gr B	2
3	2	M 4 X 13 X 2'-4 3/4" LG.	A-36	62
4	2	M 4 X 13 X 1'-4 1/2" LG.	A-36	35
5	4	C 4 X 5.4 X 0'-6" LG (CUT TO SUIT)	A-36	11
6	2	PL 3/4" X 5" X 0'-5"	A-36	11
7	2	PL 3/4" X 4" X 0'-4"	A-36	7
8	1	PL 1/16" X 4" X 0'-5"	A-36	1
TOTAL WGT.				163#

02-	215-23	11402	2	1
CVI NO	CVI SHT	REV	DWG SHT	



LOCATION PLAN FOR  
REFERENCE ONLY  
USE FAB. ISO. FOR  
CONSTRUCTION



(8) 5/8" Ø HILTI DROP-IN CONC.  
FAST. HDI-5/8 (BY FIELD)

PLAN @ ELEV. 470'-1"

THERM MVT'S	
DX	0.0"
DY	0.155" UP
DZ	0.0

BER 150 M200 SHT 173  
DATA PT. 116 PS15 (X), PS16 (Z)

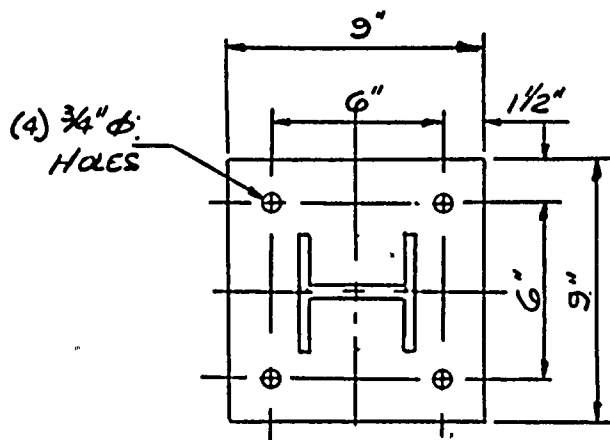
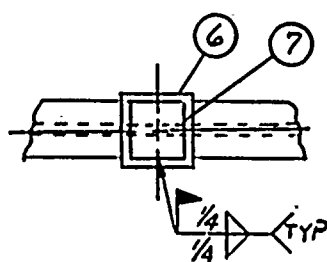
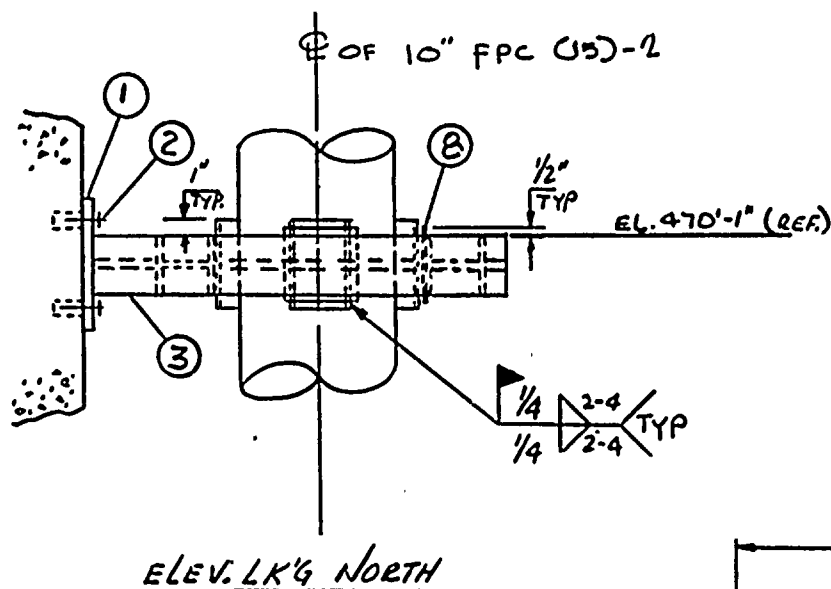
2	RELEASE FOR PSI INSPECTION	3/23/82	1/10/82
1	REV'D REDRAWN BY BER IN ACCORD W/ NRG STATUS AS-BUILT	11/1/82	1/10/82
REV NO	REVISION	DATE	OWN/CHKD/APVD
OWN	R.E. Keck Sr.	CHKD	J. GRIFFIN
SL/IDE	(P)	DATE	12-2-82
ENGINEERING REVIEW			
MECH		CIVIL	
ELEC			
REV'D	A.L. Foye	APVD	J. Griffin
DATE	12/14/82	DATE	12/7/82
MANAGER-DESIGN AND DRAFTING		CHIEF MECH.	ENGINEER

ZONE: R-23	OR LD.: FZ-5'	HYDRO. LD.: FZ-5'
THERMAL LD.: VERT: —	N-S:	E-W= 3'
SEISMIC LD.: VERT: —	N-S= 524'	E-W= 504'
CODE/CLASS: IV/3	GROUP: 1	Q.A. LEVEL: I
PAINT: I	PIPE CALC.: 8/14/54	STL. CALC.: 8/16/325
PIPING FUEL POOL SYSTEM COOLING	REF. DWG. ISO.: FPC-605-5.9 PIPING: M-701	
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2 009.0-P4		
MARK NO. FPC-64		
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
W.O.3808	DWG. FPC-64 SNT. 1 of 2	REV 2





INFORMATION ONLY



2	RELEASE FOR P.S.I. INSPECTION	3/1/82	12/1/82	12/1/82
1	REV'D & REDRAWN BY B&R IN ACCORD W/ WBG STATUS AS BUILT	12/1/82	12/1/82	12/1/82
REV NO	REVISION	DATE	OWNR	APVD
DWN R.E. KECK SR.	CHKD J. GRIFFIN	SCALE N.T.S.	DATE 12-2-82	MARK NO. FPC-64
MECH	CIVIL	ENGINEERING REVIEW		
ELEC	CIVIL	BURNS AND ROE, INC.		
REVD	APVD	Engineers and Constructors		
MANAGER-DESIGN AND DRAFTING	CHIEF MECH.	New Jersey • New York • Connecticut • California		
12/1/82	12/1/82	12/1/82	12/1/82	12/1/82
W.O.3808	DWG. FPC-64	SHT 2 OF 2		
REV	2			

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2

MARK NO. FPC-64

BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O.3808

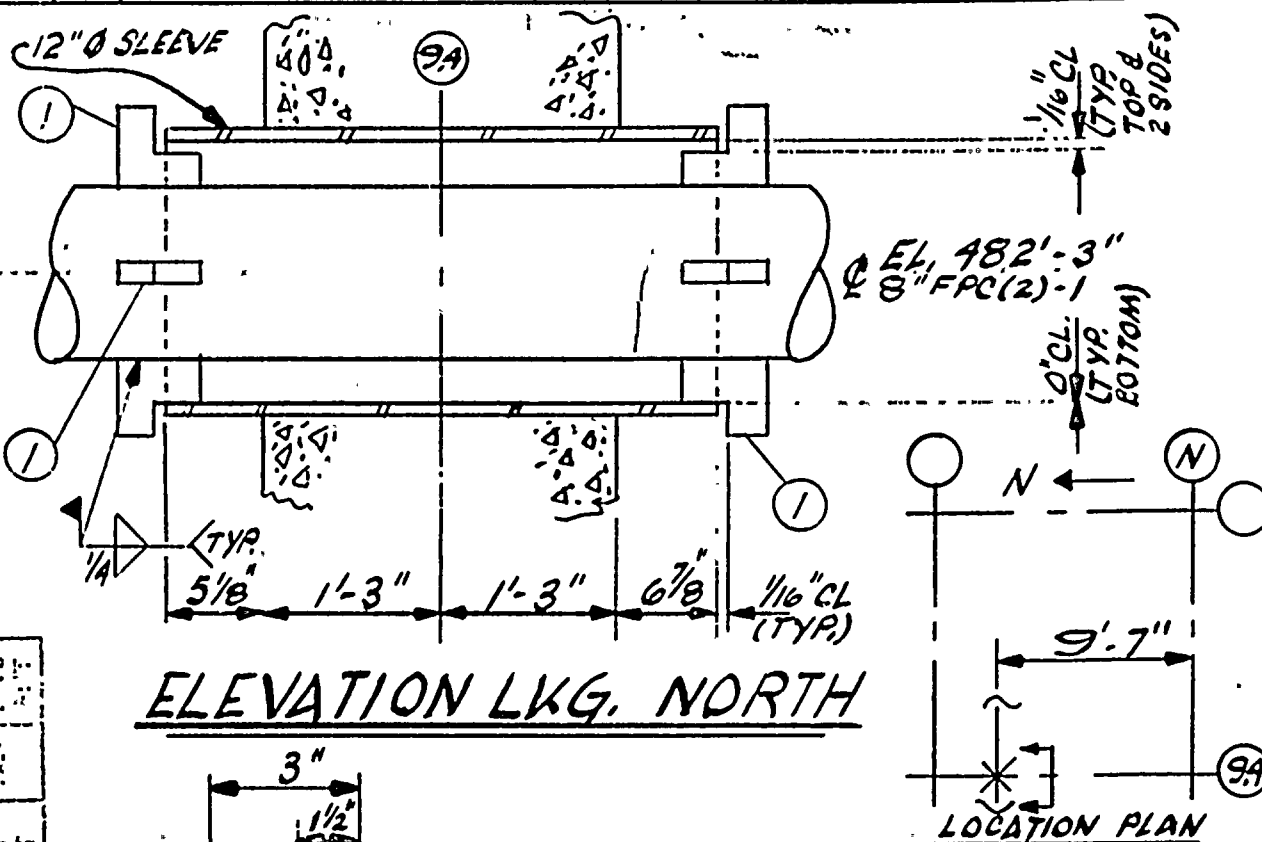
DWG. FPC-64

SHT 2 OF 2

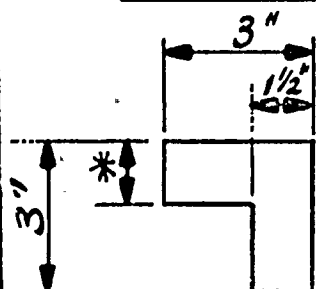
REV  
2



ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	8	2 1/2" x 3" x 0'-3" (SEE DETAIL 1)	A36	10.2
INFORMATION ONLY				
			TOTAL	10.2



ELEVATION LKG. NORTH



DETAIL 1

\* CUT TO SUIT

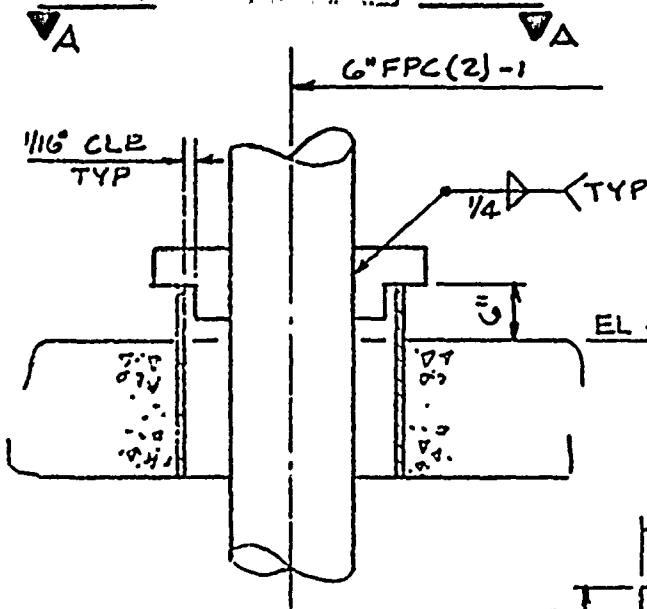
1	INCORPORATED PED-H-A 284	4-22/1981	APV
REV NO	REVISION	DATE	CHKD APVL
DWN M. GWINN	CHKD	SCALE N.T.S.	
SL/LOE	DATE 7/17/81		
ENGINEERING REVIEW			
MECH	CIVIL		
ELEC			
REVD	APV	DATE	
H.R. Frazier	7/18/81		
CHIEF DRAFTER	CHIEF MECH.	ENGINEER	

ZONE: R-33	OP. LD.: 220 #	HYDRO. LD.: 220 #
THERMAL LD.: VERT=	N-S=	E-W=
SEISMIC LD.: VERT= ±438 #	N-S= ±600 #	E-W= ±972 #
CODE/CLASS: III/3	GROUP: 2	Q.A. LEVEL: I
PAINT: I	PIPE CALC.: P-155/	STL. CALC.: 8/16.1116
PIPING FUEL POOL SYSTEM COOLING	REF. DWG. ISO.: FPC-636-16.21	PIPING: M 703
WASHINGTON PUBLIC POWER SUPPLY SYSTEM		
HANFORD NO. 2		
MARK NO. FPC-98 1-10-83		
BURNS AND ROE, INC.		
Engineers and Constructors		
New Jersey • New York • Connecticut • California		
W.O. 3808	DWG. FPC-98	REV 1



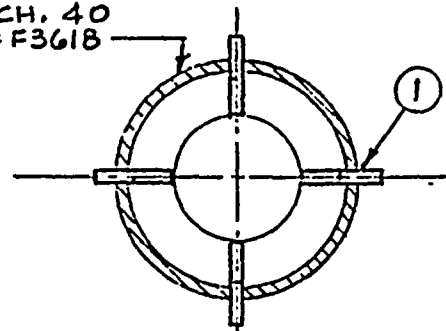
ITEM NO.	NO. REQD.	SIZE	DESCRIPTION	ASTM	WT.
1	4	1 1/2" x 1 1/2" x 3/8" THK	LUG.	A36	1.1
MATERIAL WT A=1.1 B=0 C=0 TOTAL 1.1					

02. 215-03  
 4971  
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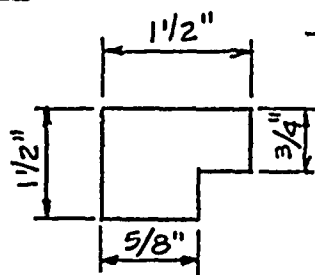


ELEVATION

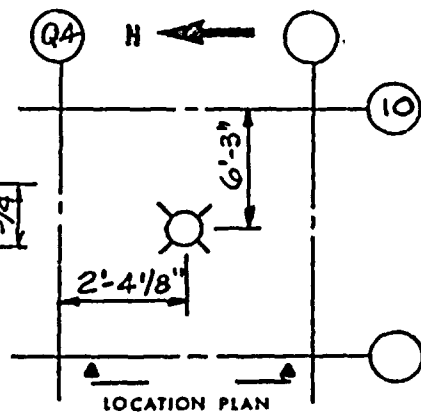
8" Ø SCH. 40  
P.S. # F3618



SECTION A-A



DETAIL 1



LOCATION PLAN

ZONE: W-34

SEIS. LOADS: VERT = ± 94 # N-S = ± 139 # E-W = ± 67 #		OP. LD. = 478 #	HYDRO. LD. = 478 #	THERM. LD. = —
CODE / CLASS: III/3	GROUP: 4	Q.A. LEVEL: I	PAINT: I	PIPE CALC. P-1514
OWNER WASHINGTON PUBLIC POWER SUPPLY SYSTEM		PIPING SYSTEM: FUEL POOL COOLING		
PROJECT HANFORD NO. 2		150. FPC-G36-25.26 H		
ENGINEER BURNS & ROE INC.		REFERENCE DWG. PIPING M-762		
CUSTOMER BOVEE & CRAIL / GERI		MARK NO. FPC-114		AM-83



nps industries, inc.

CONTRACT 215

a subsidiary of  
nuclear power services, inc.  
25 broadway new york, n.y. 10004

DRAWN	DATE	CHK'D	DATE	APP'D	DATE
LD	3/1/76	HB	9-4-76	✓	10/26/76
JOB NO.					
DWG. NO. FPC-114				REV 0	

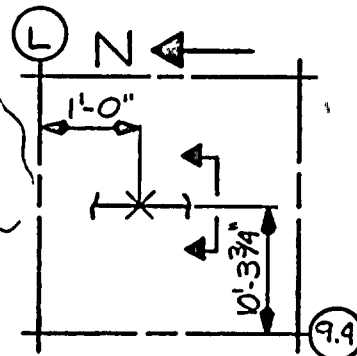
BCR-715-1596-7350-1



ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1		DELETED		
2		DELETED		
3		DELETED		
4	1	R 3/8" x 2" x 0'-5"	A-36	1
5		DELETED		
6		DELETED		
7		DELETED		
8		DELETED		
9		DELETED		
10		DELETED		
11		DELETED		
12		DELETED		
13	8	3/4" Ø (2 1/4" LG) HVY. HEX MACH. BOLT W/LW	A307GRB	(5)
14		DELETED		
15		DELETED		
16	1	TS 5x5x.375 (x 10'-7 1/2") LG	A500GRB	(233)
17	1	TS 4x4x.375 (x 1'-3") LG	A500GRB	(21)
18	2	R 3/4" x 9" x 0'-11"	A-36	42
19	1	R 1" (x 17 1/2" x 3'-4") (SEE SECT 12.Y)-D1	A-36	(191)
20	1	TS 6x4x.375 (x 8'-0 1/8") LG	A500GRB	(176)
21	1	TS 3x3x.375 (x 0'-9 1/2") LG	A500GRB	17

# INFORMATION

LOCATION PLAN FOR  
REFERENCE ONLY!  
USE FAB/ISO. FOR  
CONSTRUCTION



02-215-08	11717	3	
CVI Number	CVI Sheet	Rev.	Dwg. Sheet

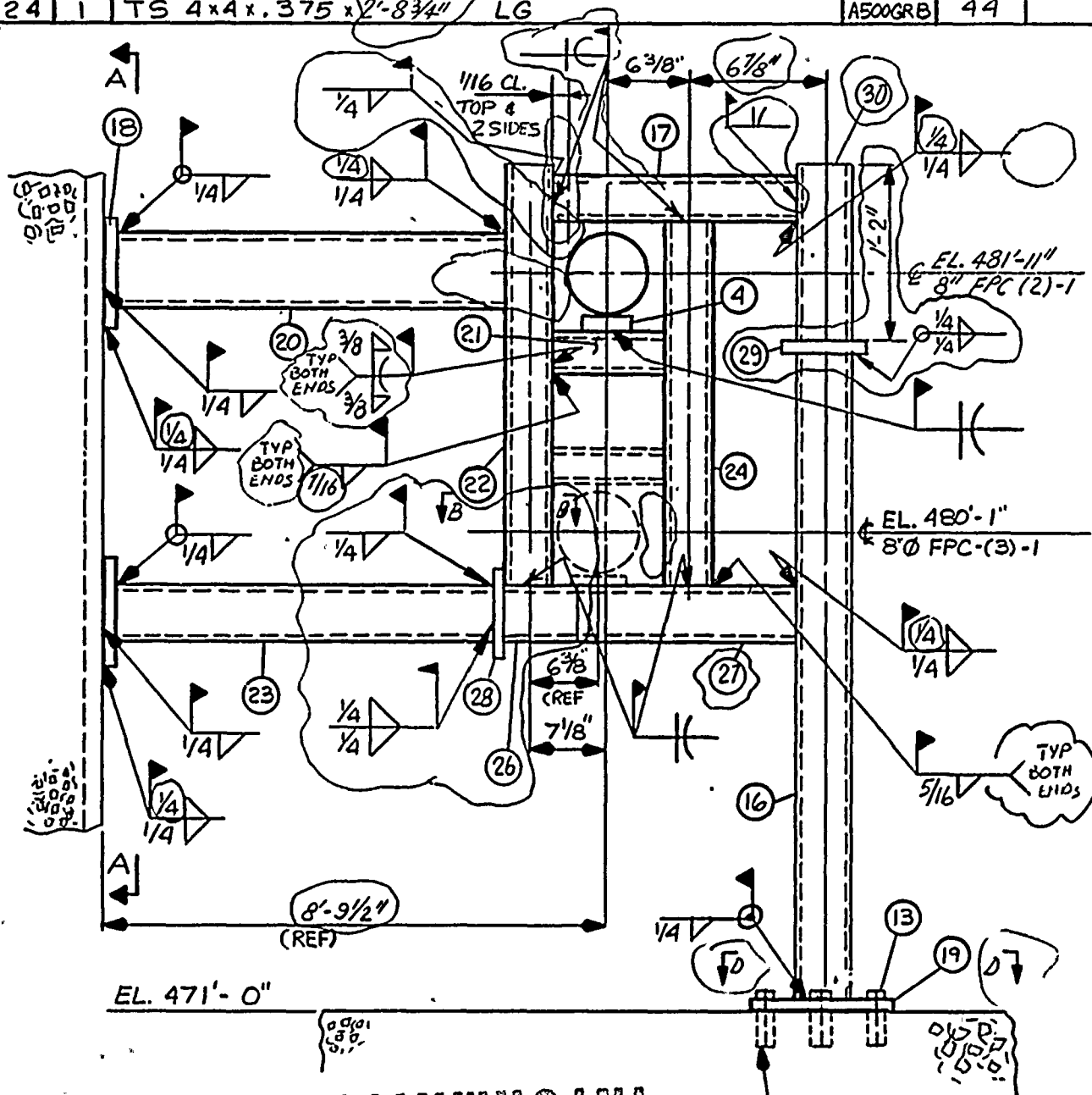
LOCATION PLAN

3		RELEASE FOR P.S.I. INSPECTION, INCORP. 120- H-B260, H-B630, H-E530, H-M095		4/14/83	11/20
REV NO	REVISION	DATE	DWN	CHKD	APVD
DWN	CHKD	DATE	SCALE	NTS	
SL/IDE	DATE	9/9/81			
ENGINEERING REVIEW					
MECH		CIVIL			
ELEC					
REVD		APVD		DATE	
H.R. Tregue		J.B. Tregue		9/14/81	
CHIEF DRAFTSMAN		CHIEF MECH.		ENGINEER	
ZONE: R-33		OR LD. $F_1 = 1442^{\circ}$ $F_2 = 101^{\circ}$		HYDRO. LD. $\pm 1604^{\circ}$	
THERMAL LD. VERT =		N-S =		E-W =	
SEISMIC LD. VERT =		±798° N-S =		E-W = ±1604°	
CODE/CLASS III/3		GROUP: 2		Q.A. LEVEL: I	
PAINT: I		PIPE CALC: 8.42.035		STL CALC: 8.16.1638	
PIPING FUEL POOL SYSTEM COOLING		REF. DWG. ISO: FPC-636-14.15H PIPING: M 703			
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2					
MARK NO. FPC-203					
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California					
W.O.3808		DWG. FPC-203		SH. 10A4	
				REV 3	





22	1	TS 4x4x.375x(3'-1 1/4" LG	A500GRB	51	
23	1	TS 5x5x.375x(7'-10 3/4" LG	A500GRB	173	
24	1	TS 4x4x.375x(2'-8 3/4" LG	A500GRB	44	

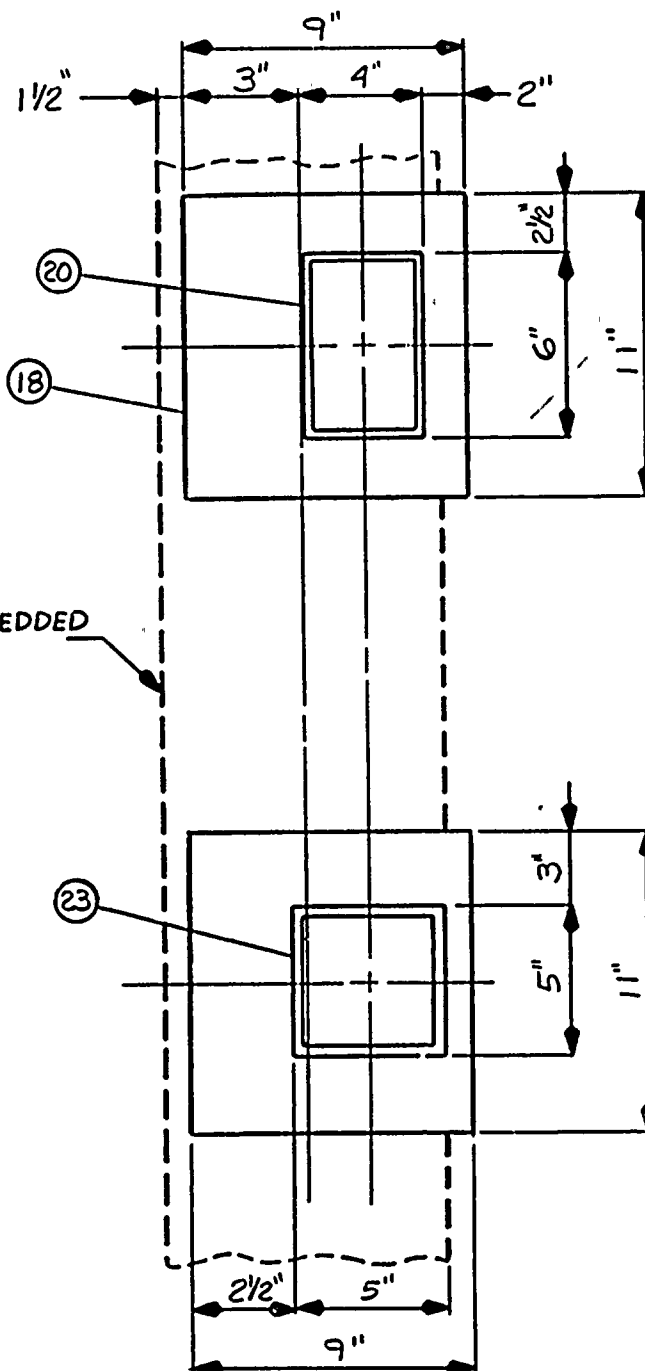


LYNO NOILWAKOIN  
ELEV. LKG. NORTH

(8) 3/4" HILTI DROP-IN  
CONC. FAST. HDI-3/4"  
(BY FIELD)

3	RELEASE FOR P.S.I. INSPECTION, INCORP. P20 H-B260, H-B680, H-E580 & H-M095		1/14/83	TH FB		(BY FIELD)	
	REV NO	REVISION	DATE	OWN CHKD	APVD	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2	
OWN	CHKD	SCALE	NTS		MARK NO. FPC-203		
SLDGE	DATE	9/5/81		BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California			
ENGINEERING REVIEW							
MECH	CIVIL						
ELEC							
REVD	APVD	DATE	W.O.3808		DWG. FPC-203		REV 3
H.R. Trogue CHIEF DRAFTSMAN	J.B. [Signature] CHIEF MECH	4/11/81 ENGINEER			SH. 20A4		





EXISTING EMBEDDED  
STRIP PLATE

INFORMATION ONLY

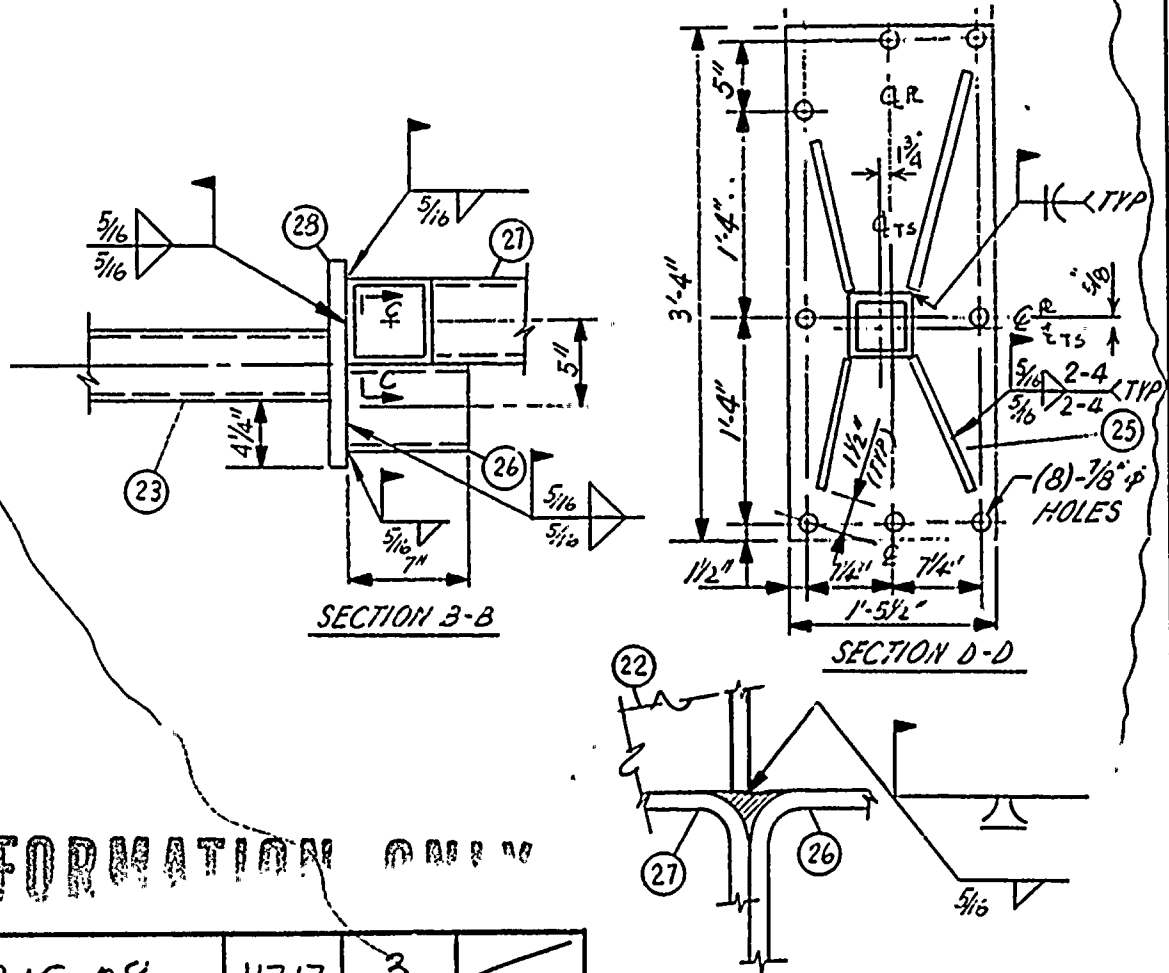
FRAME 1 OF 2  
SECTION A-A

3	RELEASE FOR P.S.I. INSPECTION, INCORP. PED- H-8260, H-8680, H-E580 H-A1025	1/19/83	1/19/83	
REV NO	REVISION	DATE	DWN	APVD
1	CHKD <i>ent</i>	SCALE	NTS	
SLKDE	DATE	9/9/81		
ENGINEERING REVIEW				
MECH	CIVIL			
ELEC				
REV'D	APVD	DATE		
<i>L.E. Torgin</i>	<i>Sh. D. Long</i>	9/14/81		
CHIEF DRAFTSMAN	CHIEF MECH.	ENGINEER		
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2			MARK NO. FPC-203	
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California				
W.0.3808			DWG. FPC-203	REV 3



SUPERIOR REPROGRAPHICS 78 397

ITEM NO.	QUAN	SIZE-DESCRIPTION	ASTN	WT.
25	4	R 3/4" x 2 1/8" x 1'-3" (CUT TO SUIT)	A36	51
26	1	TS 5 x 5 x .375 x 0'-7" LG.	A500 Gr. B	13
27	1	TS 5 x 5 x .375 x 1'-7 3/4" LG.	A500 Gr. B	37
28	4	R 3/4" x 7" x 1'-1"	A36	71
29	1	R 3/4" x 6" x 0'-6"	A36	7
30	1	TS 5 x 5 x .375 x 1'-2" LG.	A500 Gr. B	26
TOTAL WT.				1153#



INFORMATION ONLY

02-215-08	11717	3	
CVI Number	CVI Sheet	Rev.	Dwg. Sheet

SECTION C-C  
(WELD DETAIL)

3	RELEASE FOR P.S.I. INSPECTION, INCORPORATED		4/14/83	TH	PB
	H-B260, H-B630, H-F530 H-M095				
REV NO	REVISION	DATE	OWN	CHKD	APVD
OWN	TRAVIN	CHKD	BOYANICK	SCALE	NTS
SL/LDE	DATE	ENGINEERING REVIEW			
MECH	CIVIL				
ELEC					
REVD	APVD	DATE			
MANAGER-DESIGN AND DRAFTING	CHIEF	MECH.	ENGINEER		

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2

MARK NO.

FPC-203

BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O.3808

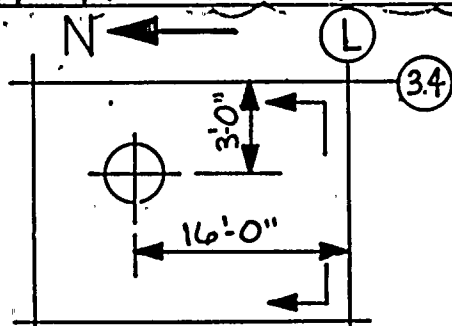
DWG. FPC-203

SH. 4 OF 4 REV 3

FRAME 2 OF 2



ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	1	R 1" x 39 1/2" x 3'-13 1/4" (SEE DETAIL 1)	A36	422
2		DELETED		
3	2	TS 5 x 5 x .375 x 3'-5 3/8"	A500GRB	151
4	2	TS 5 x 5 x .375 x 1'-10 1/2"	A500GRB	82
5	2	TS 5 x 5 x .375 x 3'-0"	A500GRB	66
6	2	TS 5 x 5 x .375 x 2'-4 3/4"	A500GRB	105
7	2	14" Ø x 0'-6 1/4" LG STD PIPE TRUNKION (COPE TO SUIT)	A106GRB	57
8	2	R 1/2" x 16" x 1'-4"	A36	72
9	2	R 3/8" x 2 1/2" x 2'-4" (BEND TO RADIUS OF 16" Ø PIPE)	A36	125
10	12	3/4" Ø x 2 5/8" LG HVY HEX MACH BOLT W/LW	A307GRB	6
11		DELETED		
12	2	STIFF R 1/2" x 3" x 2'-7" (SEE DETAIL-12)	A36	26
13	2	STIFF R 1/2" x (3" x 1'-2"	A36	10
14	8	WASHER R 1/4" x 2 1/2" x 0'-2 1/2"	A36	4
15	8	3/4" Ø x 2 5/8" LG HVY HEX MACH BOLT W/LW	A307GRB	4
16	2	R 1" x 4" x 2'-5 1/2" (SEE DETAIL-1) (CUT TO SUIT)	A36	76
17	9	R 1" x 4" x 0'-4 5/8" (SEE DETAIL-1)	A36	3
18	4	R 3/4" x 3" x 1'-2 1/2" (SEE SECTION A-A) H/D	A36	8
19	4	R 3/4" x 3" x 1'-4" (SEE SECTION A-A) H/D	A36	3
20		DELETED		
21	1	R 1" x 4" x 0'-7" (SEE DETAIL-1)	A36	8
22	1	R 1" x 4" x 0'-5" (SEE DETAIL-1)	A36	6
23	2	R 1" x 16" x 2'-0" (SEE SECTION A-A)	A36	77
24	18	5/8" Ø x 2" LG. HVY. HEX MACH. BOLT W/LW		
25	2	1" Ø HSKB W/LW		



LOC. PLAN FOR REF.  
ONLY! SEE ISO FOR  
HANGER LOC. ALONG  
PIPE.

02-215	-08	9795	6	
CVI NO	CVI SHT	REV	DWG SHT	

B.R. ISO. M200-13 DATA PT.  
76 ANCHOR (X, Y, Z)

#### LOCATION PLAN

	MOMENT 16'-ft		
	MX	MY	MZ
THERM	-5656 +3330	-91 54	6186 3-40
DEADWT	-944	-281	1270
SEISMIC	±28175	±2978	±29956

ZONE: R-32	OP. LD.	HYDRO. LD. =
THERMAL LD. VERT. 597#	Fy-900	352' N-S 845#
SEISMIC LD. VERT. ±2881#	Fz-900	497' E-W 390#
CODE/CLASS III	GROUP 1	Q.A. LEVEL I
PAINT: I	PIPE CALC. 8.14.69	STL. CALC. 8.16.450
PIPING LOW PRESSURE	REF. DWG. ISO. LPCS-750-11.15	PIPING M701 THRU M705
SYSTEM CORE SPRAY		

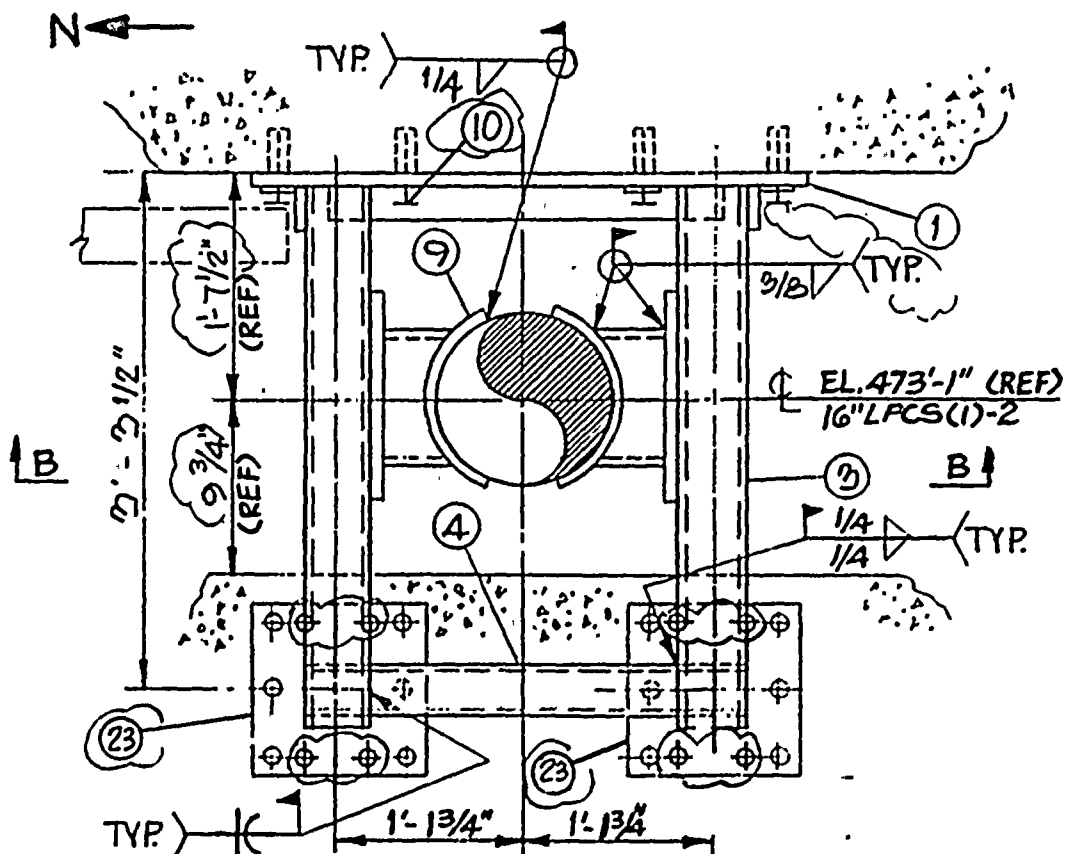
6	5/2/85	CONSTR. COMPLETE PER PRR REV. 7 (SFI) & PED 215-H-E158, E129 & K872.	RA	ERB	JMR
---	--------	---	----	-----	-----

REV NO	DATE	REMARKS	DESIGN	DS	ENG.
DWN RAKoch	CHKD	SCALE NTS			
SL/DE	DATE 8-5-85				
ENGINEERING REVIEW					
MECH		CIVIL			
ELEC					
REVD	DATE	APVD	DATE		
CHIEF DRAFTSMAN	CHIEF	MECH.	ENGINEER		

WASHINGTON PUBLIC POWER SUPPLY SYSTEM			
HANFORD NO. 2 008.0-T1			
MARK NO. LPCS-19			
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California			
W.O. 3808	DWG. LPCS-19	SH 10915	REV 6







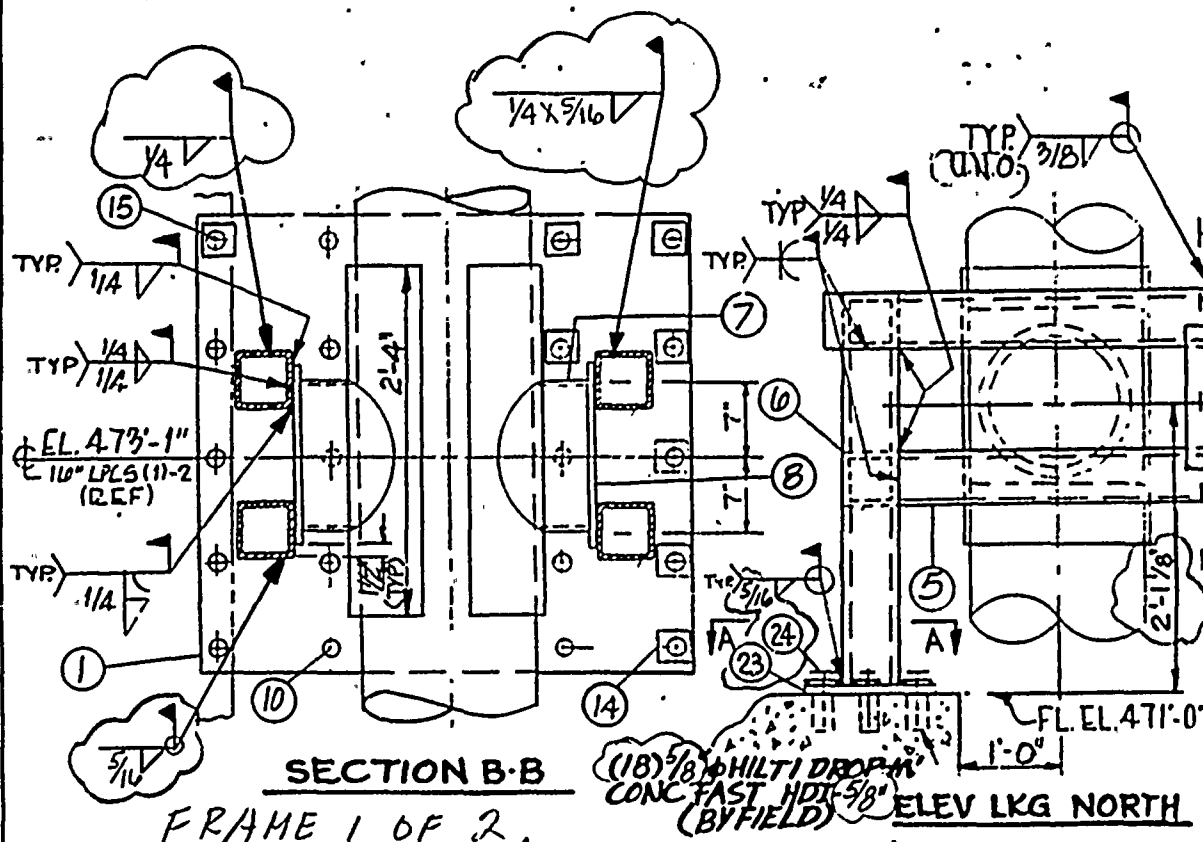
PLAN@ EL.473'-1"

INFORMATION ONLY

5 DOCUMENT UPDATE		12 12 82		1/2 1/2 5.1		WASHINGTON PUBLIC POWER SUPPLY SYSTEM					
REV NO		REVISION		DATE		OWN		APVD		HANFORD NO.2	
OWN RAKoch		CHKD		SCALE NTS		MARK O. LPCS-19					
SL/LOE		DATE 8-5-82		BURNS AND ROE, INC.							
ENGINEERING REVIEW				Engineers and Constructors							
MECH		CIVIL		New Jersey • New York • Connecticut • California							
ELEC											
REVD		APVD		DATE		W.O.38C8				DWG. LPCS-19	
CHIEF DRAFTSMAN		CHIEF MECH.		ENGINEER		SH 2 of 5				REV 6	

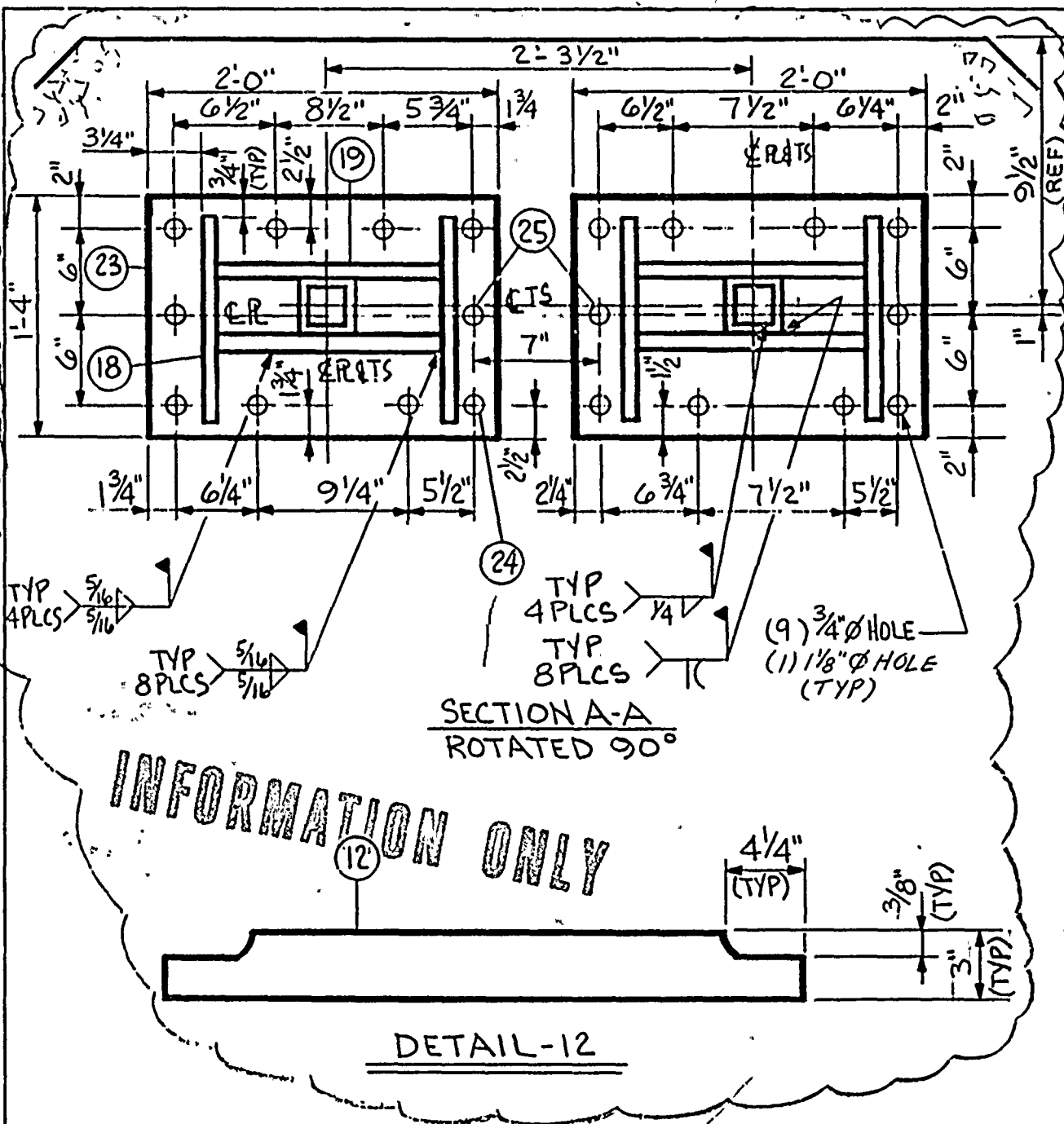


(70)  $\frac{3}{4} \phi$  MULTI-PROP-IN CONC FAST HDI-34 (BY FIELD)



5 DOCUMENT <del>DATE</del>		12 10 82		12 10 82		WASHINGTON PUBLIC POWER SUPPLY SYSTEM	
REV NO.		REVISION		DATE		HANFORD NO.2	
OWN <i>RAKoch</i>		CHKD <i>[Signature]</i>		SCALE <i>NTS</i>		MARK NO. <i>LPCS-19</i>	
SL/LOE <i>(1)</i>		DATE <i>8-5-82</i>				BURNS AND ROE, INC.	
ENGINEERING REVIEW				Engineers and Constructors			
MECH		CIVIL		New Jersey • New York • Connecticut • California			
ELEC							
REV'D <i>[Signature]</i> <i>3/1/82</i>		ATVD <i>[Signature]</i> <i>3/1/82</i>		DATE		REV	
CHIEF DRAFTSMAN		CHIEF		MECH. ENGINEER		W.O.3808 DWG. <i>LPCS-19</i> SH 3 of 5 6	





5	<del>DOCUMENT UPDATE</del>		12 10 82	OK	RP	S.F.
REV NO	REVISION		DATE	OWN	CHD	APVD
DWNG	CHKD		SCALE		MTB	
SL/IDE	DATE		8-5-82			
ENGINEERING REVIEW						
MECH			CIVIL			
ELEC						
REVD			APVD		DATE	
H.R.T. = 7-1-82 8-5-82			P. Mahoney		11-1-82	
CHIEF DRAFTSMAN			CHIEF		MECH ENGINEER	

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2

MARK NO.

LPCS - 19

**BURNS AND ROE, INC.**  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O.3808	DWG. LPCS-19
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REV

6

12-11-64

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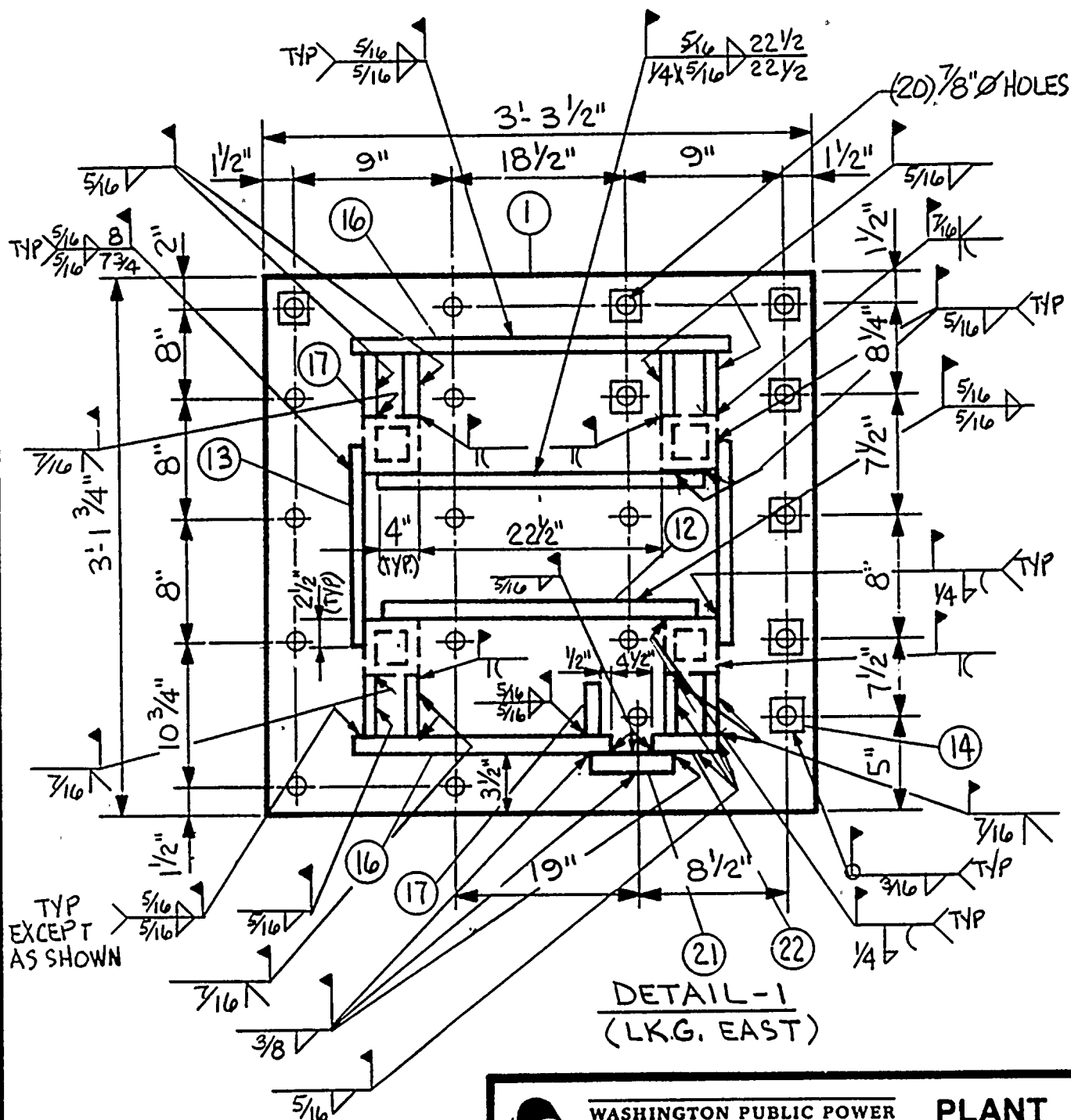
12-11-64

12-11-64

12-11-64

12-11-64

12-11-64



INFORMATION ONLY



WASHINGTON PUBLIC POWER  
SUPPLY SYSTEM

PLANT  
2

MARK NO. LPCS-19

DWG. LPCS-19

SH. 5 OF 5

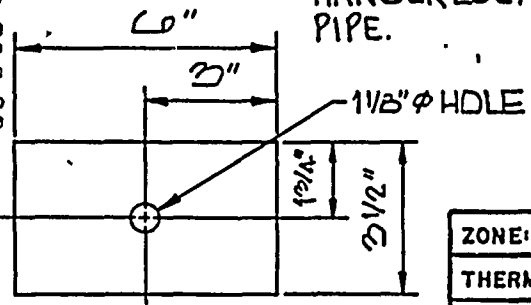
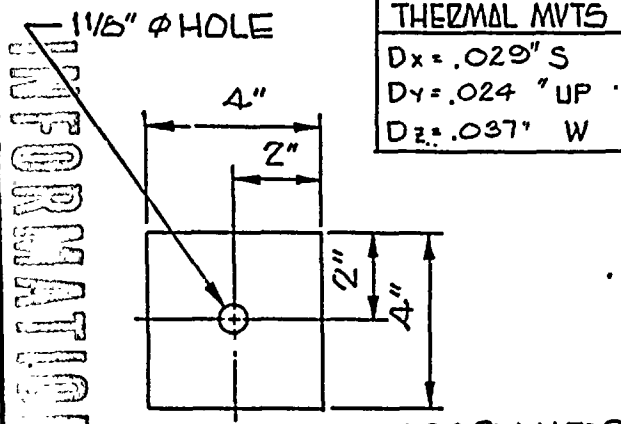
REV.  
6



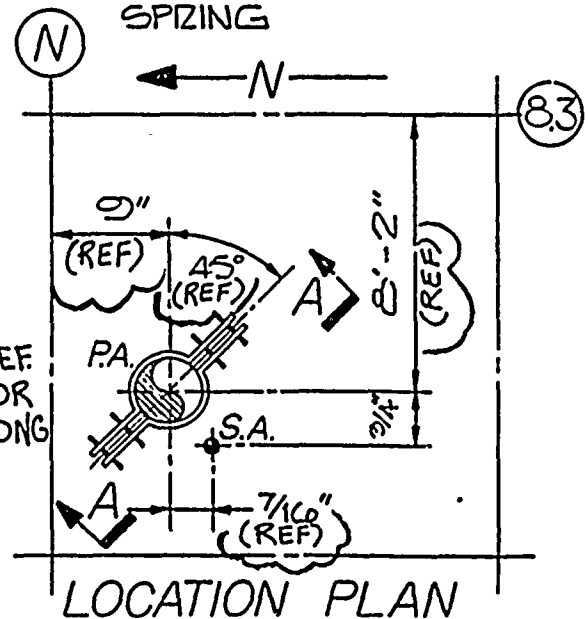


ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	1	1" FIG. 14DA BEAM BRACKET		8.3
2	1	#001 TYPE "D" VAR. SPRING H.I. = 3333 C.L. = 3347" W/T.S.		70
3	1	1" $\phi$ x 6" LG HANGER ROD FIG. 165 W/2 H/VY HEX NUTS		1.6
4	2	PL 1/4" x 4" x 6"	A36	3.4
5	2	PL 5/8" x 4" x 4" (SEE DETAIL 5)	A36	5.60
6	2	C 4 x 7.25 x 2'-10 1/2" LG	A36	41.7
7	4	LLG 3/4" x 1" x 4"	A36	1.7
8	2	1" FIG. 15S WELDLESS EYE NUT		4.9
9	1	14" SPECIAL RISED CLAMP C-C = 2'-6" STOCK SIZE: 3/4" x 4" W/5/8" x 3 1/2" W/HVY HEX NUTS SPACER SIZE 1" O.D. (SEE DET. 9)		85
10	2	1" $\phi$ x 12'-4" LG HANGER ROD FIG. 165 W/2 H/VY HEX NUTS		65
11	1	PL 5/8" x 3 1/2" x 6" (SEE DETAIL 11)	A36	3.72
		TOTAL		291

02-215-08	9413	6	✓
CVI Number	CVI Sheet	Rev.	Date Checked

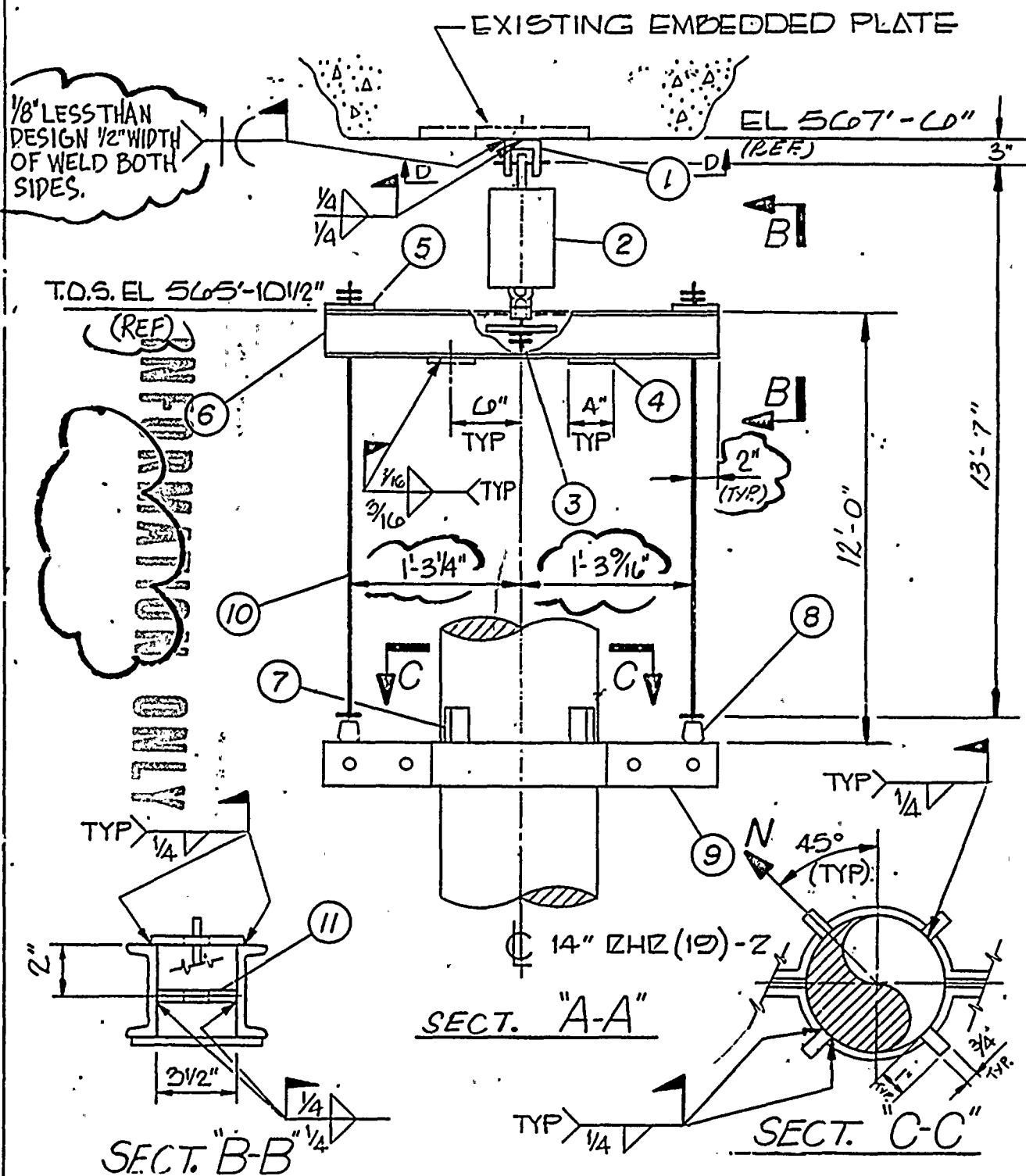


THERMAL MVTS  
 Dx = .029" S  
 Dy = .024 " UP  
 Dz = .037" W



ZONE: 12-63	OR LD.: 3042*	HYDRO. LD.: 3042*
THERMAL LD.: VERT: ~ N-S: ~ E-W: ~		
SEISMIC LD.: VERT: ~ N-S: ~ E-W: ~		
CODE/CLASS: III/2	GROUP: I	Q.A. LEVEL: I
PAINT: I	PIPE CALC.: 8.14.1278	STEEL CALC.: 8.14.477
PIPING RESIDUAL SYSTEM HEAT REMOVAL	REF. DWG. ISO.: 1242-578-1.4 . PIPING: M706	
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2 0092-P5		
MARK NO. 1242-53		
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
W.O. 3808	DWG. 1242-53 SH. 1 OF 3	REV 6





5		DOCUMENT UP-DATE		8/16	8/16	8/16	WASHINGTON PUBLIC POWER SUPPLY SYSTEM	
REV NO.		REVISION		DATE	OWN	CHKD	APVD	HANFORD NO.2
DWN SULLIVAN		CHKD L. A. RODGER		SCALE N.T.S.		MARK NO. 242-53		
SLIDE PORTER		DATE		11-14-79		BURNS AND ROE, INC.		
ENGINEERING REVIEW								Engineers and Constructors
MECH		CIVIL		New Jersey • New York • Connecticut • California				
ELEC		DATE		W.O.3808				
REV'D		APVD		DATE		DWG. 242-53 SH. 2 OF 3		
H.B. T. ...		CHIEF		MECH.		ENGINEER		

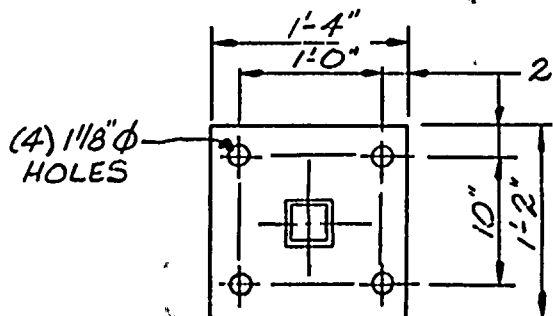






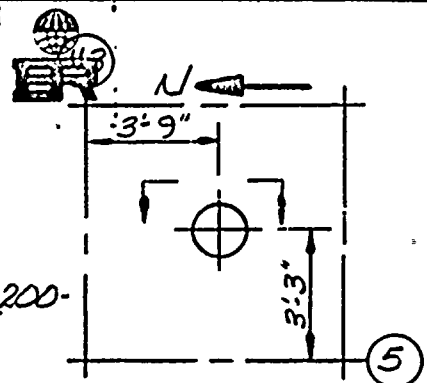
SUPERIOR REPROGRAPHICS 18 398

ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
ITEMS 1 THRU 13, 15 THRU 21 HAVE BEEN DELETED				
14	1	R 1" x 22" x 1'-10"	A36	137
23	1	TS 10 x 10 x .500 x 2'-0" LG (CUT TO SUIT)	A500GRB	71
24	1	R 1/2" x 11" x 1'-0"	A36	19
25	1	TS 6 x 6 x .375 x 4'-2" LG (CUT TO SUIT)	A500GRB	113
26	1	TS 10 x 10 x .500 x 3'-5" LG	A500GRB	208
27	1	TS 8 x 8 x .500 x 4'-4" LG (CUT TO SUIT)	A500GRB	205
28	1	R 3/4" x 9 1/2" x 1'-6"	A36	36
29	1	TS 8 x 8 x .500 x 3'-6" (CUT TO SUIT)	A500GRB	166
30	1	R 1" x 28" x 3'-4"	A36	317
31	1	TS 10 x 10 x .500 x 2'-4" LG (CUT TO SUIT)	A500GRB	142
32	1	R 1" x 14 1/4" x 1'-9 1/8"	A36	85
33	1	R 1" x 22 1/2" x 2'-0 5/8"	A36	157
34	1	R 1" x 14" x 1'-4"	A36	56
35	2	R 3/4" x 4" x 2'-10 1/2"	A36	59
36	1	R 3/4" x 4" x 2'-2 1/2"	A36	23
37	4	R 3/4" x 4" x 0'-3 1/2"	A36	107
38	2	R 3/4" x 4" x 1'-7 1/8"	A36	33
39	1	R 3/4" x 4" x 1'-9"	A36	13



DETAIL 34

LOCATION PLAN FOR  
REFERENCE ONLY!  
USE FAB. ISO. FOR  
CONSTRUCTION



LOCATION PLAN

MOMENTS (LBS-FT)			ZONE: R-32	OP. LD.: Fx = -1 * Fy = -56.9 * Fz = +101 *	HYDRO. LD.:
MX	MY	MZ	THERMAL LD.: VERT = +1984 * N-S = +339 * E-W = +173 *		
THERMAL	-3743	-2384	-3683	SEISMIC LD.: VERT = +2434 * N-S = +1167 * E-W = +1845 *	
DEAD WT.	-1105	+126	-37	CODE/CLASS: III/2	GROUP: 1
SEISMIC	+16329	+7105	+9105	PAINT: I	PIPE CALC.: 8.30.182
			STL. CALC.: 8.16.555		

7	DOCUMENT UP: DATE	8/19	RAK	1/1
REV NO	REVISION	DATE	DWN	APVL
DWN	CHKD	SCALE	NTS	
SL/LDE	DATE	7-20-66		
ENGINEERING REVIEW				
MECH	CIVIL	DATE	3/1/71	
ELEC				
REVD	APVD	DATE	1/1/72	
CHIEF DRAFTSMAN	CHIEF MECH.	ENGINEER		

PIPING RESIDUAL	REF. DWG. ISO.: RHR-897-10.14
SYSTEM/HEAT REMOVAL	PIPING: M-703
WASHINGTON PUBLIC POWER SUPPLY SYSTEM	
HANFORD NO. 2	
MARK NO. RHR-99	
BURNS AND ROE, INC.	
Engineers and Constructors	
New Jersey • New York • Connecticut • California	
W.O. 3808	DWG. RHR-99
SH. 1 OF 4	
REV	7

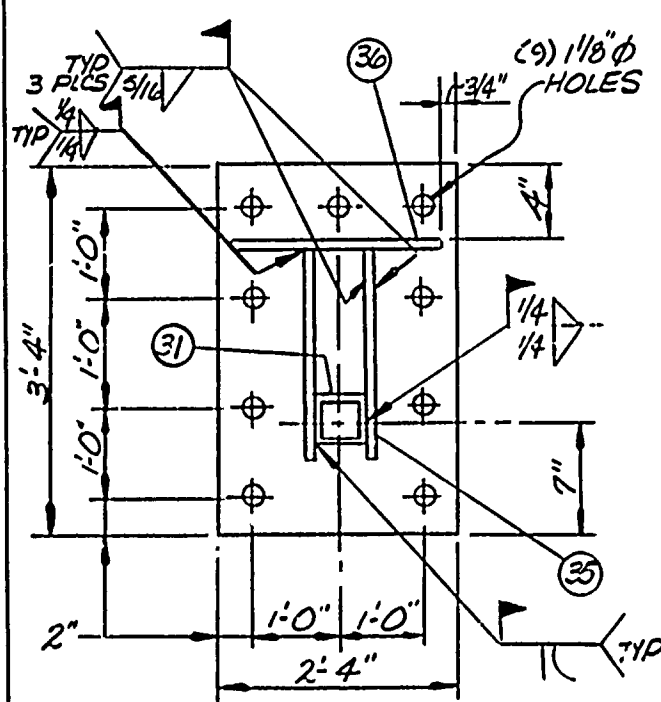
CUI # 215-08, 7871



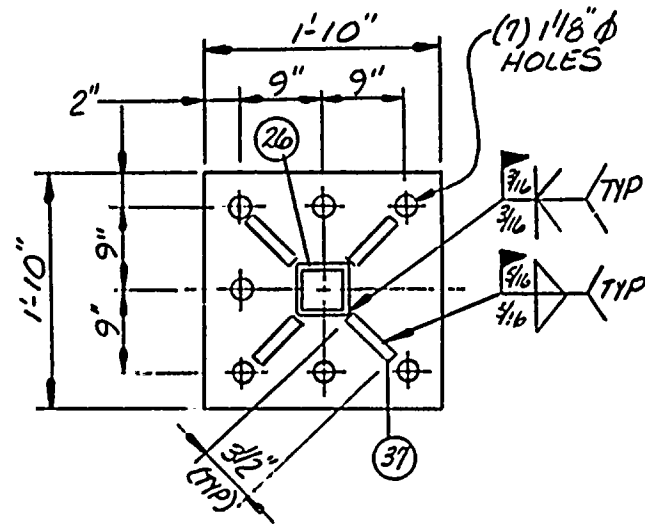


SUPERIOR REPROGRAPHICS 18 397

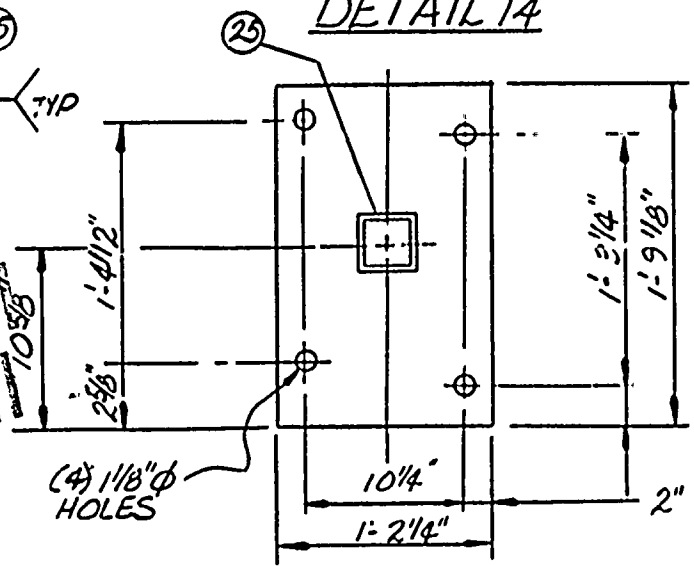
ITEM NO	QTY	SIZE-DESCRIPTION	ASTM	WT	
40	2	R 3/4" x 14" x 1'-8" (PAD) (BEND TO RADIUS OF 14" PIPE)	A36	45	
41	31	1" Ø x 6'2" EMBED SUPER KWIK BOLT w/ Hvy HEX NUT & LW			
42	2	R 7/8" x 9" x 0'-9" (COPE TO SLIT)	A36	17	



DETAIL 30



DETAIL 14



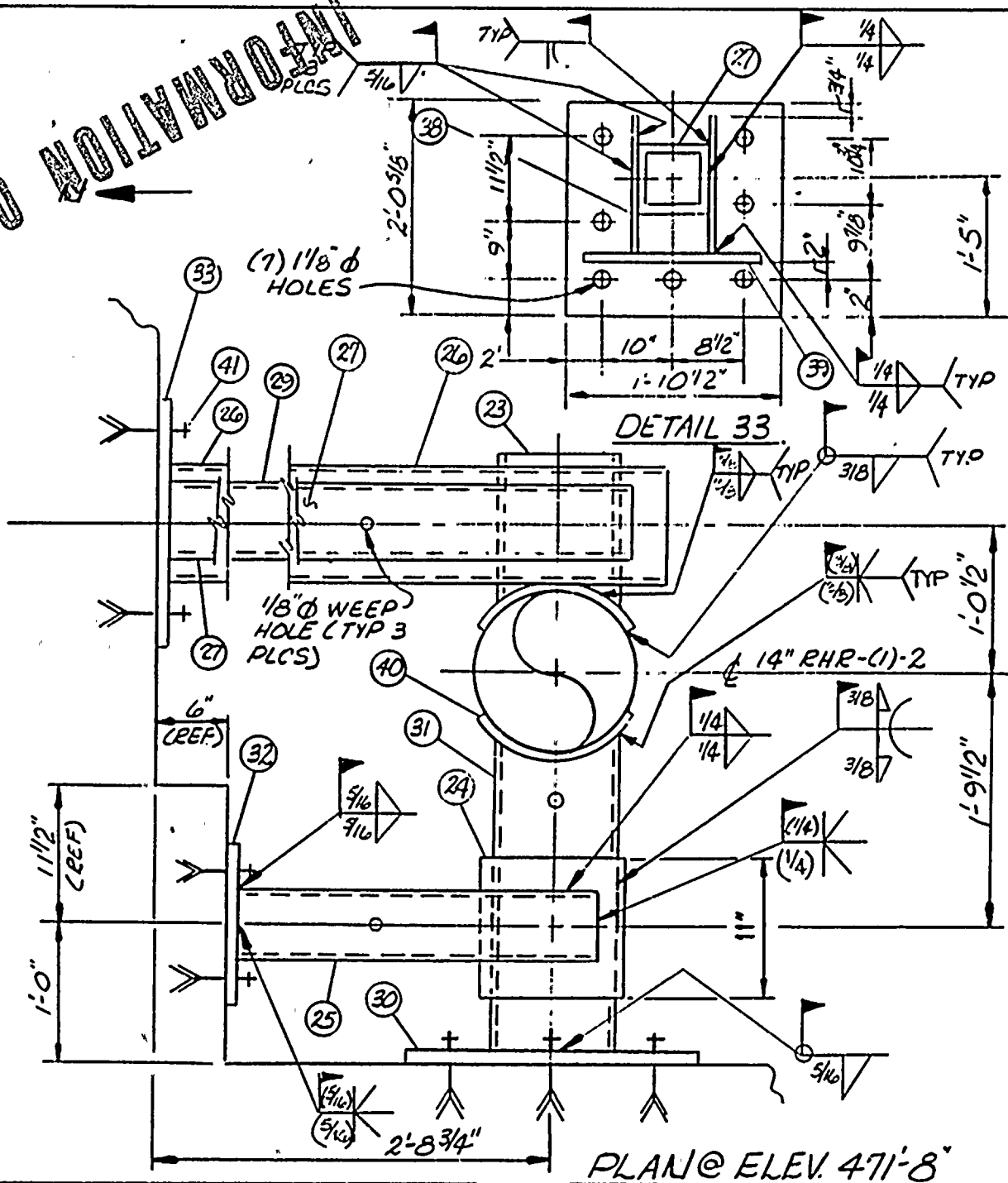
DETAIL 32

7	DOCUMENT UP-DATE	8 19 82	RK JRP S.F.	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2 9.10 1'06	
REV NO	REVISION	DATE	DWG CHKD	APVD	
DWN	2 School	CHKD	ENR	SCALE	1/8"
SL/LOE		DATE	7-20-62	MARK NO.	RHR-99
ENGINEERING REVIEW			BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
MECH		CIVIL	7/11/62	DATE	8/5/62
ELEC		APVD	7/11/62	DATE	8/5/62
REV	7/11/62	APVD	7/11/62	DATE	8/5/62
CHIEF DRAFTSMAN		CHIEF	MECH.	ENGINEER	
W.O.3808		DWG. RHR-99		REV 7	
				SH 2 OF 4	



7700 INFORMATION

SUPERIOR REPROGRAPHICS 18337



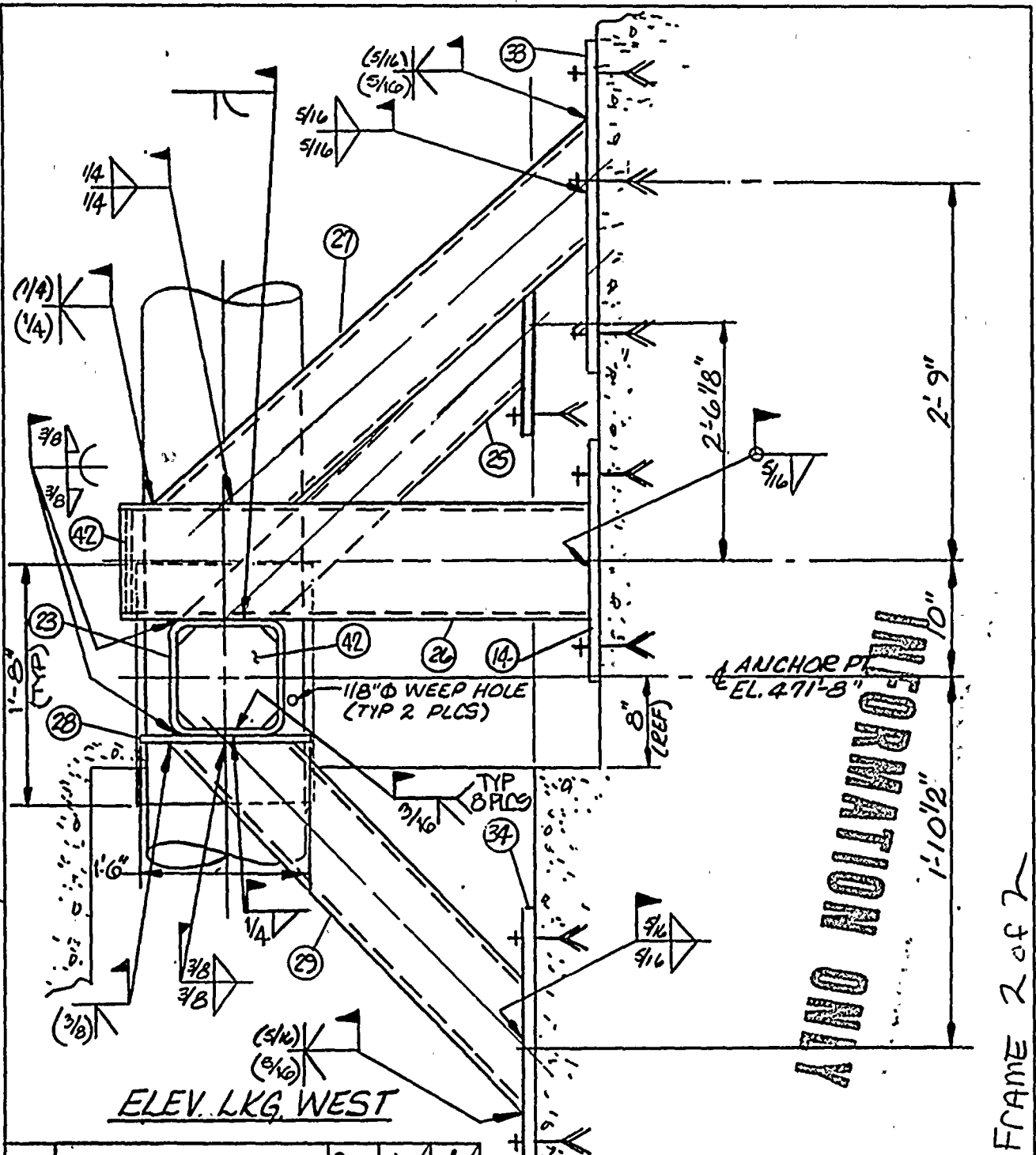
PLAN @ ELEV. 471'-8"

FRAME 1 of 2

7	DOCUMENT UP-DATE	8 19 82	REV CHKD APVD	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2 9.0 P06
REV NO	REVISION	DATE	OWN CHKD APVD	MARK NO. RHR-99
OWN	CHKD	DATE	SCALE	BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California
SL/IDE	DATE	DATE	DATE	W.O.3808 DWG. RHR-99 SH3 OF 4
ENGINEERING REVIEW				REV 7
MECH	CIVIL	APVD	DATE	
ELEC				
REVD				
CHIEF DRAFTSMAN	CHIEF MECH. ENGINEER			



CUT # 215-081787



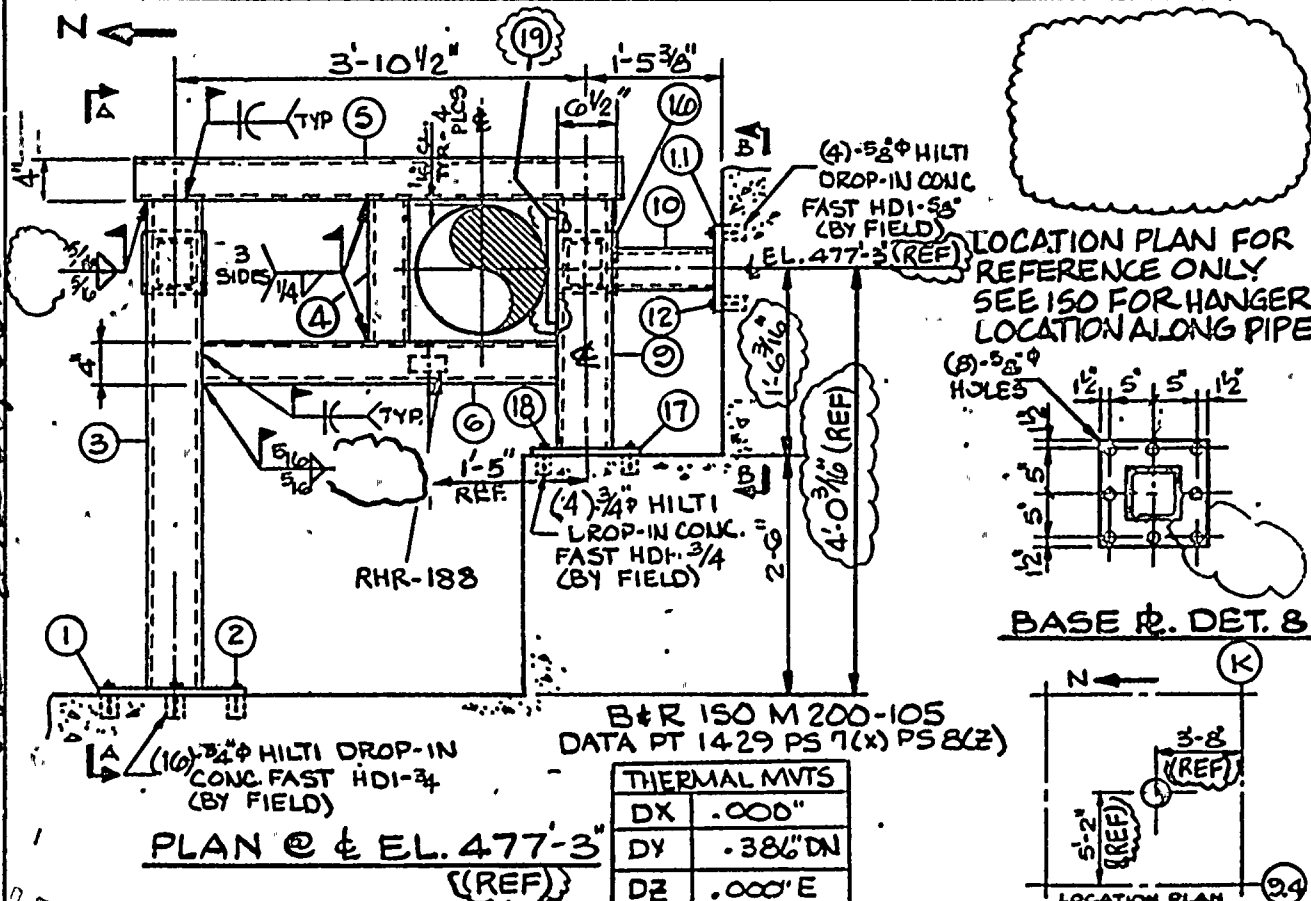
INFORMATION ONLY

FRAME 2 of 2

7		DOCUMENT UP-DATE		8	19	82	PK	SK	APVD	WASHINGTON PUBLIC POWER SUPPLY SYSTEM	
REV NO	REVISION			DATE	DWN	CHKD	APVD	HANFORD NO.2 9.0 P06			
DWN	SK	CHKD	APVD	SCALE	NTS			MARK NO. RHR-99			
SL/LDE	DATE 7-20-82			ENGINEERING REVIEW			BURNS AND ROE, INC.				
MECH	CIVIL			APVD			Engineers and Constructors				
ELEC	APVD			DATE 8-15-82			New Jersey • New York • Connecticut • California				
REV D	APVD			DATE			W.O.3808 DWG. RHR-99			REV	
CHIEF DRAFTSMAN			CHIEF MECH. ENGINEER			SH 4 OF 4			7		

215-08  
128L

ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	FT.
1	2	1" x 17" x 1'-5" LG.	A36	164
2	16	3/4" x 2 1/4" LG. HVY. HEX. MACH. BOLTS. W/L.W.	A307GR.B	8
3	1	TS 6 x 6 x .375 x 4'-8 1/4" LG	A500GR.B	127
4	1	TS 4 x 4 x .375 x 1'-6 1/4" LG		26
5	1	TS 6 x 4 x .375 x 4'-5 7/8" LG		99
6	1	TS 6 x 4 x .375 x 3'-4 1/2" LG		75
7	8	1/2" x 1 3/4" LG. HVY. HEX. MACH. BOLTS W/L.W.	A307GR.B	2
8	1	1" x 13" x 1'-1" LG (SEE DET. 8)	A36	48



B & R ISO M 200-105  
DATA PT 1429 PS 7(X) PS 8(Z)

THERMAL MVT	
DX	.000"
DY	.386" DN
DZ	.000' E

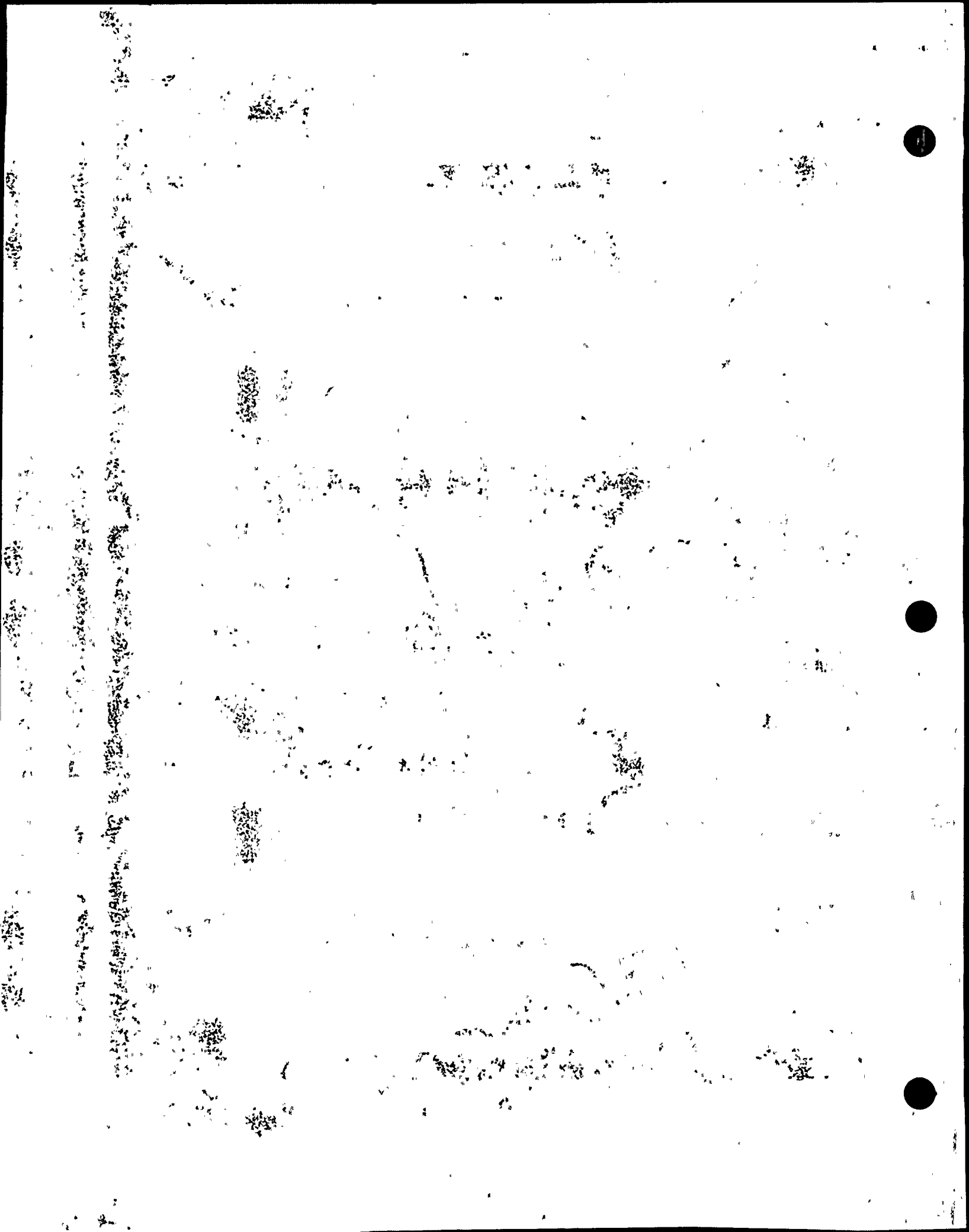
ZONE: R-31	OP.LD.: F <sub>1</sub> = -78° F <sub>2</sub> = -4°	HYDRO.LD.: —
THERMAL LD.: VERT: N-S = -1016" E-W = -1582"		
SEISMIC LD.: VERT: N-S = ± 1813" E-W = ± 841"		
CODE/CLASS: III/2	GROUP: I B	Q.A. LEVEL: I
PAINT: 1	PIPE CALC.: 8.14.630	STL CALC.: 8.15.471
PIPING RESIDUAL SYSTEM HEAT REMOVAL	REF. DWG. ISO: RHR-867-8.12 PIPING: M704	

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO. 009.0-P4

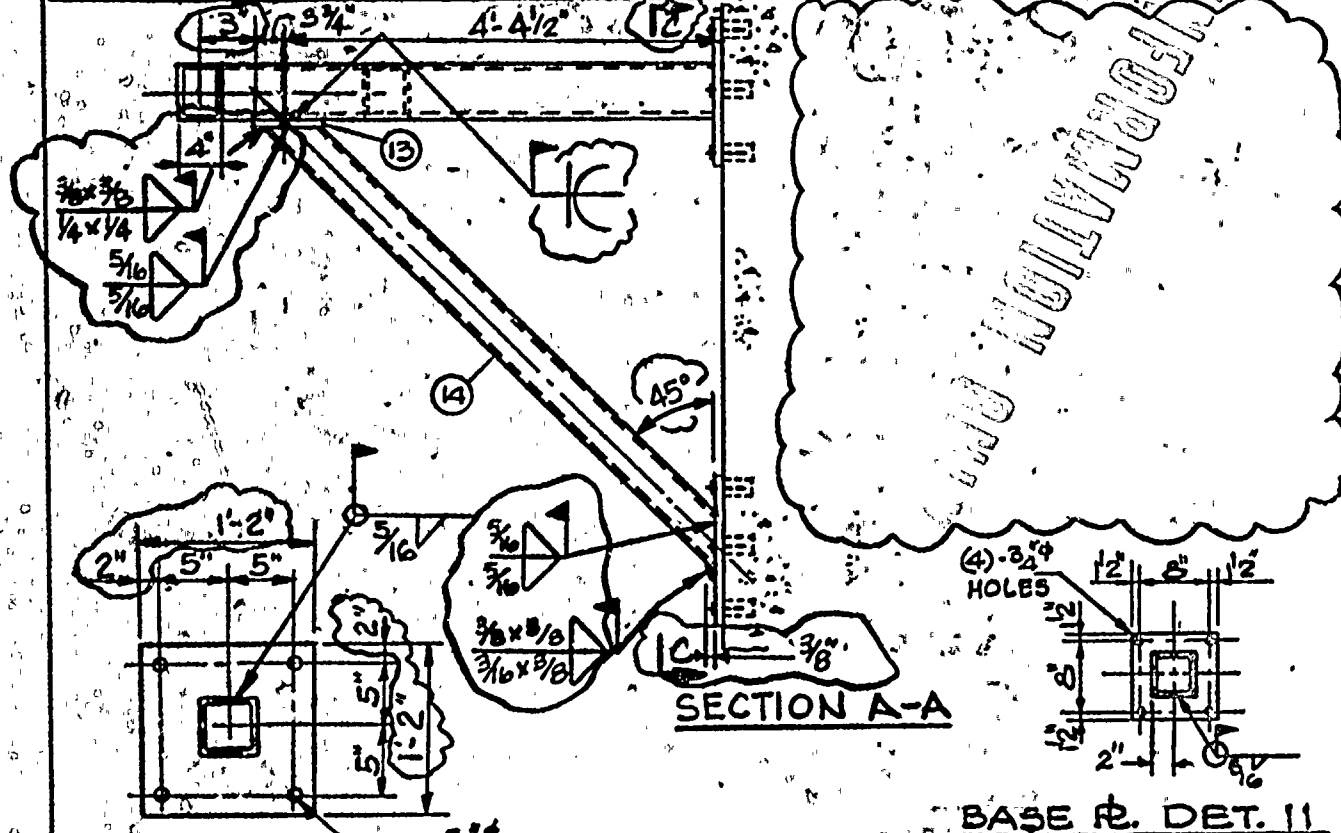
4	3/15/60	CONST. COMPLETE PER PRR REV 2 (4FO) LFF 4 PEDS 215-H-P283 215-H-W019, 215-H-W409	DS	ENG
REV NO	DATE	REVISION	DS	ENG
DWN D.H.	CHKD PAPADOPOULOS	SCALE NTS		
BL/LDR	DATE 11/12/79	ENGINEERING REVIEW		
MECH R. Ball	CIVIL J. Donnell	DATE 11/17/79		
ELEC				
REV D	APVD	DATE		
W. Tiague/EEF 4/1/79	0/5/79	1/1/79		
CHIEF DRAFTSMAN	CHIEF	MECH. ENGINEER		

MARK NO. RHR-174	
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California	
W.O.3808	DWG. H RHR-174
SHT. 1 of 4	REV 4





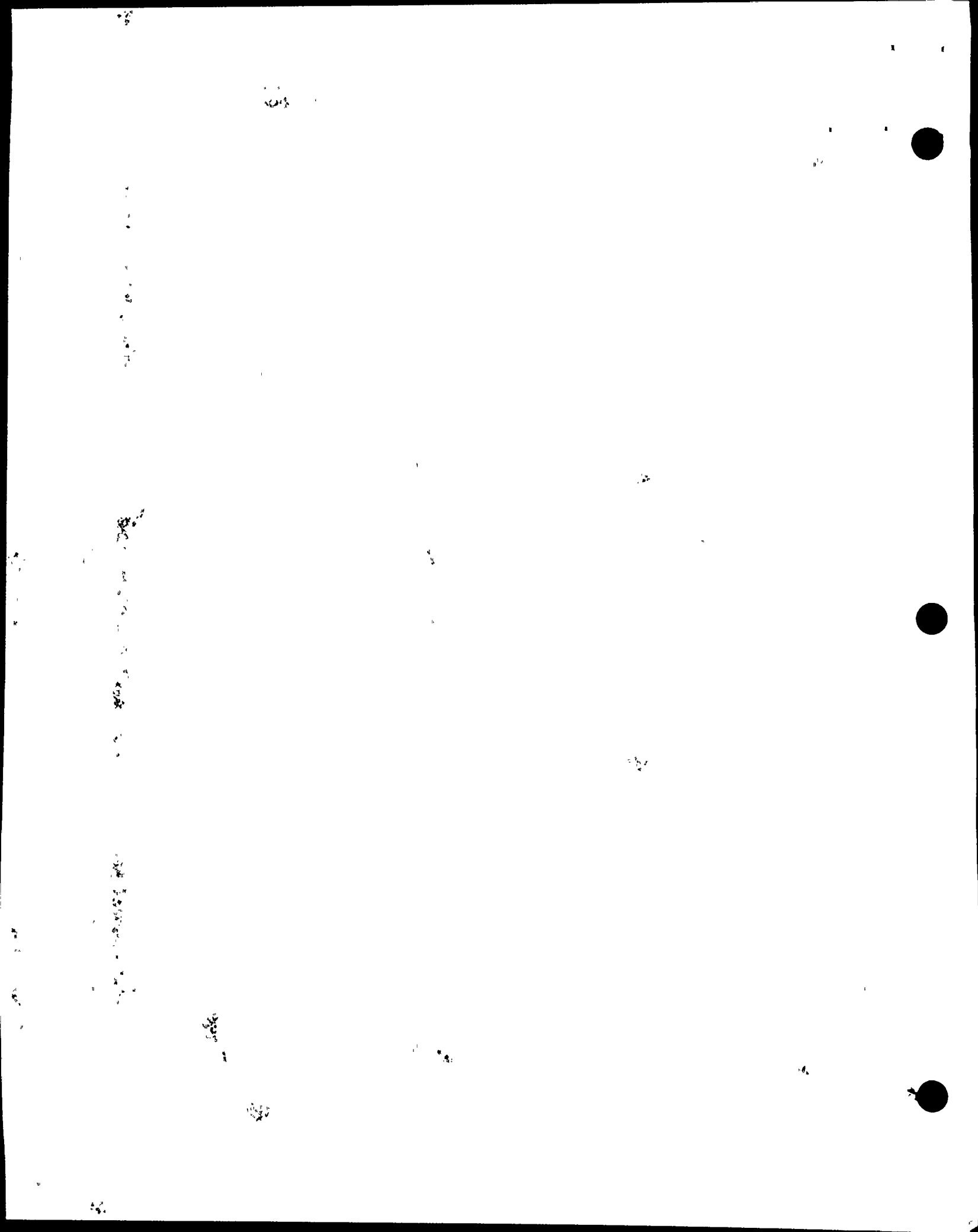
ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	WT.
9	1	TS 6x6x.375x2'-2" LG	A500GR.B	59
10	1	TS 6x4x.375x1'-1 1/4" LG	A500GR.B	24
11	1	2 1/2x11x0'-11" LG. (SEE R. DET. 11)	A36	34
12	4	5/8x2" LG. H.V. HEX. MACH. BOLTS W/L.W.	A307GR.D	1
13	1	2 1/2x7x0'-7" LG.	A36	14
14	1	TS 4x4x.375x6'-2" LG (CUT TO SUIT)	A500GR.D	.99
15	1	TS 4x4x.375x2'-9 1/2" LG (CUT TO SUIT)	A500GR.D	40
16	1	2 1/2x6x0'-7" LG.	A36	14
17	1	2 1/2x6x0'-7" LG.	A36	26
18	4	3/4x2 1/4" LG H.V. HEX. MACH. BOLT W/L.W.	A307GR.D	1



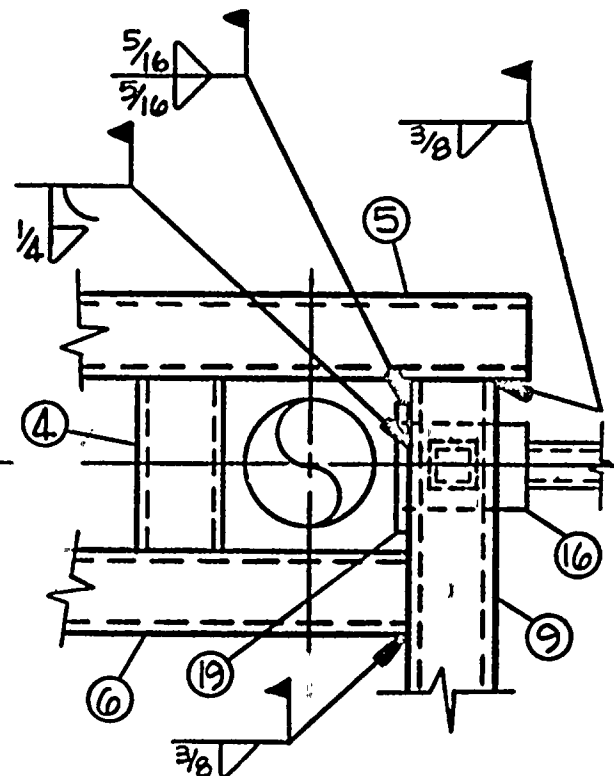
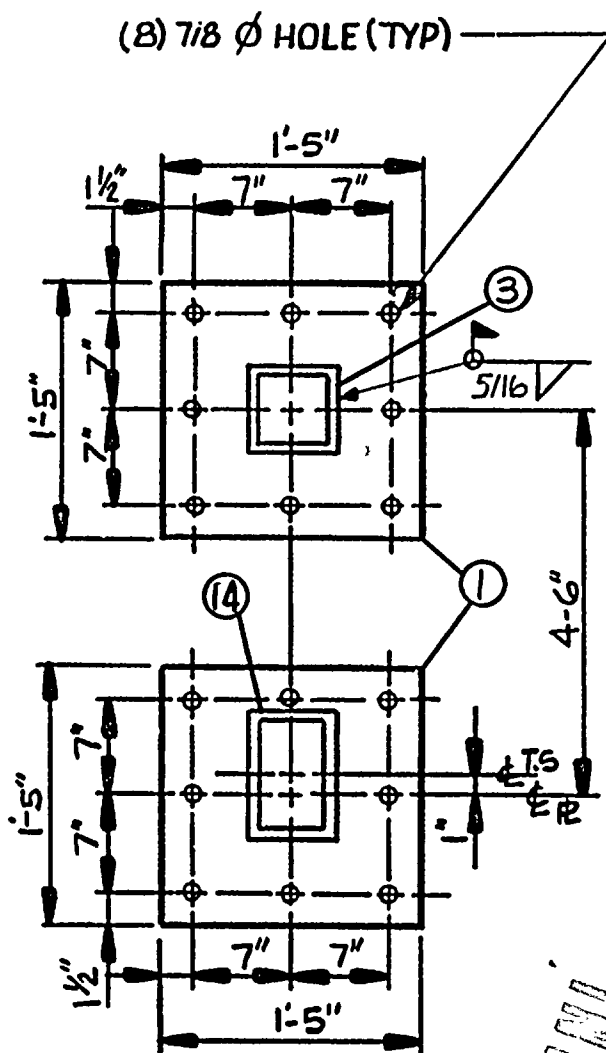
DETAIL-17

ZONE: R-31	OR.LD. =	HYDRO.LD. =
THERMAL LD. VERT. =	N-S =	E-W =
SEISMIC LD. VERT. =	N-S =	E-W =
CODE/CLASS: III/2	GROUP: 1B	Q.A. LEVEL: I
PAINT: I	PIPE CALC. 8.14.63	STL. CALC. 8.15.471
PIPING RESIDUAL	REF. DWG. 150. RHR-867-8.12	
SYSTEM HEAT REMOVAL	PIPING: M704	
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2		
MARK NO. RHR-174		
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
W.O.3808	DWG. H RHR-174	REV 4

3	REV'D BY B.R. INCORP. PED. 11-21-79 & REV'D IN AC- CORDANCE WITH STATE K&M	8/2/79	NC	1/15/79	1/15/79
2	REDESIGNED BY B.R.	7/21/78	NC	1/15/79	1/15/79
1	REV'D BY N.R.S.	7/21/78	NC	1/15/79	1/15/79
REV NO.	REVISION	DATE	CHKD	APP'D	DATE
DWN D.H.	CHKD PAPAPOULOS	SCALE	NTS		
BLDGE	D.P. M. M. M.	DATE	1/12/79		
ENGINEERING REVIEW					
MECH	B.R. M. M.	CIVIL	J. M. M.	1/15/79	
ELC					
REV'D	1/15/79	APP'D	1/15/79		
CHIEF/DRAFTSMAN		CHIEF			



ITEM	QUAN	SIZE-DESCRIPTION	ASTM	WT.
12	1	PL 1/2 X 6" X 0'-10" LG		



DETAIL-A



WASHINGTON PUBLIC POWER  
SUPPLY SYSTEM

PLANT  
2

MARK NO. RHR-174

DWG. RHR-174

SH. 3 OF 4

REV.  
4



REV  
4

242  
FOA -

11/11

242  
FOA -

242  
FOA -

242  
FOA -

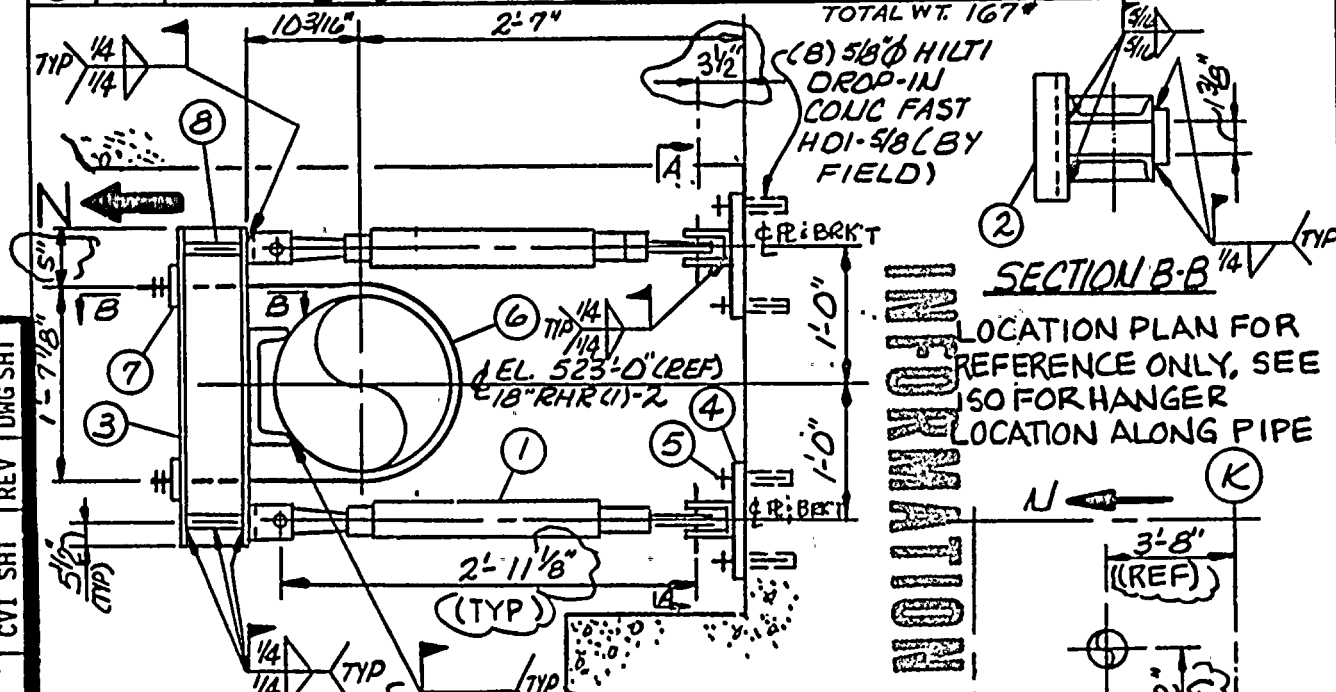
242  
FOA -

242  
FOA -

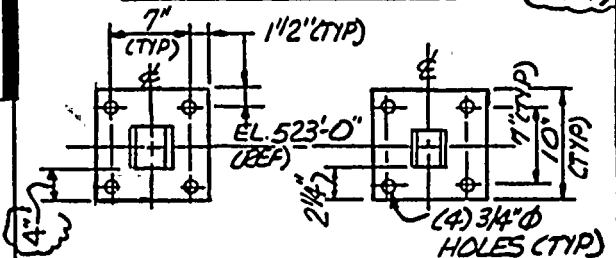
242  
FOA -

11/11

ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	2	RIGID SWAY STRUT SIZE 15 FIG 350 x 1-11/4" W (2) M146 REAR BRKTS.		26
2	1	C 10 x 2 1/2 x 1'-0" LG	A36	21
3	2	C 6 x 10.5 x 2'-6" LG	A36	53
4	2	R 3/4" x 10" x 0'-10" (SEE SECTION A-A)	A36	42
5	8	5/8" x 1 3/4" LG HVY HEX MACH. BOLT W/LW	A307 GRB	4
6	1	18" U-BOLT STD. FIG 100A B=2'-2"		13
7	2	WASHER R 1" x 1" FIG 160		4
8	4	STIFF R 3/8" x 1 1/16" x 0'-5 1/4"	A36	4



PLAN @ ELEV. 523'-0" (REF)



SECTION A-A

THERMAL MVS	
DX	.000" U
DY	.042" UP
DZ	.001" W

ZONE: R-51	OR LD. = Fx = -154#	HYDRO. LD. =
THERMAL LD.: VERT. =	N-S = 717'-188" E-W =	
SEISMIC LD.: VERT. =	N-S = 4535" E-W =	
CODE/CLASS: III/2	GROUP: 1	Q.A. LEVEL: I
PAINT: I	PIPE CALC: 8.14.127C	STL CALC: 8.15.206G
PIPING RESIDUAL HEAT SYSTEM REMOVAL	REF. DWG. ISO. RHR-867-8.12	PIPING: M-705

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2 5/4 9.0 P04

MARK NO. RHR-605

BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O.3808 DWG. RHR-605 REV 3

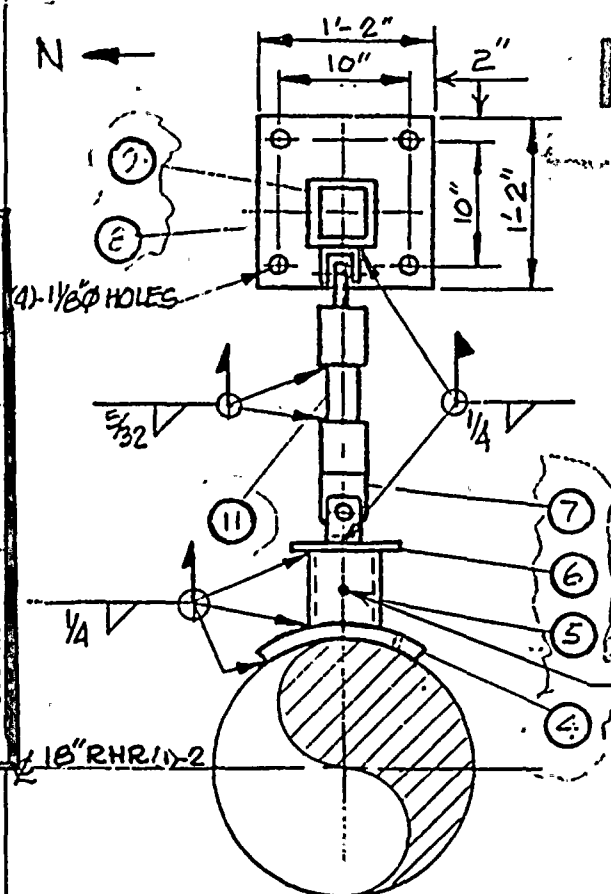
102-715-08	CVI NO	CVI SHT	REV	DWG SHT
12543			3	

3	6/12/82	CONST. COMPLETE PER PRR REV 2 (3FO) & PED. 215-H-N733, 16A34	Frp	DS	ENG
REV NO	DATE	REVISION	D/CK	DS	ENG
DWN	DATE	CHKD	SCALE	NTS	
SL/IDE	DATE	DATE	DATE	DATE	
ENGINEERING REVIEW					
MECH		CIVIL			
ELEC					
REVD	DATE	DATE	DATE	DATE	
H.R. Tregue	8-19-82	8/25/82	8/25/82	8/25/82	
CHIEF DRAFTSMAN	CHIEF MECH.	ENGINEER			





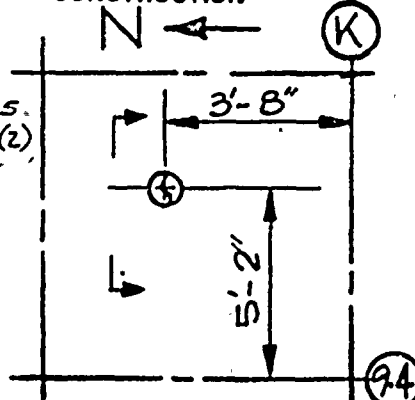
ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	WT.
4	1	R 3/8" X 12" X 1'-5." PAD (BEND TO RADIUS OF 18" RHR)	A-36	21
5	1	8" Ø SCH. 40 PIPE X 0'-6 1/2" LG. (CUT TO SUIT)	A106 GR B	15
6	1	R 1/2" X 9" X 0'-9"	A-36	5
7	2	PACIFIC SCIENTIFIC PSA-3 FORWARD ADAPTER P/N 1801319-CA		12
		W/REAR BRACKET P/N 1801558-05		
8	1	R 1" X 14" X 1'-2" (SEE PLAN)	A-36	56
9	1	TS 5 X 5 X .375 X 0'-11 1/2" LG	A500 GR B	21
10	4	1" Ø X 6 1/2" EMBED HILTI SUPER KWIK BOLT W/HVY HEX NUT & LW		



## INFORMATION ONLY

NOTE:  
ENGINEERING HAS RELOCATED THIS SUPPORT 2" UP IN Y-DIRECTION. FIELD TOLERANCE IN THIS DIRECTION IS NOW 10".

LOCATION PLAN FOR REFERENCE ONLY!  
USE FAB. ISO. FOR CONSTRUCTION



B & R ISO M200-105.  
DATA PT 1325 PS 6(2)

THERM. MVT.	
D <sub>x</sub>	0.000"
D <sub>y</sub>	0.040" UP
D <sub>z</sub>	0.000"

## LOCATION PLAN

PLAN @ EL. 522'-10"

ITEM #12 NOT SHOWN FOR CLARITY

ZONE: R-51	OP. LD. = F <sub>2</sub> = -121'	HYDRO. LD. = F <sub>2</sub> 121'
THERMAL LD.: VERT: —	N-S: —	E-W: ± 27' ± 1132"
SEISMIC LD.: VERT: —	N-S: —	E-W: ± 2812"
CODE/CLASS: III/2	GROUP: 1	QA LEVEL: I
PAINT: I	PIPE CALC. 8.14.1270	STL CALC. 8.15.2067
PIPING RESIDUAL HEAT REMOVAL SYSTEM	REF. DWG. ISO: RHR-867-B/12	PIPING: M-70

2	REDESIGNED BY B&R & REV'D IN ACCORDANCE W/ WBS STATUS AS BUILT			10/22/72	JMT	RB	MP	NED
REV NO	REVISION			DATE	OWN	CHKD	APVD	
DWN	CHKD			SCALE		NTS		
SL/LDE	DATE 11/1/72			ENGINEERING REVIEW				
MECH				CIVIL				
ELEC								
REV'D	APVD			DATE				
CHIEF DRAFTSMAN				CHIEF		MECH		ENGINEER

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO. 2

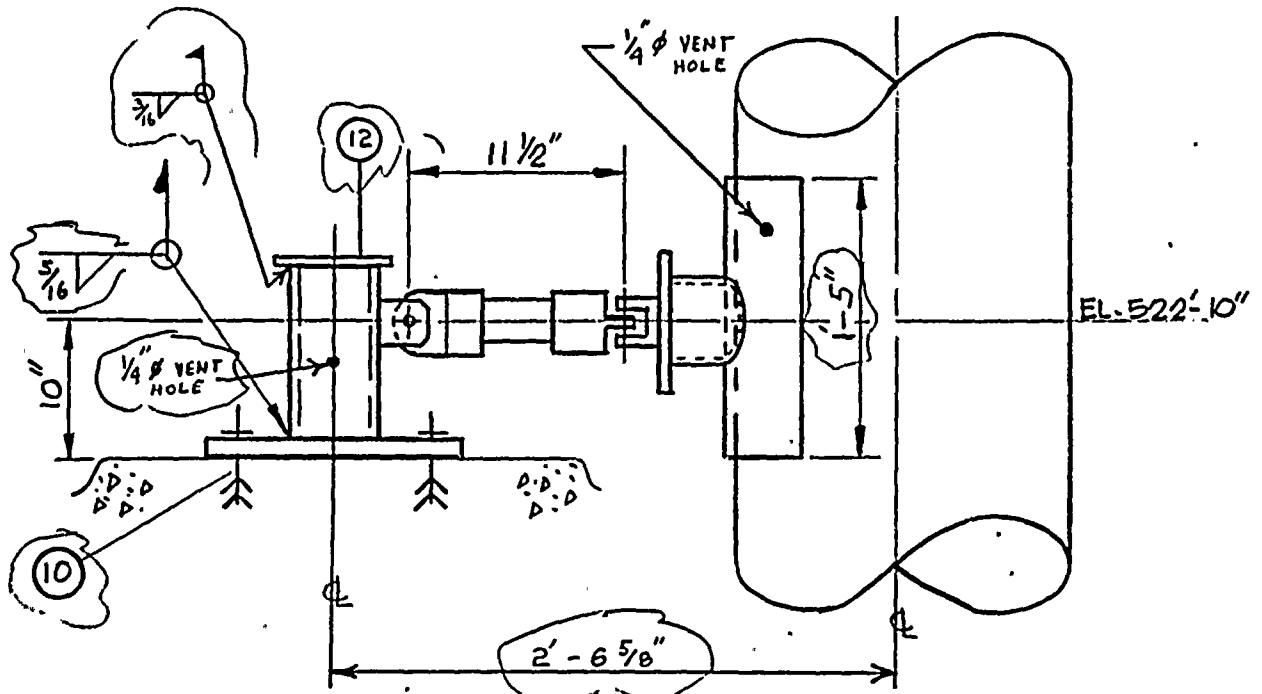
MARK NO. RHR-606

BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O. 3808	DWG. RHR-606	REV 2
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ITEM NO.	QTY	SIZE - DESCRIPTION	ASTM	WT.
11	1	1 1/2" $\phi$ PIPE SCH. 40 X 0' 6 1/2" LG	A106 GR B	2
12	1	R 1/4" x 7" x 0' 7"	A-36	3
ITEMS 1 THROUGH 3 DELETED			TOTAL WT	



ELEV. LKG. SOUTH

INFORMATION ONLY

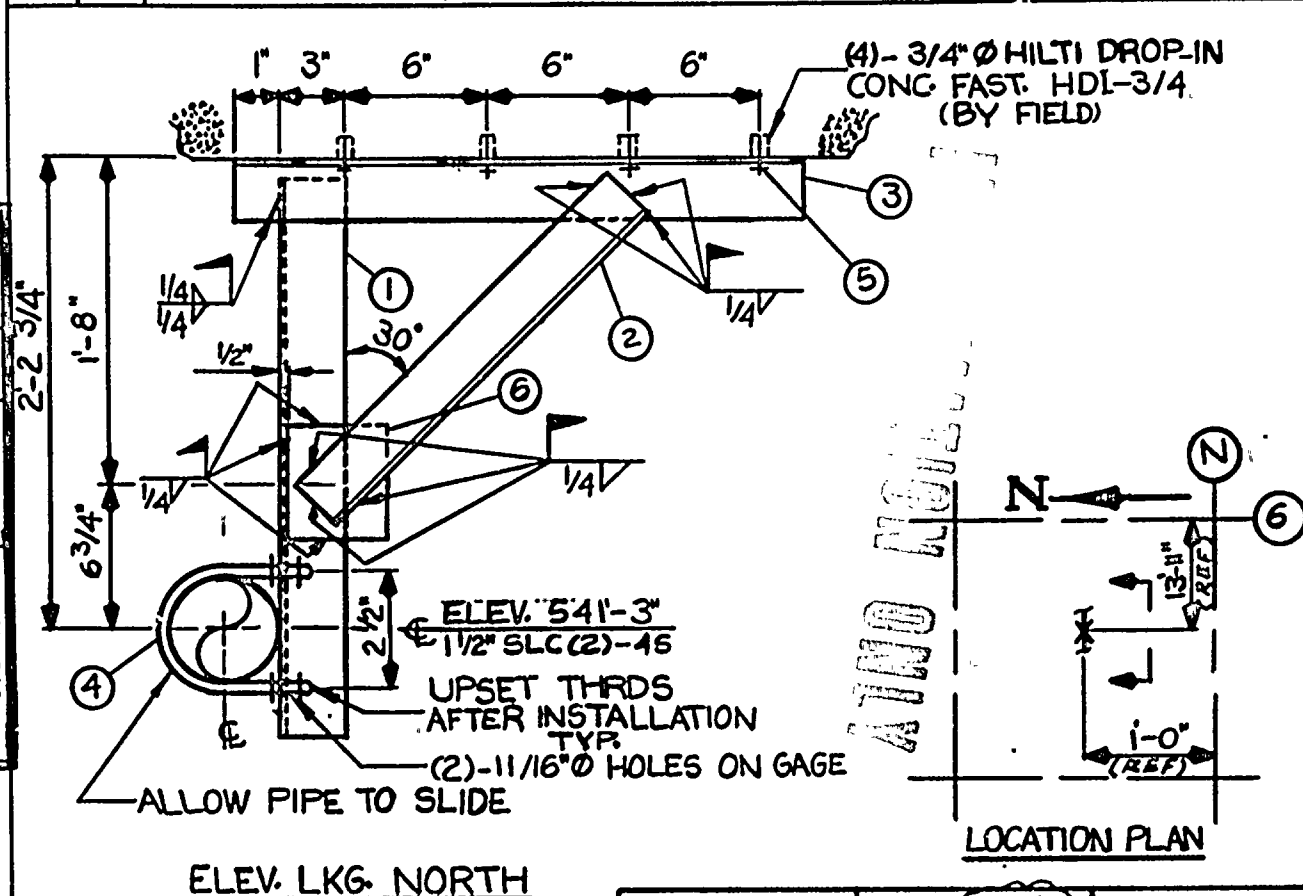
2		REDESIGNED BY BERT REV'D IN ACCORDANCE W/ WBG STATUS AS BUILT		JMT Rb MCU		WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2	
REV NO	REVISION	DATE	OWN	CHKD	APVD	MARK NO. RHR-606	
DWN	SHD	CHKD	APL	SCALE	NTS	BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California	
SL/LDE	DATE	11-1-82	ENGINEERING REVIEW				
MECH	CIVIL			W.O.3808 DWG. RHR-606			
ELEC	DATE			REV			
REV'D	APVD	DATE	2				
CHIEF DRAFTSMAN	CHIEF MECH	ENGINEER	SH20E2				



SUPERIOR REPROGRAPHICS TB 308

ITEM NO.	QUN.	SIZE--DESCRIPTION	ASTM	WT.	
1	1	L 4X4X3/8X2'-4 1/2" LG	A-36	23	
2	1	L 4X4X3/8X1'-11 3/8" LG	A-36	19	
3	1	L 4X4X3/8X2'-3" LG	A-36	22	
4	1	1 1/2" Ø U BOLT-HVY DUTY FIG. 101		1	
5	4	3/4" Ø X 2" LG-HVY. HEX. MACH BOLT W/LW	A307GRB	2	
6	1	FL 3/8" X 6" X 0'-8"	A-36	6	
TOTAL WT.				73 #	

102-215-08	712	1	REV	DWG SHT
CVI NO	CVI SHT			



ZONE: R-53	OP LD.: 78#	HYDRO. LD.: —
THERMAL LD.: VERT 117#	N-S: —	E-W: 46#
SEISMIC LD.: VERT 179#	N-S: —	E-W: 158#
CODE/CLASS: III/1	GROUP: I	Q.A. LEVEL: I
PAINT: I	PIPE CALC.: 8-14-156	STL. CALC.: 8-16-3201
PIPING STANDBY LIQUID SYSTEM CONTROL	REF. DWG. 180, SLC-4453-5 PIPING: M705 (H)	

1	DOCUMENT UPDATE	DATE 9/4/82	CHKD 10/28/81	SCALE NTS
REV NO	REVISION	DATE	CHKD	APVD
DWNA. Sayahani	CHKD 10/28/81	DATE 10/28/81		
SL/LDE				
ENGINEERING REVIEW				
MECH		CIVIL		
ELEC				
REVD		APVD	DATE	
H.R. Fager	10/28/81	10/28/81		
CHIEF DRAFTSMAN	CHIEF	MECH. ENGINEER		

WASHINGTON PUBLIC POWER SUPPLY SYSTEM		
HANFORD NO.2 S/U 10.0 PI		
MARK NO. SLC-4453-57		
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
W.O.3808	DWG. SLC-4453-57	REV 1



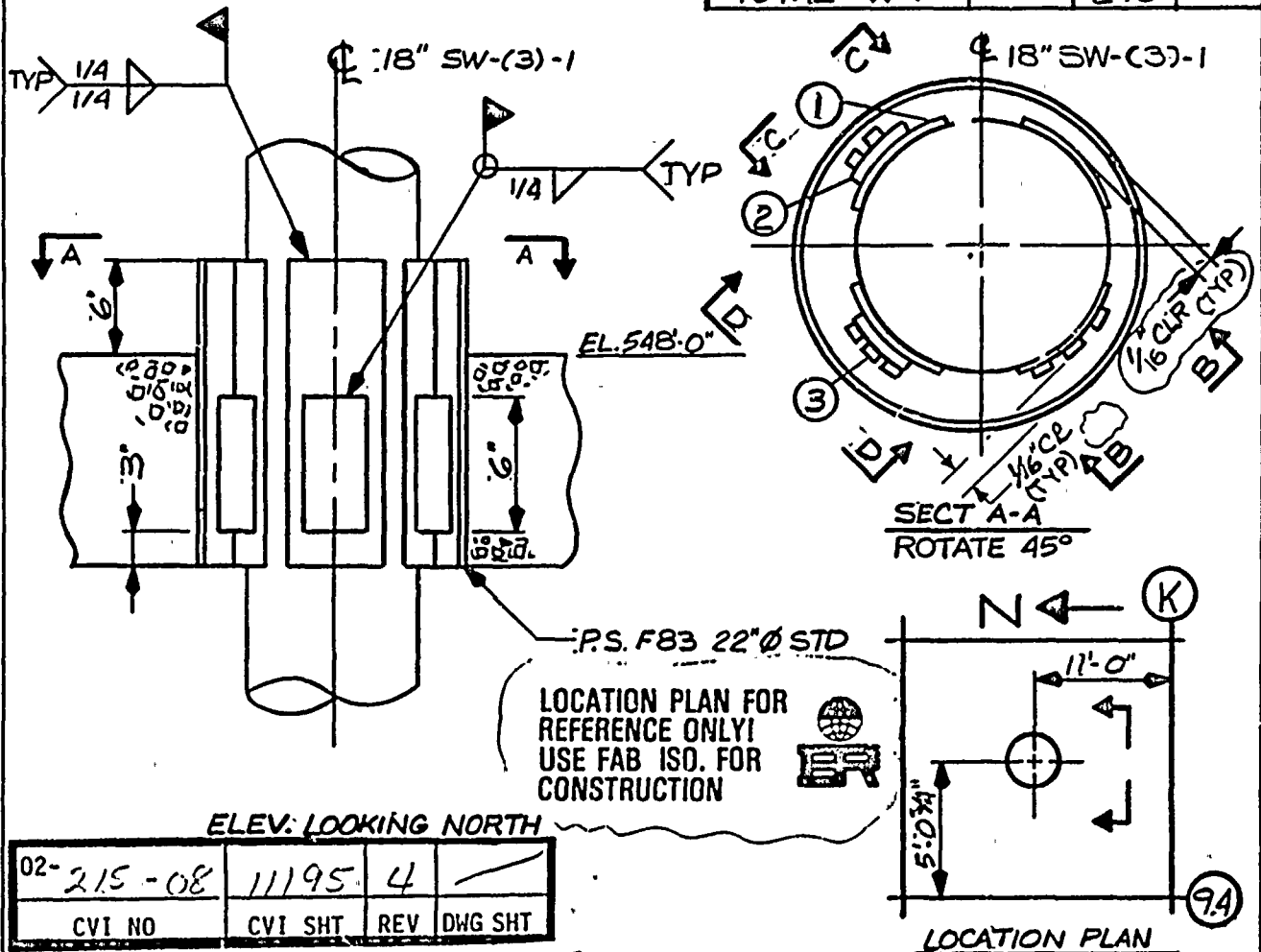






SUPERIOR REPROGRAPHICS 78-30K

ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	4	R 1"x11"x1'-6" (PAD) (BEND TO RAD OF 18" Ø PIPE)	A-36	22.4
2	2	R 9/16"x6"x0'-6" (PAD) (BEND TO RAD OF ITEM 1)	A-36	12
3	8	R 3/4"x7/8"x0'-6" (GRIND THK. TO SUIT)	A-36	10
TOTAL WT				246#

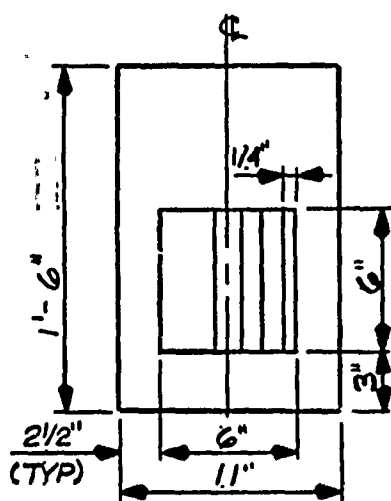


**INFORMATION ONLY**

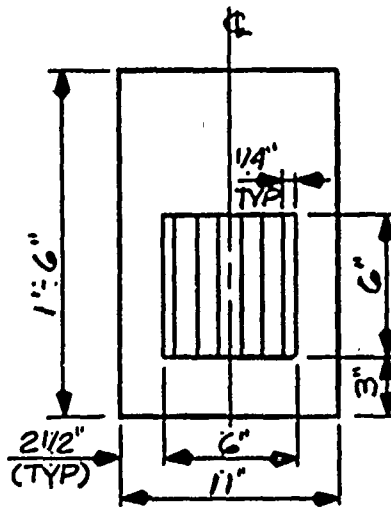
ZONE: R-61		OP. LD. F <sub>1</sub> =265# F <sub>2</sub> =499#	HYDRO. LD. =
THERMAL LD.: VERT. =		N-S =	E-W =
SEISMIC LD.: VERT. =		N-S = 3793#	E-W = 3808#
CODE/CLASS: 3/3	GROUP: 2	Q.A. LEVEL: I	
PAINT: I	PIPE CALC: 8.4.023	STL. CALC: 8.6.2381	
PIPING STAND-BY SYSTEM SERVICE WATER		REF. DWG. ISO. SW-250-52.54	
		PIPING: M-715	
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2 58.0 P5			
MARK NO.: SW-123			
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California			
W.O. 3808		DWG. SW-123 1 OF 2	REV 4

4	REV'D BY B & R	3/11/83	2P	1/18	MA	ST
3	REVISED IN ACCORDANCE W/WBG STATUS AS-BUILT	3/30/81	JM	KUM	MO	RH
REV NO	REVISION	DATE	OWN	CHKD	APVD	
DWG	CHKD KLM	SCALE NTS				
SL. NO.	DATE 7/8/81					
ENGINEERING REVIEW						
MECH	CIVIL					
ELEC						
REV'D	APVD	DATE				
H.P. Tanager	APVD	7/14/81				
CHIEF DRAFTSMAN	CHIEF MECH.	ENGINEER				

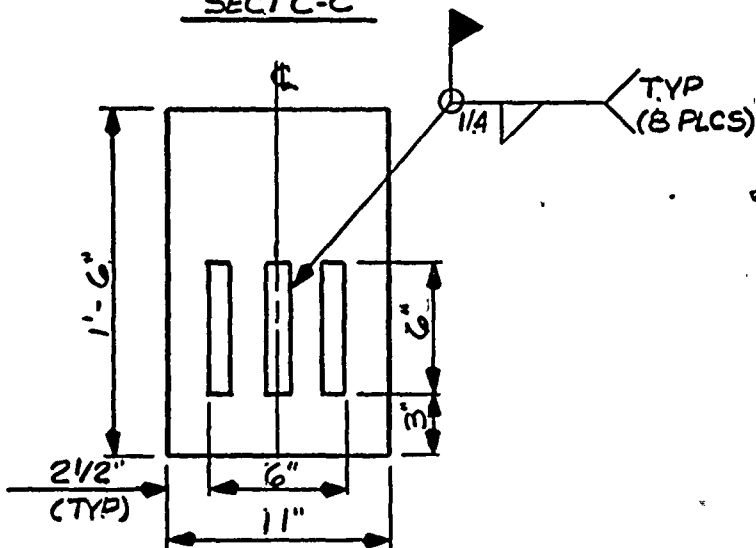




SECT. C-C



SECT. D-D



SECT. B-B

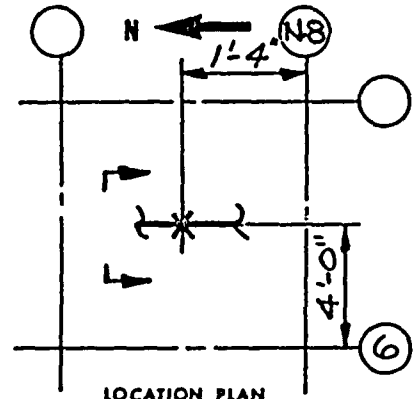
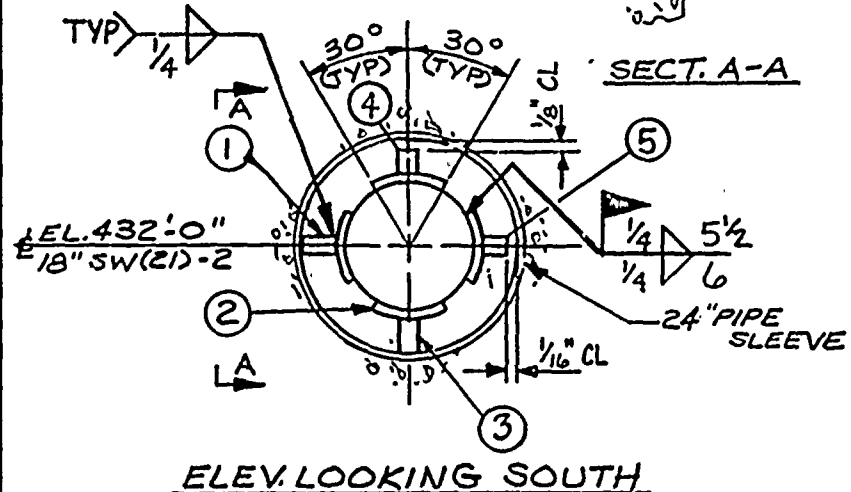
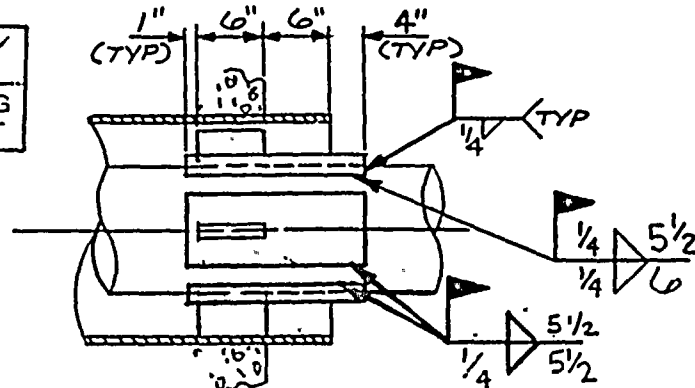
ALL DIMENSIONS IN INCHES UNLESS OTHERWISE SPECIFIED

4	REV'D BY B&R	3/14/83	2P	1/4"	1/4"	1/4"	1/4"
3	REVISED IN ACCORDANCE W/WBG STATUS AS-BUILT	5/30/81	1M	1/4"	1/4"	1/4"	1/4"
REV NO	REVISION	DATE	OWN	CHKD	APVD	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2 58.0 P5	
OWN	CHKD KLM	SCALE NTS	MARK NO. SW-123		BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California		
SL/DE	DATE 7/8/81	ENGINEERING REVIEW		W.O.3808 DWG. SW-123 2 OF 2			
MECH	CIVIL	APVD		REV 4			
ELEC		DATE 7/11/81					
REVD	CHIEF DRAFTSMAN	CHIEF MECH.	ENGINEER				



ITEM NO.	NO. REQ'D.	SIZE	DESCRIPTION	ASTM	WT.
1	1	1/2" X 2 1/8" X 6" LG PL		A36	2
2	4	9 5/8" X 3/4" PL X 1'-5" LG. ROLLED PLATE		A36	139
3	1	1/2" X 1 5/8" X 6" LG PL		A36	1
4	1	PL 1/2" X 1 7/8" X 0'-6"		A36	2
5	1	PL 1/2" X 7/8" X 0'-6"		A36	1
MATERIAL WT.				TOTAL	145 #

215-08	11040	1	/
CVI NO	CVI SHT	REV	DWG SHT



SECTION ONLY

BCBR-215-1493-

ZONE: R-13		SEIS. LOADS: VERT = 2026 <sup>#</sup> N-S = — E-W = 3,358 <sup>#</sup>		OP. LD. = 999 <sup>#</sup>		HYDRO. LD. = 999 <sup>#</sup>		THERM. LD. = 1000 <sup>#</sup> Ex: 1000 <sup>#</sup>	
CODE / CLASS: III/3		GROUP: 2		Q.A. LEVEL: I		PAINT: I		PIPE CALC. R.1465	
OWNER WASHINGTON PUBLIC POWER SUPPLY SYSTEM		PIPING SYSTEM STAND-BY SERVICE WATER							
PROJECT HANFORD NO.2		ENGINEER BURNS & ROE INC.							
CUSTOMER BOVEE & CRAIL / GERI		REFERENCE DWG. 150'SW-296-33-36H PIPING: M712							
MARK NO. SW-439						AM-184			



nps industries. inc.

CONTRACT 215

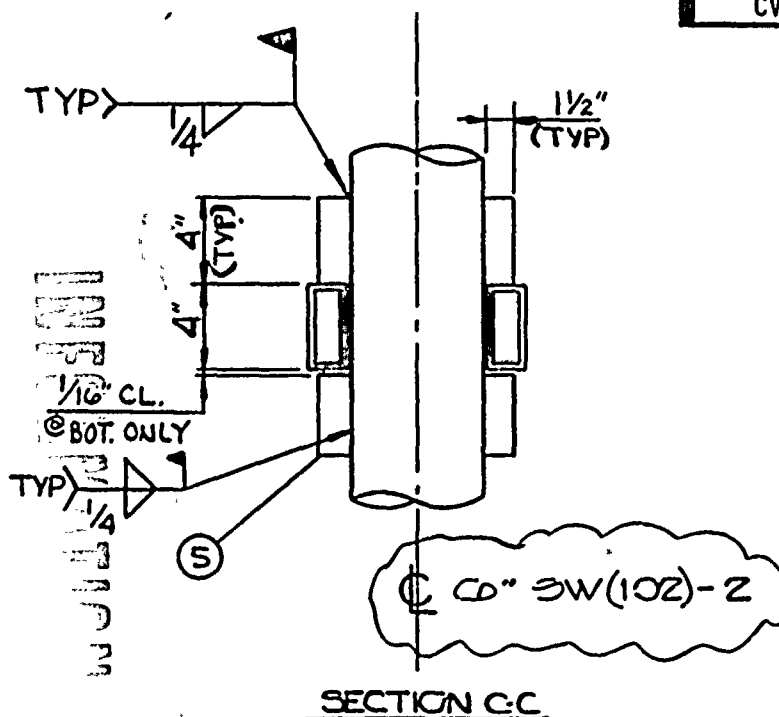
a subsidiary of  
nuclear power services, inc.  
26 broadway-new york, n.y. 10004

DRAWN	DATE	CHK'D	DATE	APP'D	DATE
MF	9-30-76	JJ.M.	10-17-76	JG	11-4-77
JOB NO.					
DWG. NO. SW-439				REV. 1	

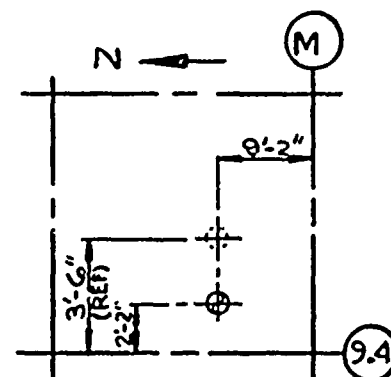


ITEM NO.	QUAN.	SIZE - DESCRIPTION	ASTM	WT.
1	2	TS 4 x 2 x .250 x 0'-11½" LG	A500GRB	17
2	1	TS 4 x 2 x .250 x 0'-3½" LG.	A500GRB	3
3	1	R ⅝" x 3" x 0'-5"	A 36	3
4	1	R ⅝" x 3" x 0'-7"	A 36	4
5	4	R 1" x 1½" x 0'-4" (LUG)	A 36	7
		TOTAL WT		34 #

02-215-68	13300	2	—
CVI HO	CVI SHT	REV	DWG SHT



**SECTION C-C**



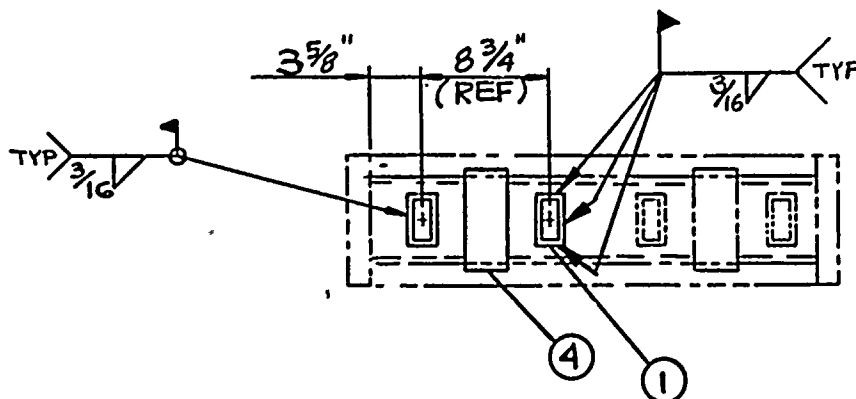
## LOCATION PLAN

2	DOCUMENT UPDATE	8/24/81	11/1/81	11/1/81
1	DOCUMENT UPDATE	8/24/81	11/1/81	11/1/81
0	RELEASE FOR FABRICATION	7/13/81	10/1/81	11/1/81
REV NO	REVISION	DATE	OWN	APVD
DWN	CHKD KLM	DATE	NTS.	
SL/LDE		DATE	7/17/81	
ENGINEERING REVIEW				
MECH	CIVIL			
ELEC				
REV	APVD	DATE		
H.B. Frazier	H.B. Frazier	7/23/81		
CHIEF DRAFTSMAN	CHIEF	MECH	ENGINEER	

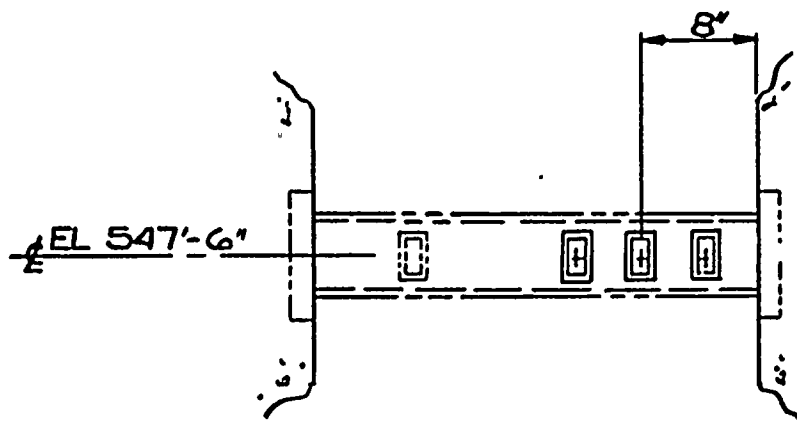
ZONE: R-63	OP.LD.: 899#	HYDRO.LD.: 899#
THERMAL LD.: VERT: — N-S — E-W: —		
SEISMIC LD.: VERT: 1798# N-S: 1900# E-W: 1796#		
CODE/CLASS: 1/3	GROUP: 2	Q.A. LEVEL: I
PAINT: I	PIPE CALC: 8-42-3001	STL. CALC: 8-16-2762
PIPING STAND BY	REF. DWG. 180	SW-251-38-46
SYSTEM: SERVICE WATER		PIPING: M-714
WASHINGTON PUBLIC POWER SUPPLY SYSTEM		
HANFORD NO.2		
MARK NO. SW-946N		
BURNS AND ROE, INC.		
Engineers and Constructors		
New Jersey • New York • Connecticut • California		
W.O.3808	DWG. SW-946N	SHT. 1 OF 3
		2







SECT. A-A



SECT. B-B

2	DOCUMENT UPDATE	8/24/81	TLC	TH	MMO
1	DOCUMENT UPDATE	8/17/81	TH	TH	MMO
0	RELEASED FOR FABRICATION	1/13/81	TLC	TH	MMO
REV NO	REVISION	DATE	OWN	CHKD	APVD
DWN	CHKD KLM	SCALE	NTS		
SL/DE	DATE	7/17/81			
ENGINEERING REVIEW					
MECH	CIVIL				
ELEC					
REVD	APVD	DATE			
H.R. Frazier	J.B. King	7/23/81			
CHIEF DRAFTSMAN	CHIEF MECH. ENGINEER				

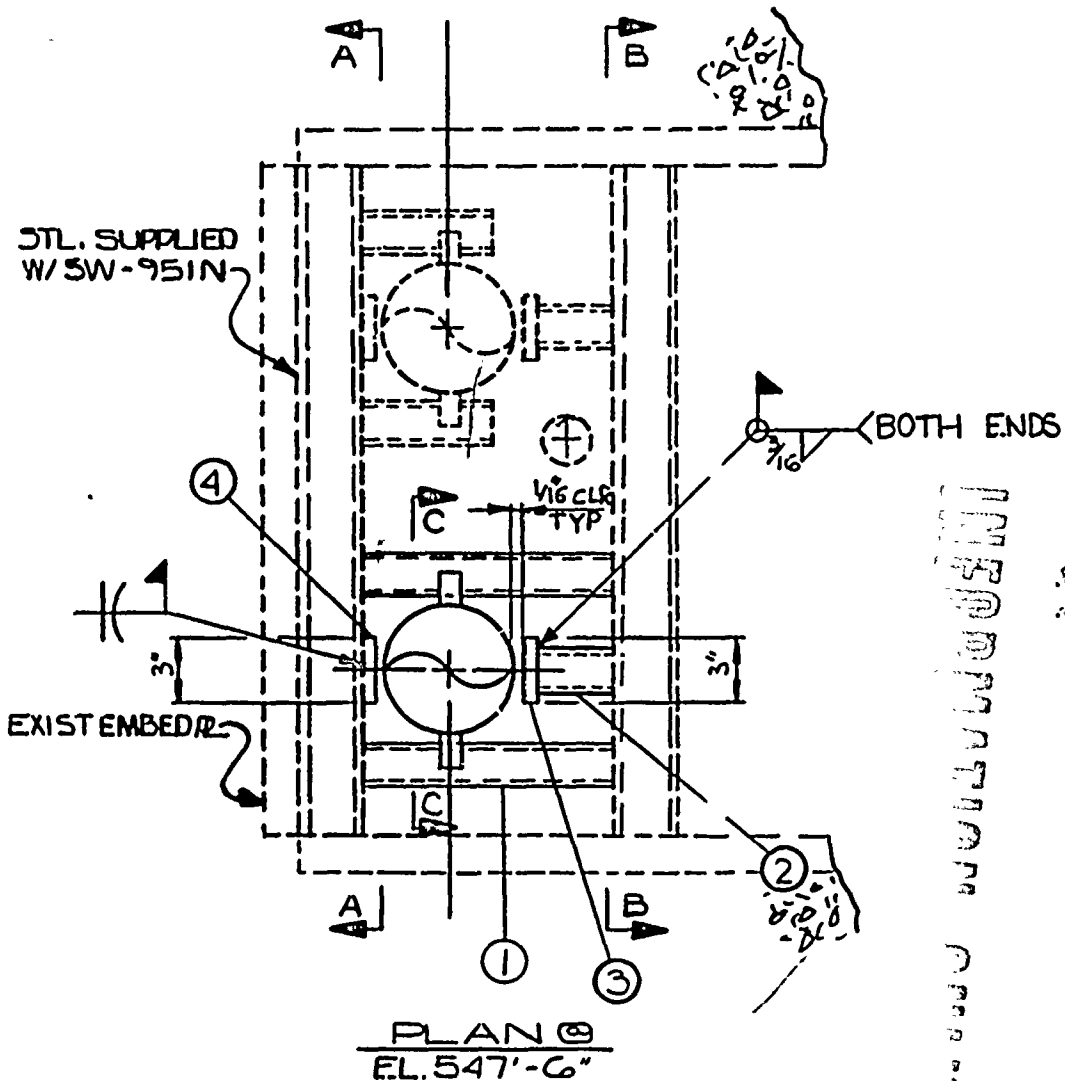
WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2

MARK NO. SW-946N

BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.0.3808 DWG. SW-946N SHT 20F3 REV 2





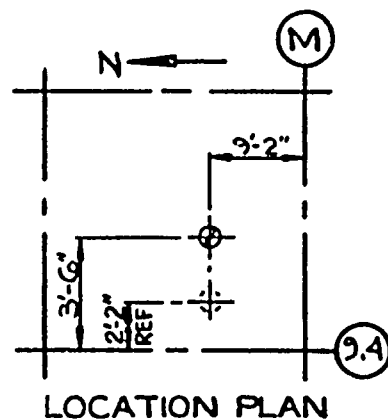
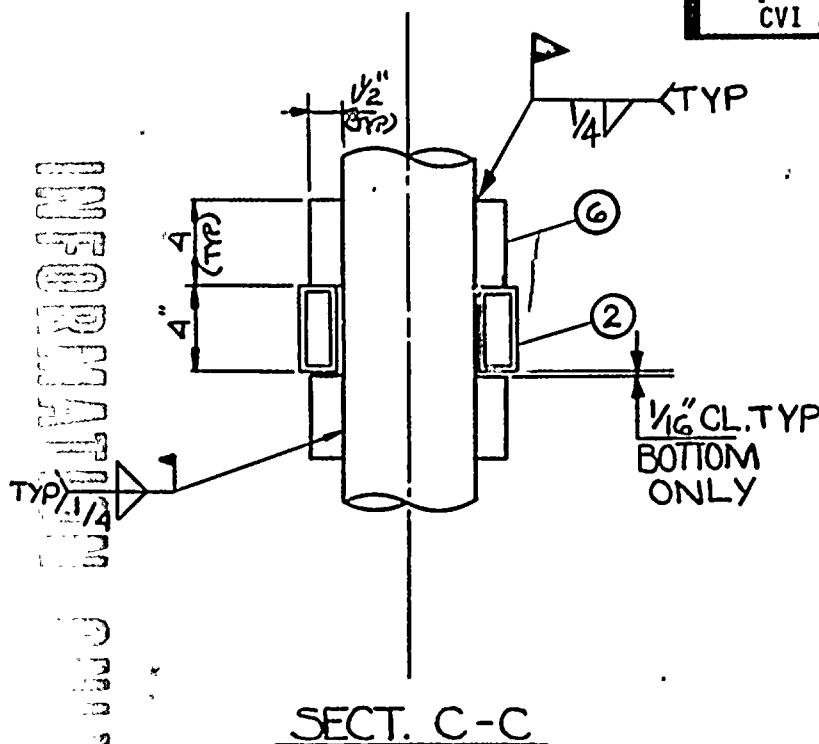
INTERSECTION

2	DOCUMENT UPDATE	9/24/81	TC	1/1	4	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2
1	DOCUMENT UP-DATE	9/1/81	TC	1/1	4	
0	RELEASE FOR FABRICATION	7/13/81	JG	KLM	RKG	
REV NO	REVISION	DATE	OWN	CHKD	APVD	MARK NO. SW-946N
DWN	CHKD KLM	SCALE NTS				
SL/CD	DATE 7/17/81	ENGINEERING REVIEW				BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California
MECH	CIVIL					
ELEC					W.O.3808 DWG. SW-946N SHT 3 OF 3	REV 2
REVD	APVD	DATE				
H.R. Trogue	APVD	7/23/81				
CHIEF DRAFTSMAN	CHIEF	MECH	ENGINEER			



ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	WT.	
1	2	TS 6 x 3 x .375 x 2'-7" LG.	A500GRB	100	
2	2	TS 4 x 2 x .250 x 0'-6" LG.	A500GRB	9	
3	1	TS 4 x 2 x .250 x 0'-3 1/2" LG.	A500GRB	3	
4	1	PL 5/8" x 3" x 0'-5"	A36	3	
5	1	PL 5/8" x 3" x 0'-7"	A36	4	
6	4	PL 1" x 1 1/2" x 0'-4" (LUG)	A36	7	
TOTAL WT				126	#

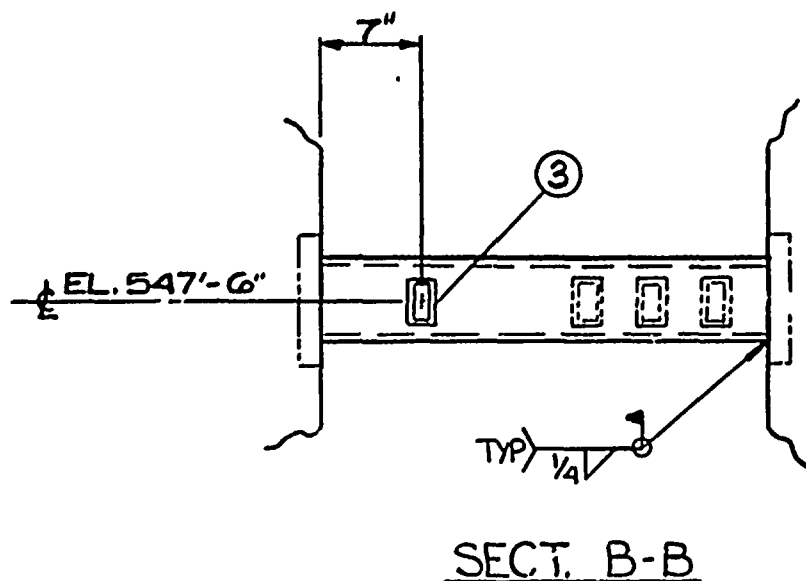
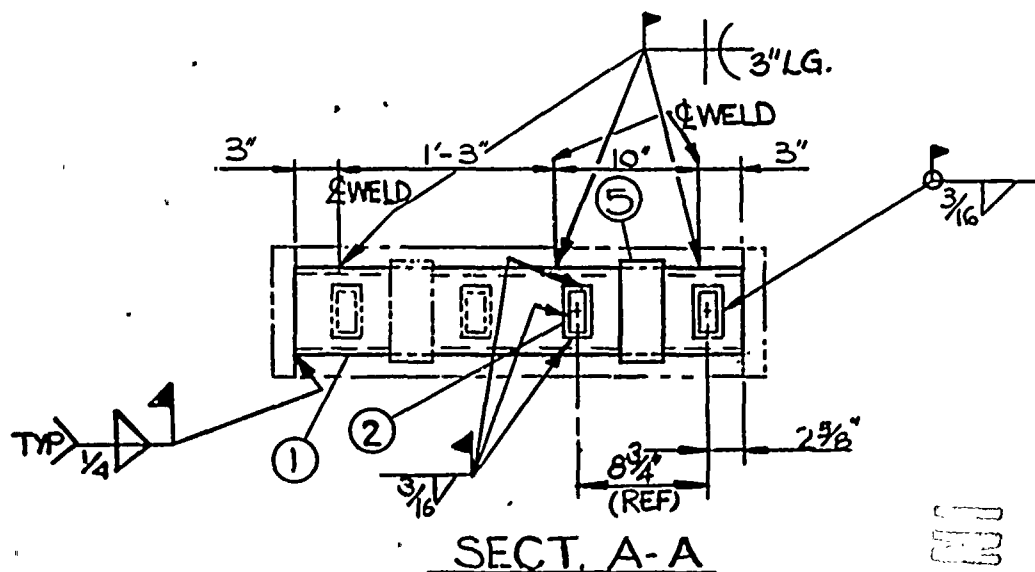
102-215-08	13287	2	—
CVI NO	CVI SHT	REV	DWG SHT



2	DOCUMENT UPDATE	8/24/81	RL	16	MOD
1	DOCUMENT UPDATE	8/1/81	RL	16	MOD
0	RELEASE FOR FABRICATION	7/13/81	RL	16	MOD
REV NO	REVISION	DATE	CHKD	ENGRD	APVD
OWN	CHKD	DATE	SCALE	NTS	
BL/LOE	ENGINEERING REVIEW				
MECH	CIVIL				
ELEC					
REVD	APVD	DATE			
H.R. Teague	CHIEF DRAFTSMAN	7/23/81	CHIEF	MECH.	ENGINEER

ZONE: R-63	OP.LD.: 911 #	HYDRO.LD.: 911 #
THERMAL LD.: VERT: —	N-S: —	E-W: —
SEISMIC LD.: VERT: ±1822 #	N-S: ±2319 #	E-W: ±1662 #
CODZ/CLASS: II/3	GROUP: 2	Q.A. LEVEL: I
PAINT: I	FPZ CALC: 8-42-300	STL CALC: 8-16-2762
PIPING STAND BY SYSTEM SERVICE WATER	REF. DWG. 100-5W-295-43-63	PIPING: M-714
WASHINGTON PUBLIC POWER SUPPLY SYSTEM		
HANFORD NO.2		
MARK NO. SW-951N		
BURNS AND ROE, INC.		
Engineers and Constructors		
New Jersey • New York • Connecticut • California		
W.O.3808	DWG. SW-951N	SHT. 1 OF 3
		REV 2

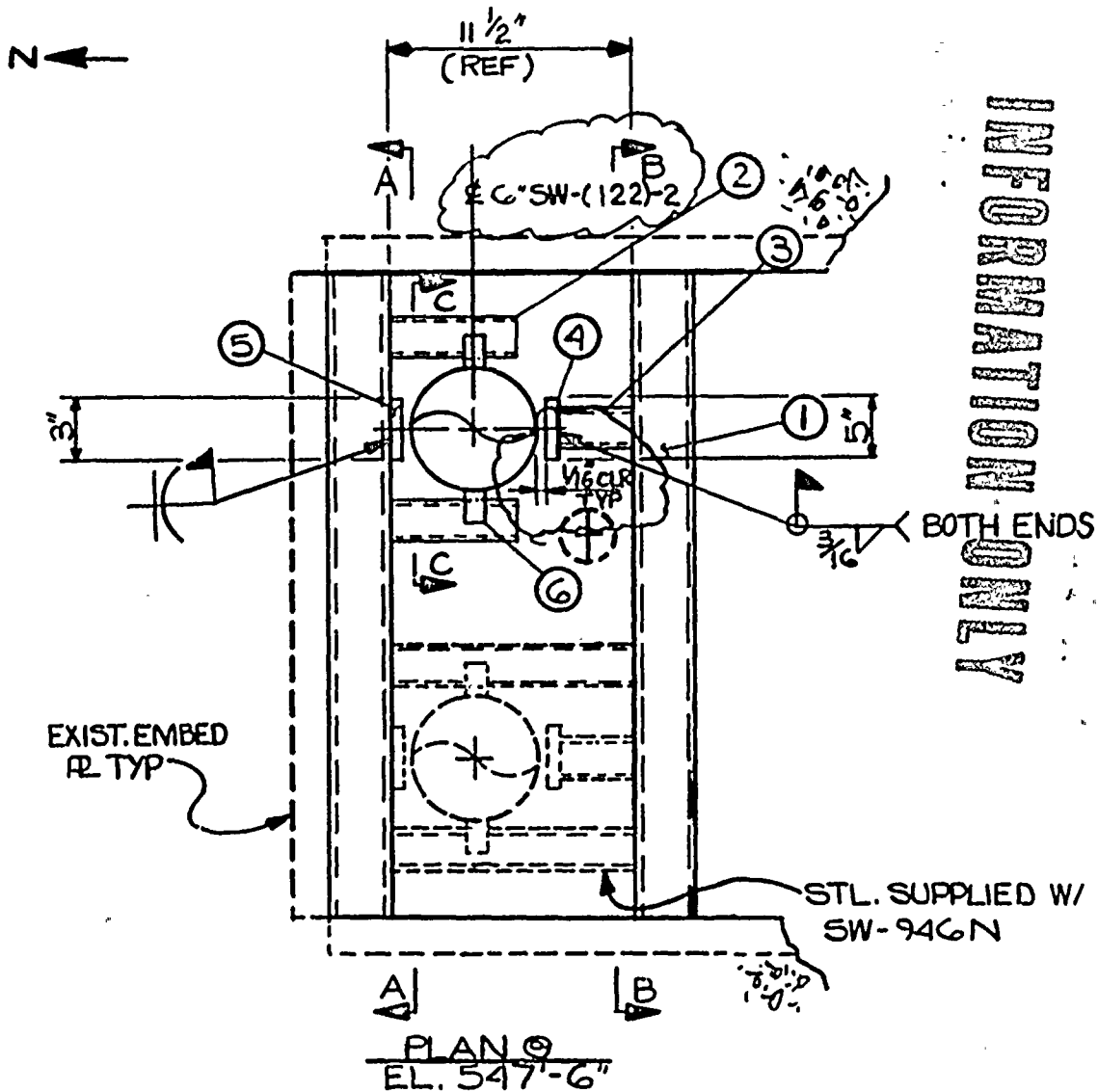




2	DOCUMENT UPDATE	8/24/81	71C	EP	MCO
1	DOCUMENT UP-DATE	8/7/81	71C	EP	MCO
0	RELEASED FOR FABRICATION	7/13/81	JG	WV	EP
REV NO	REVISION	DATE	OWN	CHKD	APVD
DWN	CHKD	KLM	SCALE	NTS	
BL/LDE	DATE	7/17/81			
ENGINEERING REVIEW					
MECH	CIVIL				
ELEC					
REVD	APVD	DATE			
H.R. Frazier	CHIEF	MECH	7/23/81	ENGINEER	
WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2					
MARK NO. SW-951N					
BURNS AND ROE, INC. Engineers and Constructors New Jersey • New York • Connecticut • California					
W.O.3808		DWG. SW-951N		SHT 2 OF 3	REV 2







INFORMATION ONLY

2	DOCUMENT UPDATE	8/24/81	11C	11C	11C
1	DOCUMENT UP-DATE	8/17/81	11C	11C	11C
0	RELEASE FOR FABRICATION	7/13/81	11C	11C	11C
REV NO	REVISION	DATE	DWN	CHKD	APVD
DWN	CHKD	KLM	SCALE	INTS	
SL/IDE	DATE	7/17/81			
ENGINEERING REVIEW					
MECH	CIVIL				
ELEC					
REV'D	APVD	DATE			
H.R. Teague	J.B. King	7/23/81			
CHIEF DRAFTSMAN	CHIEF MECH. ENGINEER				

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD NO.2

MARK NO. SW-951N

BURNS AND ROE, INC.  
Engineers and Constructors  
New Jersey • New York • Connecticut • California

W.O.3808

DWG. SW-951N

SHT 30F3

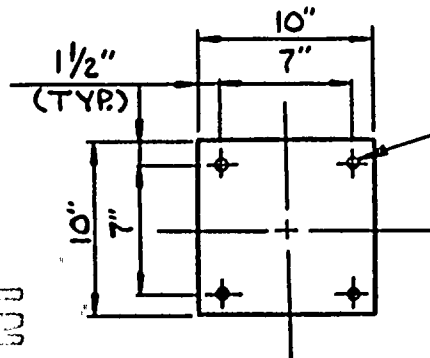
REV 2



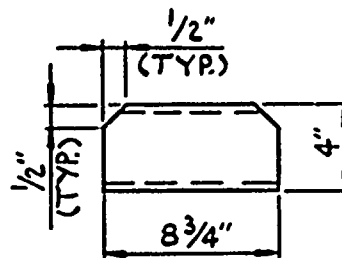
ITEM NO.	QUAN.	SIZE-DESCRIPTION	ASTM	WT.
1	1	TS 4 x 4 x .250 x 2'-2" LG.	A500GRB	26
2	2	TS 4 x 4 x .250 x 2'-0 1/2" LG.	A500GRB	49
3	2	TS 2 x 4 x .250 x 1'-4 1/16" LG.	A500GRB	12
4	2	TS 4 x 4 x .250 x 0'-8 3/4" LG. (BEVEL PER DETAIL 4)	A500GRB	16
5	1	TS 4 x 4 x .250 x 4'-2" LG. (CUT TO SUIT)	A500GRB	50
6	1	TS 4 x 4 x .250 x 1'-1 5/16" LG.	A500GRB	14
7	2	2 3/4" x 10" x 0'-10" (SEE DETAIL 7)	A-36	21
8	8	5/8" $\phi$ x 1 3/4" LG. HVY HEX MACH BOLTS w/LW	A307GRB	3

TOTAL WT. 191 LBS

02-215-08	13754	0	/
CVI NO	CVI SHT	REV	DWG SHT

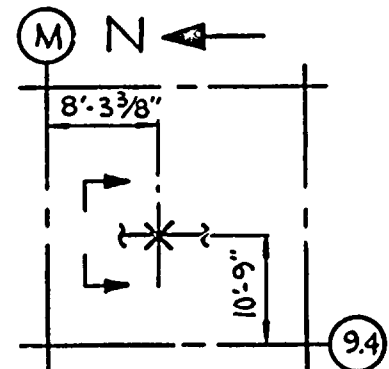


DETAIL 7



DETAIL 4

(TYP)



LOCATION PLAN

ZONE: R-53	OP.LD.: 736#	HYDRO.LD.: 736#
THERMAL LD.: VERT: —	N-S: —	E-W: —
SEISMIC LD.: VERT: $\pm 1472$ #	N-S: —	E-W: $\pm 2020$ #
CODE/CLASS: III/3	GROUP: 2	Q.A. LEVEL: I
PAINT: I	PIPE CALC.: —	STL. CALC.: 8.16.2761
PIPING STAND BY SYSTEM SERVICE WATER	REF. DWG. ISO.: SW-295-43.68 PIPING: M-715	

RELEASE FOR FABRICATION	7/8/81	REV	1/2	REV
REV NO	REVISION	DATE	CHKD	APVD
DWN R. Williams	CHKD	SCALE NTS.		
SL/LDE	DATE 7/21/81			
ENGINEERING REVIEW				
MECH	CIVIL			
ELEC				
REVD	APVD	DATE		
H.R. T. [Signature]	[Signature]	7/23/81		
CHIEF DRAFTSMAN	CHIEF MECH	ENGINEER		

WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
HANFORD 10.2

MARK NO. SW-950N

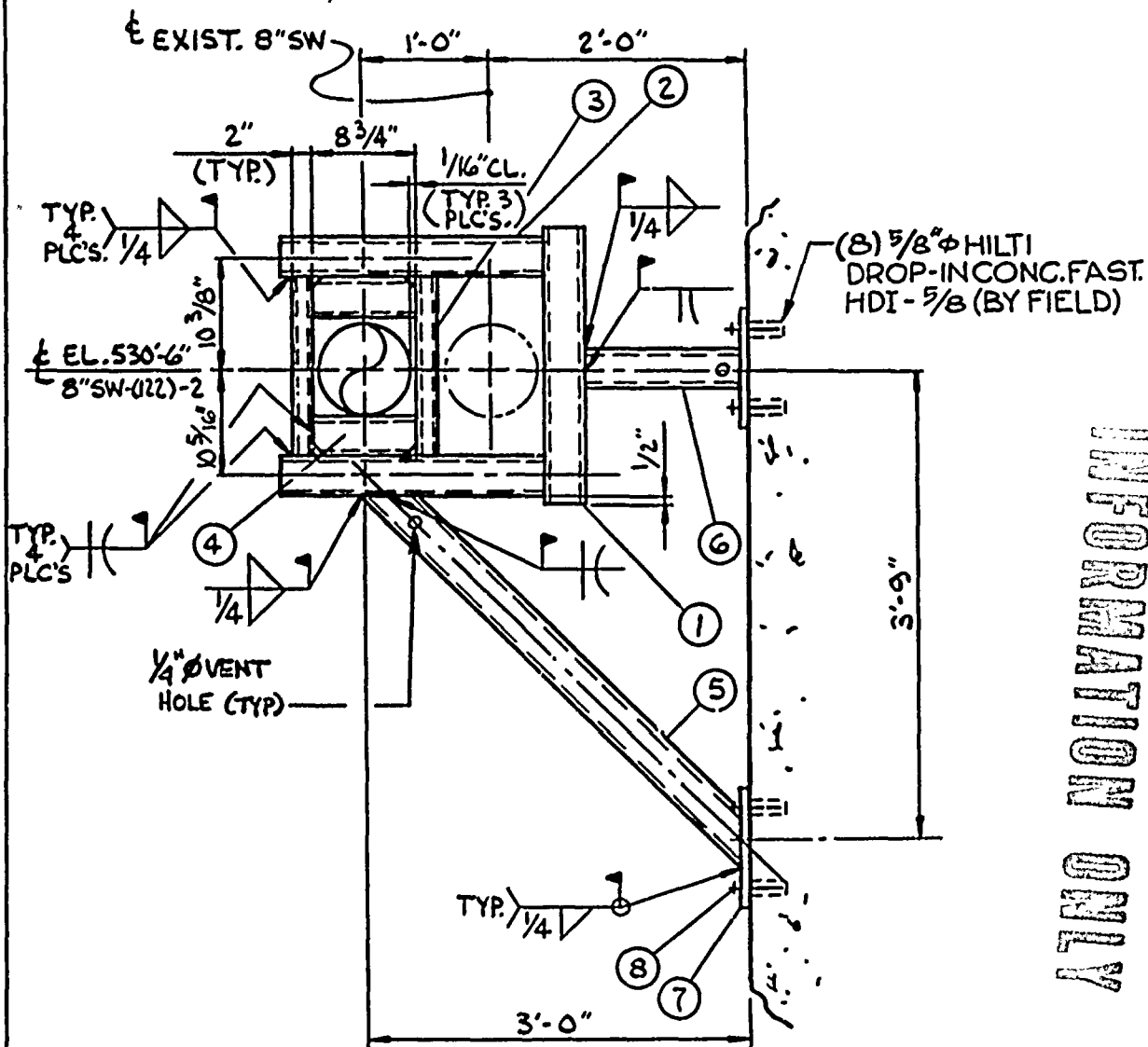
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SHT. 1 OF 2

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INFORMATION ONLY

**ELEVATION**  
LOOKING SOUTH

<input type="radio"/> RELEASE FOR FABRICATION		7/8/81	RW	PL	WASHINGTON PUBLIC POWER SUPPLY SYSTEM HANFORD NO.2	
REV NO	REVISION	DATE	DWN	CHKD	APVD	
DWN <i>R. Williams</i>	CHKD <i>B</i>	SCALE N.T.S.	MARK NO. <b>SW-950N</b>			
BL/LDE <i>[Signature]</i>	DATE <b>7/21/81</b>	ENGINEERING REVIEW				
MECH	CIVIL					<b>BURNS AND ROE, INC.</b> Engineers and Constructors New Jersey • New York • Connecticut • California
ELEC						
REVD <i>H.R. Trogue</i>	APVD <i>[Signature]</i>	DATE <b>7/23/81</b>	<b>W.O.3808</b>		<b>DWG. SW-950</b>	SHT. 2 OF 2
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**WASHINGTON PUBLIC POWER  
SUPPLY SYSTEM**

**WASHINGTON PUBLIC POWER SUPPLY SYSTEM**

**NUCLEAR PLANT 2**

# **1994 ANNUAL REPORT**

**JANUARY 1 to DECEMBER 31, 1994**

**RADIOLOGICAL  
ENVIRONMENTAL  
MONITORING PROGRAM**

**Prepared by**

**Washington Public Power Supply System**

**and**

**Teledyne Brown Engineering Environmental Services**





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## 1.0 EXECUTIVE SUMMARY

The Washington Public Power Supply System Radiological Environmental Monitoring Program activities conducted during 1994 are discussed in this report. The objective of the Radiological Environmental Monitoring Program (REMP) is to provide data for evaluating radiological impact of Plant 2 operations on the environment in the Airborne, Direct Radiation, Waterborne, and Ingestion pathways as specified by the Offsite Dose Calculation Manual (ODCM) for Plant 2. Plant 2, a 3323 MWth (1150 MWe) boiling water reactor electrical generation plant, achieved initial criticality on January 19, 1984.

To accomplish the REMP objective of determining whether Plant 2 operations affected the environment, samples of air, water, milk, soil, sediment, fish and garden produce were collected and analyzed for radionuclides specific to plant operations throughout the year. Radiation levels were also monitored continuously during 1994 with thermoluminescent dosimeters (TLDs).

The samples were collected at preselected areas near the plant and at other locations which could be impacted by Plant 2 effluents. This information was compared to samples taken in areas that were unlikely to be affected by plant operations. The 1994 REMP data was also compared to data collected during previous years of plant operation and also compared to the data collected prior to initial plant operation.

Most of the results of samples collected by the REMP during 1994 were below detection levels. Some analyses, such as gross beta in air and water, were above the detection level for nearly all samples. This is due to the low detection limit for the gross beta analysis and also to the abundance of naturally occurring beta-emitting radionuclides in the environment. Other results above detection levels, such as cesium-137 in soil and sediment, reflect the effect of past Hanford activities or fallout from Chernobyl and past nuclear weapons testing.

The REMP analytical results and TLD results were demonstrated to be accurate through intercomparison programs which are provided as part of the quality assurance activities conducted during 1994. Such intercomparisons tested the performance of the Supply System monitoring program to other monitoring programs using known radioactive standards. The Supply System REMP performed well in the Environmental Measurements Laboratory (EML) and Environmental Protection Agency Intercomparison Studies conducted during 1994.

During 1994, data collected by the REMP remained consistent with the environmental data collected during the preoperational period and the prior operational years. Based on the data, no significant new trends or changes in the environmental levels around the plant were evident during 1994.

## 2.0 DEFINITIONS

**Airborne Activity Sampling:** Continuous sampling of air through the collection of particulates and radionuclides on filter media.

Periodic soil samples are collected for gamma isotopic analysis to provide information on deposition to the soil from airborne releases.

**Alpha Particle ( $\alpha$ ):** A charged particle emitted from the nucleus of an atom having a mass and charge equal in magnitude of a helium nucleus.

**Becquerel (Bq):** One disintegration per second. One picocurie (pCi) equals 0.037 becquerel.

**Beta Particle ( $\beta$ ):** Charged particle emitted from the nucleus of an atom, with a mass and charge equal in magnitude to that of an electron.

**Blank Sample:** A sample of the same media as the field sample being analyzed but without the radionuclide(s) being measured. It enables correction for the inherent sample background.

**Composite Sample:** A series of single collected portions (aliquots) analyzed as one sample. The aliquots making up the sample are collected at time intervals that are very short compared to the composite period.

**Control Station:** A background sampling location, i.e., a location not likely to be affected by plant effluents due to its distance and/or direction from Plant 2.

**Counting Error:** An estimate of the two-sigma uncertainty associated with the sample results based respective count times.

$$\pm 1 - 2\sqrt{\left(\frac{\text{Sample CPM}}{\text{Count Time}} + \frac{\text{Bkg Cpm}}{\text{Count Time}}\right)}$$

**Curie (Ci):**  $3.7 \times 10^{10}$  disintegrations per second, or  $2.22 \times 10^{12}$  disintegrations per minute.

**Direct Radiation Monitoring:** The measurement of radiation dose at various distances from the plant is assessed through the use of thermoluminescent dosimeters and pressurized ionization chambers.

**DOH:** Washington State Department of Health.

**EFSEC:** Energy Facility Site Evaluation Council.

**FFTF:** U.S Department of Energy's Fast Flux Test Facility near Plant 2.

**Flow Proportional Sampling:** Sample collection volume or frequency determined as a function of the flow rate of the water being sampled.

**Grab Sample:** A single discrete sample drawn at one point in time.

**Indicator Station:** A sampling location that could be affected by plant effluents due to its proximity and/or direction from Plant 2.

**Ingestion Pathway Monitoring:** The ingestion pathway includes milk, soil, fish, garden produce. Also sampled (under special circumstances) are other media such as vegetation and animal products such as eggs and meat when additional information about particular radionuclides is needed.

**Lower Limit of Detection (LLD):** The smallest concentration of radioactive material in a sample that will yield a net count (above system background) that will be detected with 95% probability with a 5% probability of a false conclusion that a blank observation represents "real" signal.

$$LLD = \frac{4.66Sb}{(2.22 * Vol * Eff * Yield * e^{-\lambda t})}$$

where LLD is the "a priori" or 'before-the-fact' measurement and not "a posteriori" or 'after-the-fact' measurement.

**Mean:** The average, i.e., the sum of results divided by the number of results.

**Microcurie:**  $3.7 \times 10^4$  disintegrations per second, or  $2.22 \times 10^6$  disintegrations per minute.

**Milliroentgen (mR):** 1/1000 Roentgen; a unit of exposure to X- or gamma radiation.

**NIST:** National Institute of Standards and Technology.

**NRC:** U.S. Nuclear Regulatory Commission.

**ODCM:** Offsite Dose Calculation Manual, which contains the program requirements formerly contained in the Technical Specifications.

**Picocurie (pCi):**  $1 \times 10^{-12}$  Curie or 2.22 disintegrations per minute; one millionth of a microcurie.

**REMP:** Radiological Environmental Monitoring Program.

**Range:** The difference between the smallest and largest results.

**Restricted Area:** Any area to which access is controlled for purposes of protection of individuals from exposure to radiation and radioactive materials.

**Results:** The results of sample collection are discussed and interpreted by comparing them to similar measurements made during the preoperational and previous operational periods and to the detection capabilities associated with the current methods of analysis.

**Roentgen:** Unit of exposure to X- or gamma ( $\gamma$ ) radiation in air.

**Site Certification Agreement (SCA):** The Plant 2 licensing agreement with the State of Washington.

**Spike Sample:** A sample containing a known concentration of the radionuclide(s) being measured.

**Standard Deviation:** A measure of the scatter of a set of observations (or samples) around their mean value. Indicated by ( $\sigma$ ).

**Standard Error of the Mean:** An estimate of the uncertainty associated with the mean of observation (or sample) averages.

$$SE = \sqrt{\left(\frac{S^2}{n}\right)}$$

where  $S^2$ , the variance is

$$S_m^2 = \frac{1}{(n-1)} \sum^n (X_i - \bar{X})^2$$

**SWTF:** Sanitary Waste Treatment Facility; sanitary waste processing facility for Plant 2 and WNP-1 and WNP-4 sites.

**TEDA:** Triethylene diamine

**TLD:** A thermoluminescent dosimeter that contains a phosphor which stores energy from exposure to radiation and emits that energy in the form of light when heated.



### 3.0 INTRODUCTION

#### 3.1 Site Description

The Washington Public Power Supply System's Nuclear Plant 2 is located in a sparsely populated shrub-steppe region within the Department of Energy's Hanford Site in southeastern Washington. The plant is located approximately three miles west of the Columbia River and is surrounded on all sides by uninhabited desert land. The nearest population centers are Richland, Kennewick and Pasco, which are 12 miles south, 21 miles southeast, and 18 miles southeast, respectively. The nearest privately-owned lands are located approximately four miles ENE of the plant, across the Columbia River. Given the prevailing wind directions, shown in the 1994 wind frequency distribution in Figure 3-1, the focus of REMP sampling is the farming region east of the river.

Because Plant 2 is located on the Hanford Site, other potential sources of radioactive materials are in close proximity to Plant 2. For this reason, sampling locations near the plant (near-plant locations) provide useful information for separating the potential environmental effects of Plant 2 from the effects of the other sources on the Hanford Site.

#### 3.2 Program Background

The REMP is designed to conform to the regulatory guidance of the Nuclear Regulatory Commission (NRC) as provided by Regulatory Guides 4.1<sup>(1)</sup> and 4.8<sup>(2)</sup>, including the Radiological Assessment Branch Technical Position<sup>(3)</sup>.

The quality assurance aspects of the program and the thermoluminescent dosimetry are conducted in accordance with Regulatory Guides 4.15<sup>(4)</sup> and 4.13.<sup>(5)</sup> The REMP also must adhere to the requirements of the Washington Energy Facility Site Evaluation Council<sup>(6)</sup>, the Plant 2 Technical Specifications<sup>(7)</sup> and the Offsite Dose Calculation Manual (ODCM)<sup>(8)</sup>. These requirements cover not only the environmental sampling and sample analysis aspects of the program, but also the reporting and quality assurance requirements of the program.

The preoperational phase of the program, which lasted from March 1978 until January 1984, the date of initial criticality, provided a baseline of environmental data. Variability in the background levels of radioactivity due to differences in geologic composition, Chernobyl and nuclear weapons test fallout, meteorological conditions and seasonal changes is reflected in that preoperational data.

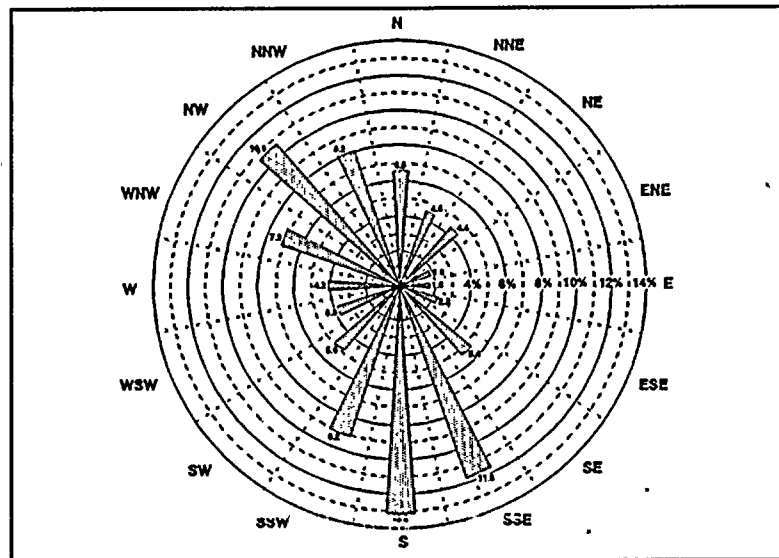


Figure 3-1 Average Wind Direction (percent) During 1994

REMP environmental samples have always been analyzed by a contract analytical laboratory. Since June 1986, Teledyne Brown Engineering Environmental Services (formerly Teledyne Isotopes, Inc.) in Westwood, New Jersey has performed the analysis of REMP samples. The thermoluminescent dosimeters used in the REMP to assess the direct radiation are processed by the Supply System External Dosimetry Laboratory within the Health Physics Department.

### 3.3 Program Objectives

The REMP provides a mechanism for determining whether the levels of radioactivity in the plant environs are within established limits and for ensuring that the accumulation of radionuclides in the environment will not become significant as a result of plant operations.

While in-plant monitoring programs are used to ensure that 10CFR20<sup>(9)</sup> and 10CFR50<sup>(10)</sup> criteria for releases of radioactive effluents are met, the REMP provides supplemental verification that the concentrations of radionuclides in the environment are not greater than anticipated.

Any radiological effect of Plant 2 on the environment must be distinguished from the normal variation in background radiation levels and from the effects of other sources of radioactive effluents in the area.

The monitoring results obtained during each year of the plant's operation are compared to the preoperational data and to data from previous operating years, to determine whether a significant accumulation of plant-produced radionuclides has occurred in the environment. Quarterly averages of the results are also compared to the NRC non-routine reporting levels listed in the ODCM. In addition to evaluating the environmental concentrations against federal standards or limits, the REMP also compares the results to state standards.<sup>(11)(12)(13)</sup> The results are discussed and interpreted by comparing them to similar measurements made during the preoperational and previous operational periods and to the detection capabilities associated with the current methods of analysis. The quality assurance and quality control aspects of the program are also discussed in this report.

## **4.0 PROGRAM DESCRIPTION**

The requirement for the Radiological Environmental Monitoring Program (REMP) is defined by the WNP-2 Offsite Dose Calculation Manual (ODCM). The sampling plan to meet ODCM requirements is presented as Table 4-1 in this report and provides a summary of the sample locations, collection frequency and types of analyses performed. The methods of sampling and sampling frequencies utilized in the program have been determined by such factors as the half-lives and major exposure pathways for the radionuclides potentially released from the plant to the surrounding environment.

### **4.1 Sample Locations**

Eighty sample locations were included in the 1994 monitoring program. Seventy-three indicator and two control, i.e. background, locations were within 10 miles (16 kilometers) of Plant 2. Three additional control stations and two indicator stations were outside the 10-mile radius from the plant. Sample stations are listed in Table 4-2 by meteorological sector, sample media and approximate distance from the plant. The numbers and locations of sample stations are based not only on factors such as population distribution and meteorological conditions, but also on station accessibility, security throughout the year and the requirements of applicable regulations. Other factors, such as the need to monitor locations which potentially could be impacted by Plant 2 operations, influence the location of REMP sampling sites.

The REMP sampling locations listed in Tables 4-1 and 4-2 are shown in Figure 4-1. Figure 4-3 provides a more detailed map of sampling locations in the Sunnyside/Grandview area. Figure 4-4 shows the locations of the storm drain (Station 101), the Sanitary Waste Treatment Facility (Station 102) and the containerized storage area (Station 118), which are special interest stations.

### **4.2 Land Use Census**

The land use census for areas within 5 miles of Plant 2 was performed in August. The objectives of the land use census are to identify the locations of the nearest milk animal, residence, and garden greater than 50 m<sup>2</sup> (500 ft<sup>2</sup>) producing broadleaf vegetation and to determine whether any site located during the census has a calculated dose or dose commitment greater than the sites currently monitored for the same exposure pathway. If a new location with a higher dose commitment is found, routine sampling of that dose pathway would be initiated at that new site.

The results of the 1994 land use census within 5 miles of Plant 2 are given in Table 4-3. No significant changes from the 1993 land use census were observed. No milk animals are located within the 5-mile radius. The closest milk locations were at 7.2 miles ESE and 8.3 miles SE.

### 4.3 Sampling Methods

Environmental samples were collected according to the schedule in Table 4-1. All samples were collected by Supply System personnel. Documented procedures for sample collection and TLD analyses are in the Supply System's Health Physics Instruction (HPI) Manual. The sample analyses procedures are prepared and maintained by the analytical contractor and reviewed by the Supply System prior to implementation.

The following sections describe the sampling and preparation methods.

#### 4.3.1 Direct Radiation

During 1994, thermoluminescent dosimeters (TLDs) were used to determine the direct radiation levels at fifty-six (56) monitoring locations listed in Table 4-1. No new TLD locations were added to the program this year, however, Station 55 which consists of an annual and quarterly TLD and is located near the Department of Energy's 300 Area was moved approximately 0.7 miles to the south to meet the ODCM requirement that the station be between 9,000 and 12,000 meters from the center of containment in that same sector. The control station TLD (background) is located at Station 9A in Sunnyside. The remaining TLDs served as indicator TLDs throughout the year.

Two sets of TLDs placed three feet above ground were employed at each location. One set of TLDs was exchanged on a quarterly basis (Quarterly TLDs) and the other exchanged on an annual basis (Annual TLDs). Exposure received by the field TLDs during transport to the TLD sites was monitored by a set of trip control dosimeters that accompanied the field dosimeters to and from the field locations. Another set of TLDs were used as building controls which were used to determine the exposure of the TLDs at the controlled storage location. The TLD exposure during transport to and from the field was determined by subtracting the difference between the building control results and the trip control results.

The environmental dosimeters were processed on a Teledyne Isotopes Model 9100 Automatic Reader. Following the initial processing, the field dosimeters were annealed and given a calibration exposure of 100 mR of cesium-137 gamma, in order to determine response (i.e., calibration) factors for each dosimeter. The calibration factors were then used to determine the total exposure received by each TLD.

The exposure values determined for calibration exposures, as well as the exposures of the QA dosimeters (processing control dosimeters) and audit dosimeters (spiked dosimeters), were based on the calculated field strength of the encapsulated cesium-137 source. The calculated field strength of the source was determined from National Institute of Standards and Technology (NIST)-traceable ionization chamber measurements made over a period of several years during the routine use and calibration of the source. Ionization chamber measurements made during TLD calibration were used to confirm the calculated exposure. If the calculated exposure and the ionization chamber reading differed by 5% or more, an investigation was performed to resolve the difference.

Three Reuter Stokes pressurized ionization chambers (PICs) provided additional capability for measuring direct radiation exposure. These units are no longer part of the routine monitoring program, but they are used in special monitoring situations and maintained as back-up monitoring systems.

#### 4.3.2 Airborne Particulate/Iodine

Air particulate and air iodine (I-131) samples were obtained through the use of portable, low volume (1.5 cfm) constant flow-rate sampling units at each of twelve locations. The samples drawn at Station 9A (Figure 4-3) were considered control samples; the ones drawn at the other locations (Figure 4-1) were indicator samples. Air particulates were collected by drawing air through a 47mm diameter glass fiber filter. Air iodine was collected by drawing air through a 57mm diameter TEDA impregnated charcoal cartridge. The particulate air filter and charcoal cartridge were placed in tandem, particulate filter first, in a holder that attached to the air inlet of the sampler unit. The sampler units were placed in ventilated metal weatherproof housings mounted on elevated platforms at each air sample location. The filter media are changed weekly and shipped to the analytical contractor for analysis within one or two days of collection.

#### 4.3.3 Water

There were nine sampling locations for water sampling: three for the evaluation of river/drinking water, one for plant discharge water, three for groundwater, one for the storm drain water, and one for sanitary waste water. One river/drinking water location, Station 26, was used for evaluation of the plant intake water, i.e., the river water taken upstream of the plant discharge point. This Station 26 sample can be considered a drinking water sample since the site draws its drinking water from the intake water. It is considered the river/drinking water control sample because of its upstream location. Two additional locations, Stations 28 and 29, were used to evaluate the water at the two nearest drinking water locations, the Department of Energy 300 Area and the Richland Water Treatment Plant. These two stations were considered indicator stations.

The ODCM requirement for a downstream water sample "near but beyond the mixing zone" was met by sampling water from Station 27, the plant discharge line to the Columbia River. This sample reflects the radioactivity present in the plant discharge prior to any river dilution, rather than the concentrations that would be found after dilution in the mixing zone. Water is drawn at this location because it was not feasible to perform flow-proportional composite sampling in the mixing zone area of the river downstream from the plant discharge point. The Station 27 sample is also considered an indicator sample.

Composite samplers are installed at the Columbia River pumphouse to monitor the cooling tower discharge line (Station 27). There are composite samplers at two drinking water locations (Stations 28 and 29), and the control location (Station 26). Samples collected are 25-ml aliquots of water at regular intervals of time or flow.

Non-routine analyses on the drinking water samples include strontium-90 and iodine-131 analysis. Strontium-90 analysis is required when the gross beta activity exceeds either 8 pCi/liter or ten times the mean of the previous three months' activity for a specific location. Iodine-131 analysis is required when the dose calculated for the consumption of water exceeds one millirem per year. Neither of these analyses were required during 1994.

Only three wells within the vicinity of Plant 2 are capable of supplying water. These are a deep well on the Plant 2 site (0.1 mile north of the Reactor Building) and two wells on the WNP-1 site (1.2 miles downgradient from Plant 2). Water from the Plant 2 well can be used as a backup source for drinking and fire protection. Water from the WNP-1 wells supplies the drinking and fire protection water for the WNP-1 site. Although none of these wells draw from the unconfined aquifer, they are considered indicator samples. Quarterly grab samples were taken from each of these wells. One gallon (3785 ml) was collected from each well for gamma analysis and 250 ml was drawn for tritium analysis.

Water samples were also collected from the storm drain (Station 101) for Plant 2, and at the Sanitary Waste Treatment Facility (SWTF; Station 102). In January 1994, a flume was installed at the storm drain outfall allowing flow-proportional composite sampling to be done using the automatic sampler in place there. The SWTF samples are grab samples collected monthly, quarterly and before discharge.

#### 4.3.4 Soil

As required by the Site Certification Agreement, annual soil samples were taken at Stations 1, 7, 21 and 23. One sample was taken at the control location, Station 9A (Figure 4-3). Quarterly soil samples were collected at two special interest locations, Station 101 and Station 118, as shown in Figures 4-4.

Each sample was collected from an area of approximately one square foot to a depth of approximately one inch. Approximately two kilograms of soil were collected in each sample. Soil samples were shipped to the analytical contractor after collection and analyzed for gamma activity.

If the gamma isotopic analysis indicates that cesium levels in any of the indicator samples exceeds ten (10) times the level in the control sample, a strontium analysis is performed on the sample(s). No strontium analysis was required during 1994.

#### 4.3.5 Sediment

Sediment samples were collected twice during 1994 from two sampling locations along the Columbia River. The upstream sediment sample (Station 33) was collected from a location approximately two miles upriver from the plant discharge. The downstream sample (Station 34) was collected approximately one mile downstream of the plant discharge. Each sample consisted of approximately two kilograms of the shallow surface sediment scooped from below the waterline. The samples were shipped to the analytical contractor.

Sediment samples were also taken from the storm drain (Station 101) outfall and pond and the SWTF (Station 102) north stabilization pond. Sediment sampling in these locations was performed in a manner similar to river sediment sampling. Special care was taken to prevent loss of the fine particulates in the sediment. In addition, formalin was added to the sanitary pond sediment prior to shipping, to inhibit gas formation within the sample container.

#### 4.3.6 Fish

Fish sampling was performed in late September when the likelihood of obtaining anadromous species was high. Fish samples collected from the Columbia River (Station 30 in Figure 4-1) were indicator samples, whereas the fish collected on the Snake River (Stations 38 and 38A in Figure 4-2) were control samples.

Four separate fish samples, consisting of an anadromous species and three other species generally considered edible or potentially edible (such as carp, catfish and whitefish) were collected at each location. Most of the fish were collected using electro-shocking, but samples of the anadromous species were also collected from the Ringold hatchery on the Columbia River and at the Lyons Ferry Fish Hatchery on the Snake River. The fish were filleted to obtain one kilogram of edible flesh per sample. The fillets were placed in clean plastic bags and frozen until shipment to the analytical contractor. The Site Certification Agreement was changed to require only annual fish sampling, unless elevated radiation levels related to plant operations are observed in which case semi-annual sampling would resume.

#### 4.3.7 Milk

Milk samples were collected monthly during January, February, March, October, November and December and semi-monthly during the remaining six months when the cows were likely to be grazing. One gallon of raw milk was collected from each sampling location. The milk samples were chilled thoroughly and shipped to the analytical contractor within a day of collection.

Routine samples were collected from three indicator locations (Stations 36, 62 and 64) across the Columbia River in Franklin County. Milk samples for Station 62 were only taken until March at which time milk production ceased at this farm.

Milk samples were also collected at one indicator station (Station 9B) and one control location (Station 96) in the Sunnyside/Grandview area (in Figure 4-3). Station 9B in Sunnyside continued to serve as an indicator station in 1994 because a portion of the feed for the cows at that location is hay from Franklin County north of Pasco. That factor makes it unsuitable for use as a control location.

#### 4.3.8 Garden Produce

Samples of local garden produce were collected monthly from April to September when the produce was readily available. When possible, three types of produce samples, a root crop, fruit and a leafy vegetable, were collected at each location.

The indicator samples were collected from a region in a predominant downwind direction (Station 37 in Figure 4-2) where crops are irrigated with Columbia River water. The control samples were obtained from produce stands in the Sunnyside area (Station 9C in Figure 4-3), the direction least likely to be affected by plant effluents. Apples were collected in September from Station 91, the Rio Vista Farms orchard, which is irrigated with Columbia River water.

#### **4.3.9 Vegetation**

The annual sample of vegetation growing in the storm drain pond was collected in June. Cattails and grasses were the principal types of vegetation collected. Approximately two kilograms of sample were collected each time. Care was taken to avoid including the roots or soil from around the roots in the samples.

#### **4.4 Analytical Procedures**

The analytical procedures used for the 1994 REMP samples are described below. Teledyne Brown Engineering Environmental Services performed all routine analyses of REMP samples during 1994.

##### **4.4.1 Gross Beta Activity on Particulate Filters**

The particulate filters were counted in a gas flow-proportional counter after a delay of five or more days to allow for the radon-222 and radon-220 (thoron) daughter products to decay. An unused air particulate filter was counted as the blank with each weekly set of filters.

##### **4.4.2 Measurement of Gamma Emitters**

A shielded Ge(Li) detector system was coupled to a computer-based data acquisition system which performed pulse height and gamma energy analysis. The information collected about each peak was compared to a library of known peaks. Isotopic identification was performed as was the radioactivity calculation which used the appropriate fractional gamma ray abundance, half-life, detector efficiency, and net counts in the peak region.

##### **Milk and Water**

A 1-liter Marinelli beaker was filled with a representative aliquot of the sample. The sample was then counted for at least 1000 minutes (16.7 hours).

##### **Foodstuff**

As much of the edible portion of the sample as possible was loaded into a tared Marinelli beaker and weighed. The sample was then counted for at least 1000 minutes (16.7 hours).



### Vegetation

As much sample as possible is placed in a 1-liter Marinelli beaker and counted for approximately 1000 minutes (16.7 hours). The sample is not dried prior to counting, so the results are given in terms of wet weight.

### Soils and Sediments

A large quantity of the sample was dried at a temperature below 100°C. As much sample as possible was loaded into a tared 1-liter Marinelli beaker and weighed. The sample was then counted for at least 360 minutes (6 hours).

### Charcoal Cartridges (Air Iodine)

Charcoal filters were counted up to five at a time, with one positioned on the face and up to four on the side of the calibrated Ge(Li) detector. The detection limit for a charcoal cartridge was uniquely determined for each filter and by using its position. In the event that iodine-131 would have been observed in the initial counting of a set, each charcoal cartridge in the set was then positioned separately on the face of the detector and counted.

### Air Particulate Filters

Four air particulate filters for a quarterly composite from each field station were aligned one in front of another and counted for at least 360 minutes (6 hours).

## **4.4.3 Gross Beta Activity in Water**

A one-liter aliquot of each sample was evaporated to a small volume and transferred to a stainless steel planchet. The sample was dried under heat lamps, cooled, then counted on an automatic beta proportional counter. The results were calculated using empirical self-absorption curves which enabled the correction of effective counting efficiency based on the sample residue mass.

## **4.4.4 Iodine-131 in Water**

Two liters of sample were first equilibrated with a stable iodide carrier. A batch treatment with anion exchange resin was used to remove iodine from the sample. The iodine was then stripped from the resin with sodium hypochlorite solution, reduced with hydroxylamine hydrochloride, and extracted into carbon tetrachloride as free iodine. It was then back-extracted as iodide into sodium bisulfite solution and precipitated as palladium iodide. The precipitate was weighed for chemical yield and mounted on a nylon planchet for low-level beta counting. The chemical yield was corrected by measuring the stable iodide content of the water with a specific ion electrode. During 1994, this procedure was used only on intercomparison samples, since the doses calculated via ODCM methodology for the consumption of drinking water did not exceed one millirem per year.

#### 4.4.5 Tritium in Water

The analysis of tritium in water is performed utilizing liquid scintillation. Liquid scintillation requires 10 milliliters of water mixed with 10 milliliters of liquid scintillation "cocktail." The mixture is then counted in an automatic liquid scintillation detector.

#### 4.4.6 Strontium-89 and 90 in Water, Milk and Soil

During 1994, strontium analyses were not required for any routine REMP water, milk or soil samples. It was used for intercomparison water and sediment analyses. The techniques used to analyze for strontium in the various media are described below.

##### Water

Stable strontium carrier was added to one liter of sample and the volume is reduced by evaporation. Strontium is precipitated as  $\text{Sr}(\text{NO}_3)_2$  using fuming (90%) nitric acid.

##### Milk

Stable strontium carrier is added to one liter of sample. The sample is then evaporated and ashed in a muffle furnace. The ash is dissolved and strontium precipitated as a phosphate. The sample is then redissolved and strontium precipitated as  $\text{Sr}(\text{NO}_3)_2$  using fuming (90%) nitric acid.

##### Soil and Sediment

The sample is first dried under heat lamps and a 10-gram aliquot is taken. Stable strontium carrier is added and the sample is leached in hydrochloric acid. After the mixture is filtered, phosphates are then precipitated, collected by filtration, and dissolved in nitric acid. Strontium is precipitated as  $\text{Sr}(\text{NO}_3)_2$  using fuming (90%) nitric acid. A barium chromate scavenge and an iron (ferric hydroxide) scavenge are then performed. Stable yttrium carrier is added and the sample is allowed to stand for five days or more for yttrium ingrowth. Yttrium is then precipitated as hydroxide, dissolved and reprecipitated as oxalate. The yttrium oxalate is mounted on a nylon planchet and counted in a low-level beta counter to infer strontium-90 activity. Strontium-89 activity is determined by precipitating  $\text{SrCO}_3$  from the sample after yttrium separation. This precipitate is mounted on a nylon planchet and covered with an 80 mg/cm<sup>2</sup> aluminum absorber for low-level beta counting.

#### 4.4.7 Iodine-131 in Milk

Two liters of sample are first equilibrated with stable iodide carrier. A batch treatment with anion exchange resin is used to remove iodine from the sample. The iodine is then stripped from the resin with sodium hypochlorite solution, reduced with hydroxylamine hydrochloride, and extracted into carbon tetrachloride as free iodine. It was then back-extracted as iodide into sodium bisulfite solution and precipitated as palladium iodide. The precipitate was weighed for chemical yield and mounted on a nylon planchet for low-level beta counting. The chemical yield is corrected by measuring the stable iodide content of the milk with a specific ion electrode.

## 4.5 Data Analysis Methods

Since mid-1984, the results of the REMP analyses have been presented as net results calculated from the gross or total counts determined for each radionuclide minus the background counts of the counting or detection instrument. Consequently, for several sample types, the results range from negative to positive numbers. This manner of presenting environmental data prevents the bias and loss of individual results inherent in the use of "less than" ( $<$ ) values, where the "less than" numbers can have a variety of meanings, such as "less than the lower limit of detection (LLD)." The net results for REMP samples are presented with an asterisk (\*) in front, if the results are less than the LLD. A listing of the LLDs determined for each analysis is provided in Table 4-4 as a reference when reviewing the sample results.

Plots of the sample results versus time are used to represent the results for analyses such as gross beta on air particulate filters, where the results are normally above the lower limits of detection. In such cases, the indicator station results are plotted with the control station results for easy comparison. Other data analysis techniques, such as frequency distributions, are also used to represent the data and to determine whether trends that could be attributed to Plant 2 operations are evident. Thermoluminescent dosimeter (TLD) data is presented in terms of the net mR/day exposure rate. These results are determined from the total exposure (in mR) calculated for each TLD from its total thermoluminescent output minus the TLD background, minus any transit (or trip) exposure received during distribution and retrieval, and divided by the number of days the TLD was in the field. Circular plots, frequency distributions and graphs of TLD data by meteorological sector and distance from the plant are used to interpret trends in the results.

TLD data summaries include the term "standard error" along with "standard deviation." The standard error, which is the estimate of the precision of the mean, is used for the means of quarterly and annual data. The standard deviation is used for the TLD data involving a single dosimeter result that has been determined from the four reader areas of the TLD. Both terms provide an indicator of the uncertainty associated with the results.

TABLE 4-1  
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM PLAN

SAMPLE TYPE <sup>(a)</sup>	SAMPLE STATION <sup>(b)</sup> NUMBER	SAMPLING AND COLLECTION FREQUENCY <sup>(c)</sup>	TYPE AND FREQUENCY OF ANALYSIS
<b>1. AIRBORNE</b>			
Particulates and radioiodine (6/12) <sup>(d)</sup>	1, 4-9A, 21, 23, 40, 48, and 57	Continuous sampling; weekly collection	Particulate: Weekly gross beta <sup>(e)</sup> ; gamma isotopic <sup>(f)</sup> of quarterly composite (by location)  Iodine: Weekly gamma analysis.
Soil <sup>(g)</sup> (0/7)	9A, 1,7,21 and 23, 101, 118	Annually Quarterly or more often as needed.	Gamma isotopic <sup>(f)</sup> ; strontium-90 <sup>(h)</sup> Gamma isotopic
<b>2. DIRECT RADIATION</b>			
TLD <sup>(i)</sup> (34/57)	1-9A, 10-25, 40-47, 49-51, 53-56, 71-86 (1S-16S) <sup>(j)</sup>	Quarterly, annually	Thermoluminescent output; quarterly and annual processing.
PIC	Various locations, as needed <sup>(k)</sup>	Continuous recording, as needed	Exposure rate accumulated on mag card and in internal memory
<b>3. WATERBORNE</b>			
River/Drinking Water <sup>(l)</sup> (3/4)	26, 27, 28 and 29	Composite aliquots <sup>(m)</sup> ; monthly collection	Gamma isotopic <sup>(f)</sup> , gross beta, quarterly; tritium composite; strontium-90 <sup>(a)</sup> ; I-131 <sup>(o)</sup>
Storm Drain Water (1/1)	101	Composite aliquots <sup>(m)</sup> , weekly collection; grab samples	Gamma isotopic <sup>(f)</sup> , tritium, gross beta
Sanitary Waste Treatment Facility Water (1/1)	102	Monthly, annually, pre-discharge and as needed.	Gamma isotopic <sup>(f)</sup> , gross beta, gross alpha, strontium-90 <sup>(a)</sup> , tritium
Ground Water (2/3) <sup>(p)</sup>	31, 32, and 52	Quarterly	Gamma isotopic <sup>(f)</sup> ; tritium
River Sediment (1/2) <sup>(q)</sup>	33 and 34	Semiannually	Gamma isotopic <sup>(f)</sup>
Sanitary Waste Treatment Facility Sediment (1/1)	102	Monthly or more often as needed	Gamma Isotopic <sup>(f)</sup>

TABLE 4-1 (Cont.)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM PLAN

SAMPLE TYPE <sup>(a)</sup>	SAMPLE STATION <sup>(b)</sup> NUMBER	SAMPLING AND COLLECTION FREQUENCY <sup>(c)</sup>	TYPE AND FREQUENCY OF ANALYSIS
4. INGESTION			
Milk <sup>(a)</sup> (4/4)	9B, 36, 62 <sup>(a)</sup> , 64 and 96 <sup>(a)</sup>	Semi-monthly during grazing season, monthly at other times	Gamma isotopic <sup>(d)</sup> ; iodine-131; strontium-90 <sup>(d)</sup>
Fish <sup>(a)</sup> (2/2)	30, 38 and 39	Annually <sup>(e)</sup>	Gamma isotopic <sup>(d)</sup>
Garden Produce <sup>(a)</sup> (1/3)	9C, 91 <sup>(a)</sup> and 37	Monthly during growing season in the Riverview area of Pasco and a control near Grandview; annual collection at Station 91.	Gamma isotopic <sup>(d)</sup>
Vegetation	101	annually	Gamma isotopic <sup>(d)</sup>

- (a) The fraction in parentheses for each sample type indicates the ratio of ODCM-required sample locations to the total number of sample locations currently being monitored in the surveillance program. The SCA also requires certain numbers of sampling stations for each type of media.
- (b) The underlined sample location designates a control station.
- (c) Deviations are permitted if samples are unobtainable due to hazardous conditions, seasonal availability, malfunction of automatic sampling equipment, or other legitimate reasons. Such deviations are documented in Section 5
- (d) The SCA requires nine or more air sampling stations.
- (e) Particulate sample filters will be analyzed for gross beta after at least 24 to 48 hours to allow for the decay of radon daughter products. If gross beta activity is greater than 10 times the mean of the result for the control, Station 9A, gamma isotopic analysis shall be performed on the individual sample.
- (f) Gamma isotopic means identification and quantification of gamma-emitting radionuclides that may be attributable to the effluents of Plant 2.
- (g) Soil samples are collected to satisfy the requirements of the Site Certification Agreement (SCA)<sup>6</sup> for Plant 2. The SCA requires that soil samples be collected at five air sampling locations.

TABLE 4-1 (Cont.)

- (h) Strontium-90 analysis shall be performed on any indicator soil sample having cesium results greater than ten times the results for the control location.
- (i) TLD refers to thermoluminescent dosimeter. For purposes of the REMP, a TLD is a phosphor card (31.75mm x 44.75mm x 0.4mm) with eight individual read-out areas (four main dosimeter areas and four back-up dosimeter areas) in each badge case. TLDs used in the REMP meet the requirements of Reg Guide 4.13<sup>5</sup> and ANSI N545-1975, except for specified energy-dependence response. Correlation factors are available for energy ranges with response outside of specified tolerances.
- (j) TLD Stations 71-86 are special interest stations and are not included among the 34 routine TLD stations required by the ODCM Table 6.3.1.1-1 (3.12-1). Their alternate designations are 1S-16S. The SCA requires that 25 or more TLD stations are located within a 10-mile radius of the plant.
- (k) Pressurized ion chambers (PICs) are not required as part of the routine monitoring program, but they are required by the SCA to be maintained as a supplemental or backup system. PICs were used routinely at various locations during 1994 to provide supplemental information.
- (l) The term "river/drinking water", instead of "surface/drinking water", is used throughout this report because the surface water is taken from the Columbia River. Station 26, Plant 2 makeup water intake from the Columbia River is both an upstream surface, or river, water sample and the drinking water control sample location. Station 28 (300 Area) and Station 29 samples are drinking water samples. The Station 27 sample, which is drawn from the plant discharge line, is taken in place of a "downstream" water sample near but beyond the mixing zone. It reflects the radioactivity present in the plant discharge prior to any river dilution. The SCA requires two drinking water locations downstream from the plant discharge and requires sampling from the plant intake and discharge water. Station 101, the storm drain pond, and Station 102, the Sanitary Waste Treatment Facility, are represented individually because they are unique sampling locations requiring special attention.
- (m) Composite (integrated grab) samples are collected with equipment which collects an aliquot at time intervals that are short relative to the compositing period.
- (n) When the gross beta activity in drinking water exceeds 8 pCi/liter, a strontium-90 analysis is performed.
- (o) When the dose calculated via ODCM methodology for consumption of water exceeds 1 mrem per year, iodine-131 analyses are performed on the drinking water samples.
- (p) The SCA requires sampling from wells used for fire protection and as backup drinking water sources.
- (q) The SCA requires sediment sample collection upstream and downstream of the plant discharge.
- (r) Milk samples will be obtained from farms or individual milk animals which are located in the most prevalent wind directions from Plant 2. Routine milk samples are collected in areas of high dose potential instead of within 5 kilometers, due to the locations of milk animals. The SCA requires at least three milk locations within the 10-mile radius of the plant and one in a control location.
- (s) Station 96 is the control station for milk samples because it was determined that the cows at Station 9B in Sunnyside were given feed grown in the Franklin County area across the Columbia River from Plant 2. Station 62 was removed as a milk location in March 1994.

TABLE 4-1 (Cont.)

- (t) If cesium-134 or cesium-137 is measured in an individual milk sample in excess of 30 pCi/l, then the strontium-90 analysis will be performed..
- (u) There are no commercially important species in the Hanford Reach of the Columbia River. Most recreationally important species in the area are anadromous (primarily salmonids), which ascend rivers from the sea for breeding. Four fish species will normally be collected by the electroshock technique in the vicinity of the plant discharge (Station 30) and from the Snake River (Station 38). If electroshocking produces insufficient anadromous fish samples from the Snake River, samples may be obtained from the fish trap at Ice Harbor Dam, Lyons Ferry Fish Hatchery, or other similar facility (Station 38A). If insufficient anadromous fish samples are produced through electroshocking on the Columbia River, samples may be obtained at the Ringold Fish Hatchery (Station 39).
- (v) If an impact is indicated, sampling will be conducted semiannually.
- (w) Garden produce will routinely be obtained from farms or gardens using Columbia River water for irrigation. One sample of a root crop, leafy vegetable, and a fruit is collected each sample period, if available. The variety of the produce obtained will be dependent on seasonal availability.
- (x) Station 91 is an apple orchard irrigated with Columbia River water. The apple crop from Station 91 is sampled annually.

TABLE 4-2

REMP SAMPLE LOCATIONS BY SECTOR

SECTOR <sup>(a)</sup>	STATION <sup>(b)</sup> NUMBER	ESTIMATED DISTANCE <sup>(c)</sup>		SAMPLE TYPE <sup>(d)</sup>
		MILES	METERS	
N (1)	52	0.1	161	GW
	71(1S)	0.3	483	TLD
	47	0.5	805	TLD
	57	0.8	1201	AP/AI
	18	1.1	1770	TLD
	53	7.5	12068	TLD
NNE (2)	72(2S) <sup>(e)</sup>	0.4	644	TLD
	2	1.8	2896	TLD
	54	6.5	10459	TLD
NE (3)	73(3S)	0.5	805	TLD
	19	1.8	2896	TLD
	48	4.5	7241	AP/AI
	39	4.4	7084	FI
	46	5.0	8045	TLD
ENE (4)	101	0.3	483	SDW/SE/SO/VE
	74(4s)	0.4	644	TLD
	21	1.5	2414	AP/AI/SO/TLD
	20	1.9	3057	TLD
	11	3.1	4988	TLD
	33	3.6	5792	SE
	45	4.3	6919	TLD
	44	5.8	9332	TLD
E (5)	75(5S)	0.4	644	TLD
	22	2.1	3379	TLD
	10	3.1	4988	TLD
	26	3.2	5149	PW
	27	3.2	5149	DW
	30	3.3	5311	FI
	43	5.8	9332	TLD
ESE (6)	76(6S)	0.4	644	TLD
	31	1.1	1770	GW
	32	1.2	1931	GW
	51	2.1	3379	TLD
	23	3.0	4827	AP/AI/TLD
	34	3.5	5632	SE
	91	4.4	7079	FR
	8	4.5	7241	AP/AI/TLD
	42	5.6	9010	TLD
	36 <sup>(e)</sup>	7.2	11585	MI
	5	7.7	12389	AP/AI/TLD
	64	9.7	15610	MI
	38	26.5	42639	FI



TABLE 4-2 (Cont.)

REMP SAMPLE LOCATIONS BY SECTOR

SECTOR <sup>(a)</sup>	STATION <sup>(b)</sup> NUMBER	ESTIMATED DISTANCE <sup>(c)</sup>		SAMPLE TYPE <sup>(d)</sup>
		MILES	METERS	
SE (7)	102	0.3	483	SSW/SE
	77(7S)	0.5	805	TLD
	24	1.9	3057	TLD
	3	2.0	3218	TLD
	41	5.8	9332	TLD
	40	6.4	10298	AP/AI/TLD
	62	10.9	16730	MI
SSE (8)	78(8S)	0.7	1126	TLD
	25	1.6	2574	TLD
	55 <sup>(e)</sup>	5.5/6.2	8850/9976	TLD
	28	7.4	11907	PW
	4	9.3	14964	AI/AP/TLD
	29	11.0	17699	PW
	37 <sup>(h)</sup>	16.0	25744	GP
S (9)	118	0.3	483	SO
	79(9S)	0.7	1126	TLD
	1	1.3	2092	AP/AI/SO/TLD
	6	7.7	12389	AP/AI/TLD
SSW (10)	80(10S)	0.8	1287	TLD
	50	1.2	1931	TLD
	56	7.0	11263	TLD
SW (11)	81(11S)	0.7	1126	TLD
	13	1.4	2253	TLD
	96	36.0	49250	MI
WSW (12)	82(12S)	0.5	805	TLD
	14	1.4	2253	TLD
	9A, 9B, 9C <sup>(i)</sup>	30.0	48270	AP/AI/MI/GP/TLD/SO
W (13)	83(13S)	0.5	805	TLD
	15	1.4	2253	TLD
WNW (14)	84(14S)	0.5	805	TLD
	16	1.4	2253	TLD
	7	2.7	4344	AP/AI/SO/TLD
NW (15)	85(15S)	0.5	805	TLD
	49	1.2	1931	TLD
NNW (16)	86(16S)	0.4	644	TLD
	17	1.2	1931	TLD
	12	3.1	9815	TLD

TABLE 4-2 (Cont.)

- (a) The area in the vicinity of Plant 2 is separated into 16 separate sectors for reporting purposes. The 16 sectors cover 360 degrees in equal 22.5 degree sections, beginning with Sector 1 (N) at 348.75 to 11.25 degrees and continuing clockwise through Sector 16 (NNW).
- (b) The alternate designations for TLD Stations 71-86 are given in parentheses, i.e., 1S-16S.
- (c) Distances are estimated from map positions for each location as a radial distance from Plant 2 containment.
- (d) Sample Type Key:

AI	- Air Iodine	AP	- Air Particulate
DW	- Discharge Water	FI	- Fish
FR	- Fruit	GP	- Garden Produce
GW	- Ground Water	MI	- Milk
PW	- Surface (River)/Drinking Water	SDW	- Storm Drain Water
SE	- Sediment	SO	- Soil
SWW	- Sanitary Waste Water	TLD	- Thermoluminescent Dosimeter
VE	- Vegetation		

Station 9 designates the Sunnyside-Grandview control area. It is actually three separate stations (Stations 9A for TLD, AI/AP and SO, 9B for milk, and 9C for GP) within a few miles of each other and all within 30-35 miles of Plant 2. Station 96, which is the control station for milk, is also located within the control area. It is 36 miles from Plant 2. Station 9B, which was the control location for milk until 1986, is now an indicator milk location.

- (e) TLD station 55 was moved from its former location to its present location to meet the ODCM requirement that the station to be between 9,000 meters and 12,000 meters from center of containment.
- (f) Duplicate samples, i.e., samples drawn at the same time as the routine samples and submitted for analysis as a quality assurance check, are collected at this location. The station designation for the duplicate of Station 27 is Station 72 for second and fourth quarters and 92 for first and third quarters. The station designation for the duplicate of Station 36 is Station 37.

TABLE 4-3

1994 FIVE MILE LAND USE CENSUS RESULTS

SECTOR <sup>(a)</sup>	NEAREST RESIDENT <sup>(b)</sup>	GARDEN (> 50M <sup>2</sup> )	DAIRY <sup>(c)</sup> ANIMALS	LIVESTOCK
NE	4.3	none	none	4.8
ENE	4.1	4.1 <sup>(d)</sup>	none	none
E	4.5	none	none	none
ESE	4.1	none	none	none
SE	none	none	none	none

(a) Eleven of the sixteen meteorological sectors within the five-mile radius of Plant 2 are on the federally-owned Hanford Site; the remaining land is comprised of 4.5 sq. miles of privately-owned farm land. Only those sectors containing points of interest are presented here.

(b) In miles. These are estimated distances.

(c) The closest dairy animal locations are at 8.3 and 10.9 miles SE and 7.2 and 9.7 miles ESE. The dairy at 8.3 miles SE is not used for milk sample collection due to the owner's reluctance to participate in the sampling program.

(d) Small garden with broadleaf; samples were not available due to the small amounts grown.

TABLE 4-4

**COMPARISON OF TELEDYNE NOMINAL LOWER LIMITS OF DETECTION  
WITH BRANCH TECHNICAL POSITION<sup>(3)</sup> REQUIREMENTS**

MEDIA (UNITS)	ANALYSIS	TELEDYNE LLDs	BTP REQUIRED LLDs
Air (pCi/m <sup>3</sup> )	Gross Beta	0.003	0.01
	Gamma Spectrometry		
	Cs-134	0.001	0.05
	Cs-137	0.001	0.06
	I-131	0.01	0.07
Water: (pCi/l)	Gross Beta	4	4
	Tritium	100-200	2000
	I-131	1	1
	Sr-90	1	—
	Gamma Spectrometry		
	Mn-54	10	15
	Fe-59	20	30
	Co-58	10	15
	Co-60	10	15
	Zn-65	20	30
	Zr-95	20	30
	Nb-95	10	15
	Cs-134	10	15
	Cs-137	10	18
	Ba-140	20	60
	La-140	10	15
Soil/Sediment: (pCi/kg dry)	Gamma Spectrometry		
	Co-57	120	—
	Co-60	30	—
	Zn-65	100	—
	Cs-134	30	150
	Cs-137	40	180
	Sr-90	10	—
Fish: (pCi/kg wet)	Gamma Spectrometry		
	Mn-54	20	130
	Fe-59	30	260
	Co-58	20	130
	Co-60	20	130
	Zn-65	30	260
	Cs-134	20	130
	Cs-137	20	150
Milk: (pCi/l)	I-131	0.5	1
	Gamma Spectrometry		
	Cs-134	10	15
	Cs-137	10	18
	Ba-140	20	60
	La-140	10	15
	Sr-90	1	—
Garden Produce: (pCi/kg wet)	Gamma Spectrometry		
	Cs-134	20	60
	Cs-137	20	80
	I-131	30	60



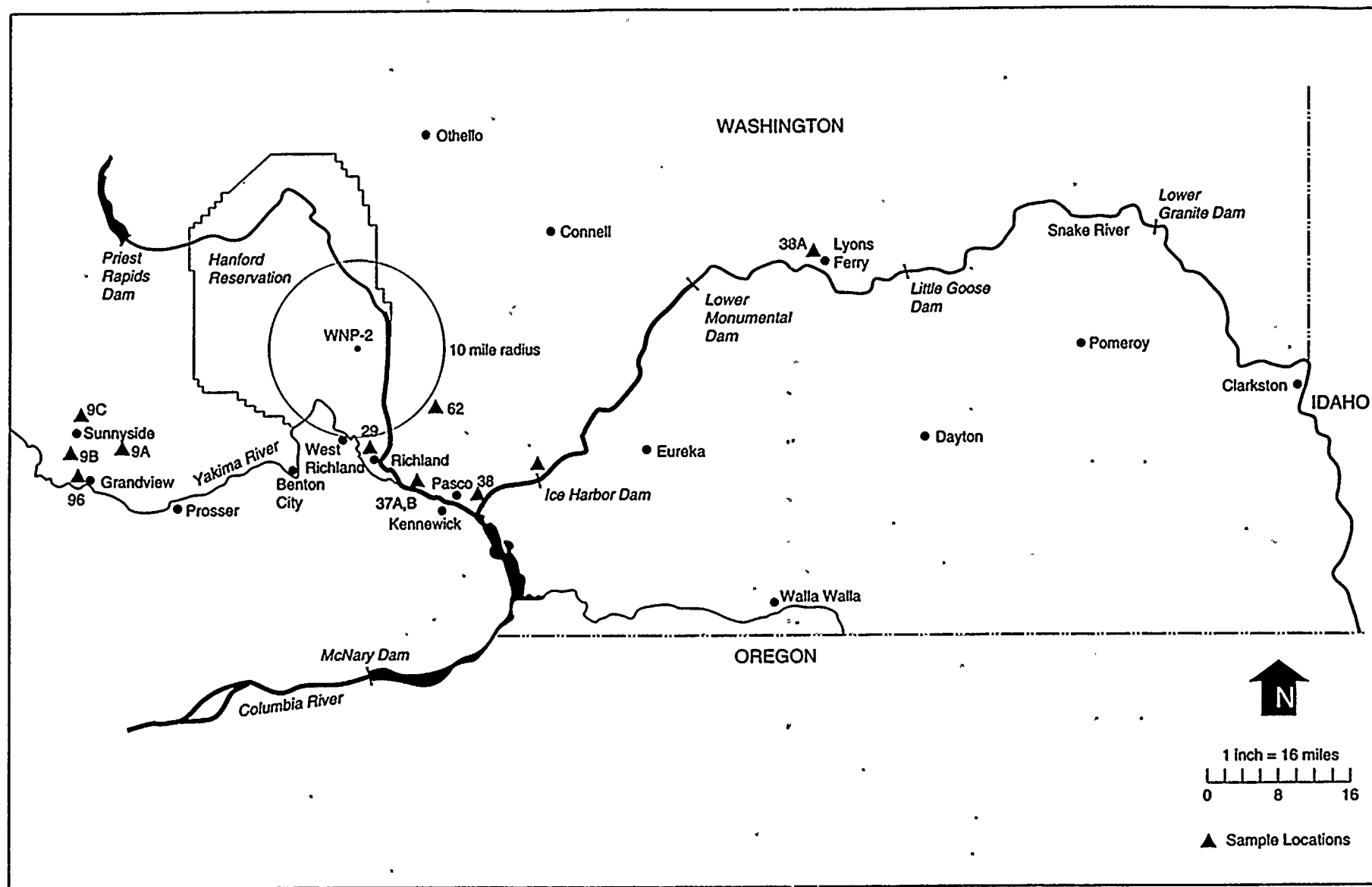


FIGURE 4-2 REMP SAMPLING LOCATIONS OUTSIDE THE 10-MILE RADIUS

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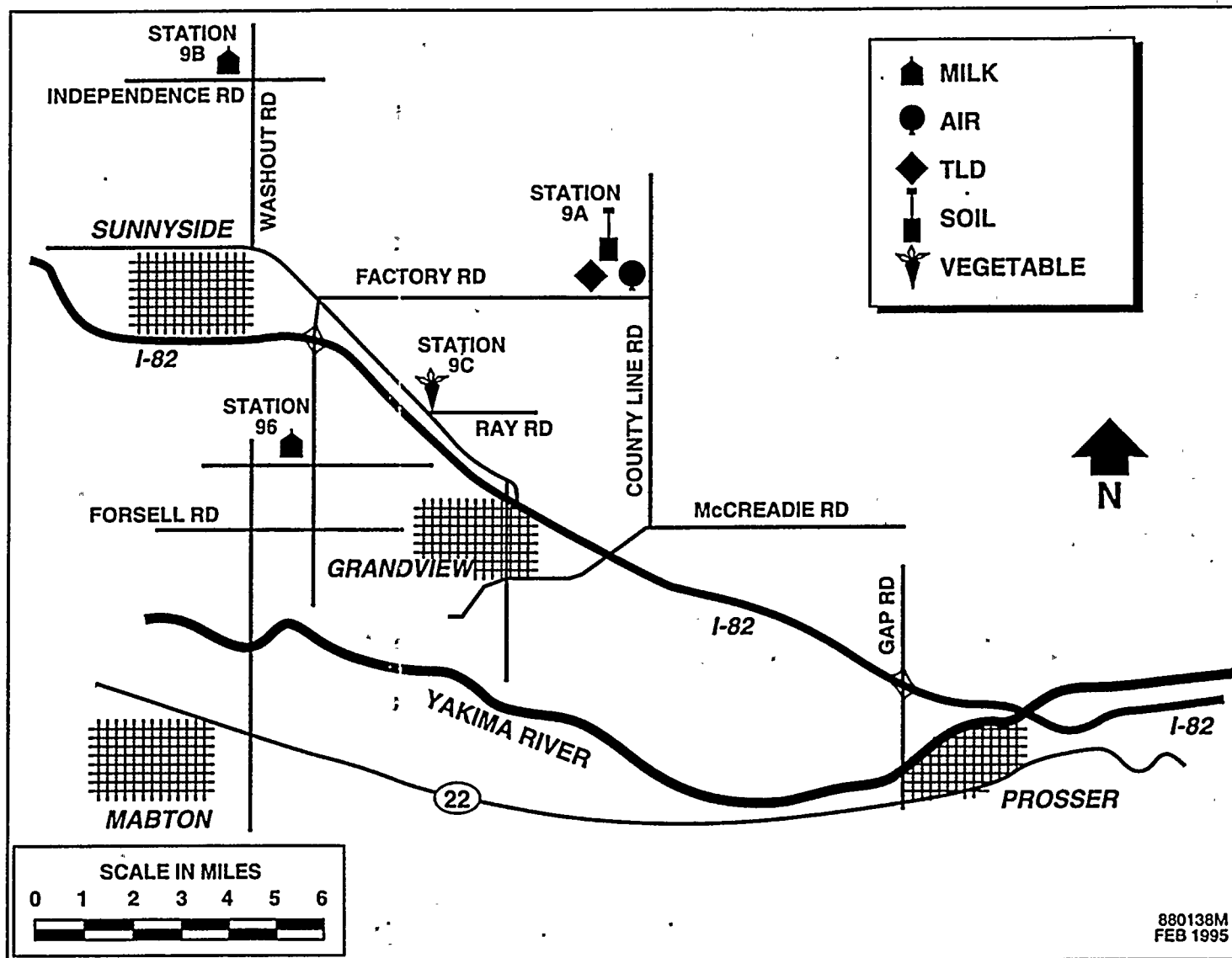


FIGURE 4-3 REMP SAMPLING LOCATIONS SUNNYSIDE/GRANDVIEW AREA

**FIGURE 4-4 REMP NEAR PLANT SAMPLING LOCATIONS**



## 5.0 RESULTS AND DISCUSSION

During 1994, the analyses of REMP samples were performed by Teledyne Brown Engineering Environmental Services in Westwood, New Jersey. The thermoluminescent dosimeters were processed by the Supply System External Dosimetry Laboratory. Table 5-1 presents the means and ranges of selected 1994 results for each type of sample collected and Table 5-3 provides a summary of detectable results. The means and ranges of the preoperational and the previous operational data from 1984 to 1993 are also included in the table for comparison. The data tables of 1994 results comprise a separate volume that is available to interested parties.

The data for the preoperational period and the first six months of 1984 included "less than" (<) designations for results below the actual LLD, the contractual LLD, or the two-sigma error, depending upon the convention employed by the analytical contractor. Consequently, the data averages using "less than" values are biased high. The use of the "less than" values was discontinued in mid-1984. Since then, REMP data have been reported as net (total results minus the detector counting background).

Since the primary focus of the REMP is to determine whether Plant 2 operations had an impact on the environment, the 1994 results are compared in this report to the results during the preoperational period and the results obtained during the previous years of Plant 2 operation. They are also compared to state and federal regulatory limits. Because of the use of "less than" values, rather than net results, during the preoperational period and during the first year of operation, and because of the impact of the 1986 Chernobyl accident on environmental radiation levels, the interpretation of the 1994 measurements relative to previous measurements must bear this in mind. Some of the parameters considered in the evaluations discussed in this report are the means, ranges and standard deviations or standard errors of the results. Comparative plots and frequency distributions of the data are some of the tools that have been employed in the interpretation of the 1994 REMP data.

The 1994 analytical results for the REMP sampling locations established since the preoperational period are very similar to the results reported for previous years. The 1994 annual and quarterly TLD results were also very much like those observed previously. No significant trends indicating an environmental impact or unexpected change in the environmental concentrations or exposure rates at REMP monitoring stations were observed.

## 5.1 Direct Radiation

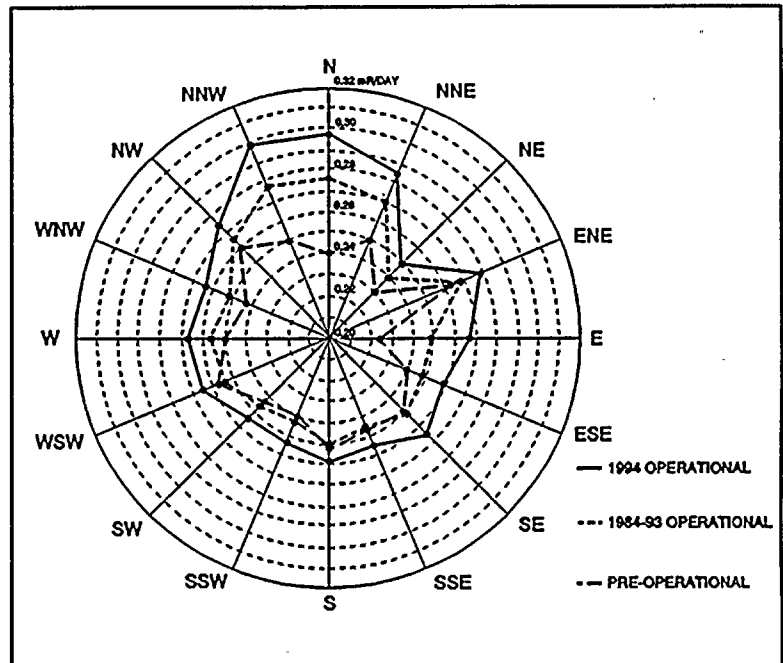
Summaries of the environmental radiation exposure rates, determined by thermoluminescent dosimeters (TLDs) are presented in Tables 5-4 and 5-5, and 5-6.

Figure 5-1 presents a circular plot of the mean 1994 quarterly TLD results for each of the sixteen meteorological sectors at the site boundary ("S" stations). The chart also includes preoperational and previous operational periods.

The relationship of the mean 1994 results to the results for the preoperational and previous operational periods is very similar for each sector. This indicates that there were no significant directional effects observed in the 1994 TLD results.

The increase in the mean 1994 TLD results over the means of the preoperational and previous operational periods is due to the effect of averaging only the four quarters of 1994 data, versus averaging numerous quarters for the other two sets of results.

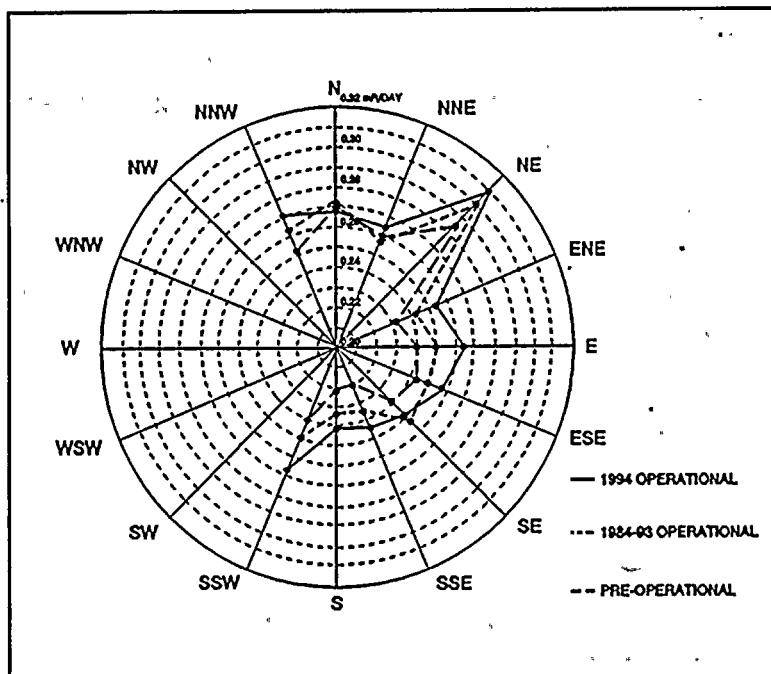
The shift to the north is a result of those sector TLDs being physically closer to the plant than the rest of the near-plant TLDs. Compare the data presented in Figure 5-1 with that of Figure 5-2, where the TLDs are more evenly placed from the plant.



For the remote TLDs, Station 46 in the Wahluke Reserve (NE sector) remained the location with the highest mean exposure rate, as shown in Figure 5-3. This is consistent with the mean exposure rates determined for that station in prior years.

Since the preoperational measurement phase, the results for this location have exceeded the results for all other locations. Variations in the ambient background due to differences in the soil and underlying rock composition account for such local differences in Station 46 TLD results.

Frequency distribution plots of the 1994 quarterly TLD results are presented in Figure 5-4. The plots are very similar from quarter to quarter, with 0.26 mR/day being the most frequent result and with the Station 46 results being upscale and separate from the main distribution. The frequency distributions for the previous operational TLD results are shown in Figure 5-5. The distributions agree well.



## 5.2 Airborne Particulate/Iodine

The 1994 mean weekly gross beta on particulate filter results for selected indicator stations near Plant 2 are plotted in Figure 5-6. The gross beta in air results for 1994 were within the ranges observed during the preoperational period and during previous operational periods, as shown in Table 5-1. In Figure 5-7, the similarity between results from near-plant locations and those from remote locations can be seen. The control location (Station 9A) results follow a very similar pattern to the remote and near-plant indicator locations. As observed previously, gross beta levels increased during periods of inversion occurring in the fall and winter months. In fact, the gross beta results plotted over a period of several years show a cyclic pattern of fall and winter increases. The increase, which was evident in the results of all the air sampling locations, is due to an increase in radon and radon daughter concentrations during the inversions.

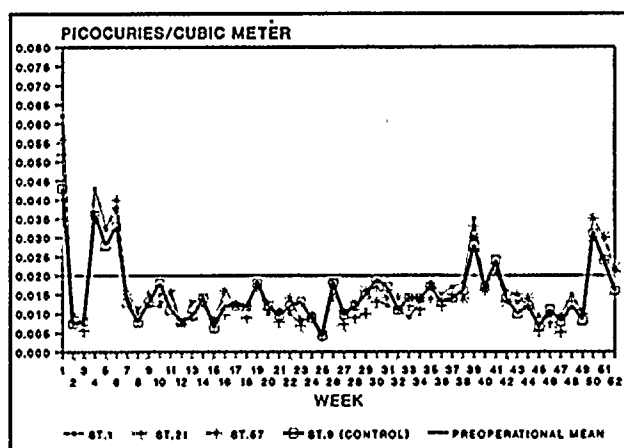


Figure 5-6 Gross Beta in Air, Near-Plant Stations - 1994

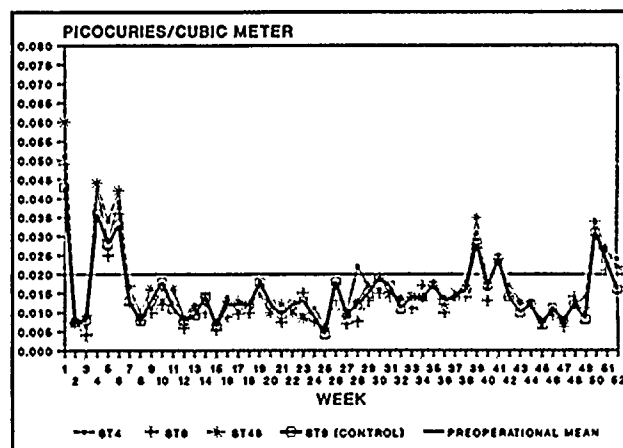


Figure 5-7 Gross Beta in Air, Remote Stations - 1994

The quarterly gamma analyses of the particulate filter composites indicated only the presence of two naturally-occurring radionuclides, beryllium-7 and potassium-40, at levels above detection limits at indicator locations and the control location. All iodine-131 in air results for 1994 were less than 0.02 picocuries/cubic meter (pCi/m³) LLD.

No evidence of any impact of plant operations on the environment was apparent in the particulate filter and charcoal cartridge results for 1994.

### 5.3 Water

All river/drinking water results for gross beta were within the ranges normally observed and less than 8 picocuries/liter (pCi/l), the level at which a strontium analysis is performed to verify compliance with the Washington State drinking water standard for strontium-90\*. The 1994 gross beta concentrations in river/drinking water, relative to the state annual average concentration limit<sup>(1)</sup>, are presented in Figure 5-8. The mean gross beta results in discharge water, for 1994 are presented in Figure 5-9. The 1994 average results compare well to the averages from previous periods.

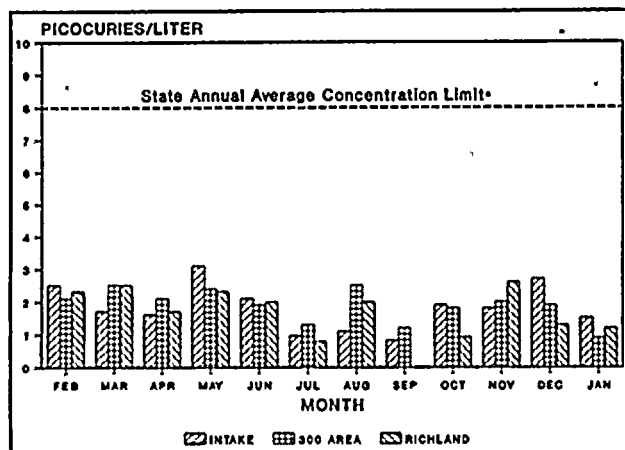


Figure 5-8 Gross Beta in River/Drinking Water-1994

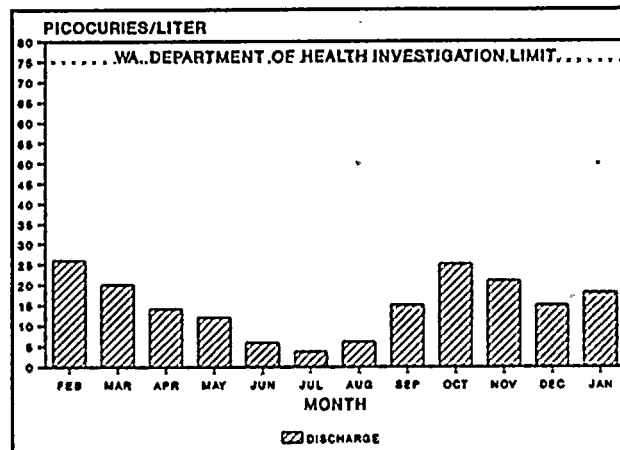


Figure 5-9 Gross Beta in Cooling Tower Discharge Water-1994

The gross beta levels in the discharge sample reflect the concentrations of naturally-occurring radionuclides, principally potassium-40, and any radionuclides from upstream sources of past Hanford activities present in the makeup water, in addition to radionuclides from Plant 2 discharges. The discharge sample results are representative of the radioactivity present in plant discharges before any mixing with river water occurs.

The 1994 tritium levels in the river/drinking water and groundwater were comparable with results obtained for prior years. Tritium levels in the discharge water were higher than the levels observed for the river/drinking water samples because of plant releases and because discharge water samples are taken prior to the water reaching the river and becoming diluted. As shown in Figure 5-10, the concentration of tritium was lower than the mean levels observed for the 1992 and 1993 periods. This reduction is due to an overall reduction in the volume of the radwaste discharges and the finding that a corrosion inhibitor used in Plant 2 had been introducing small amounts of boron into the primary system, which helped to increase tritium levels.

Tritium concentrations in the discharge water for 1994 ranged from 500 to 3500 pCi/l, which is low when compared to the NRC reporting level of 20,000 pCi/l for a quarterly average concentration in drinking water.

\*Strontium-90 is assumed to account for the gross beta result.

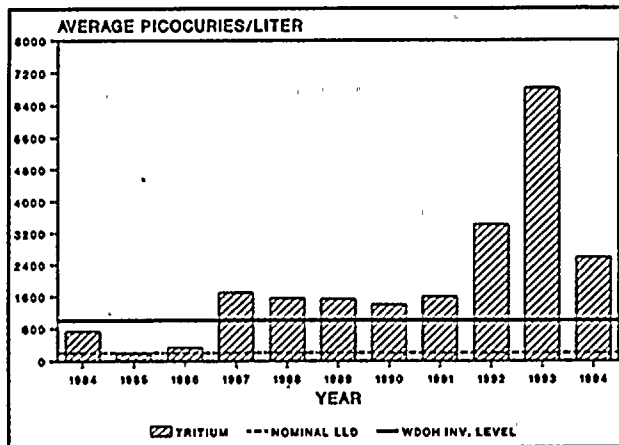


Figure 5-10 Tritium in Discharge Water 1984 - 1994

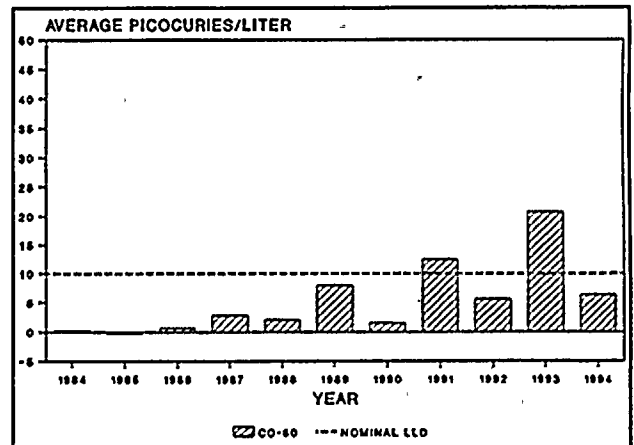


Figure 5-11 Cobalt-60 in Discharge Water 1984 - 1994

For gamma in discharge water, only cobalt-60, cesium-137 and potassium-40, a natural occurring radionuclide, were detected in samples during 1994. Figures 5-11, 5-12, and 5-13 show the annual averages for cobalt-60, zinc-65 and cesium-137 from 1984 to 1994. The detectable cobalt-60 results ranged from 4.6 to 17.5 pCi/l, the detectable cesium-137 result was 5.9 pCi/l, and the potassium-40 result was 46.8 pCi/l. Although above detection levels, these results were low and did not exceed any reporting levels.

There were no detectable nuclides in either the river/drinking water or the groundwater in samples taken during 1994.

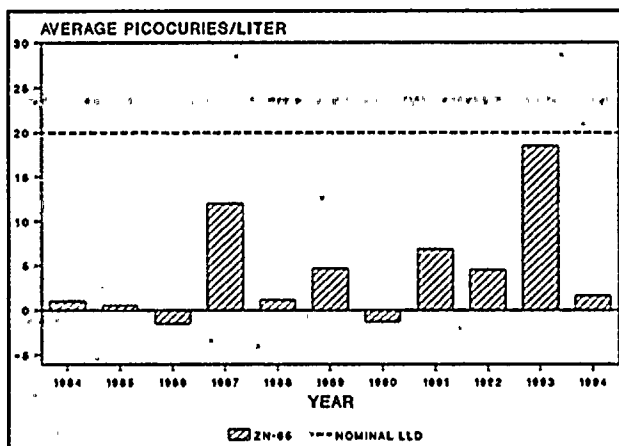


Figure 5-12 Zinc-65 in Discharge Water 1984 - 1994

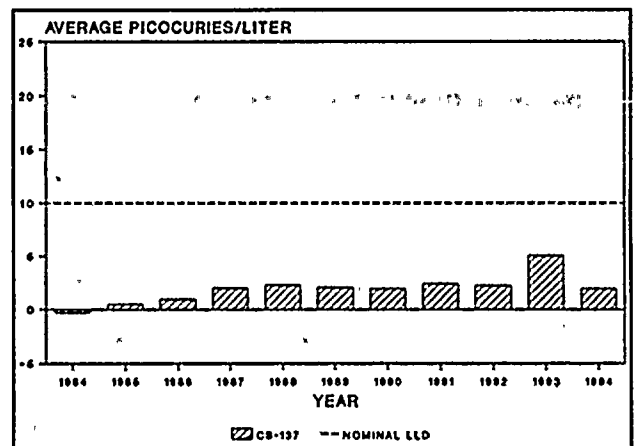


Figure 5-13 Cesium-137 in Discharge Water 1984 - 1994

## 5.4 Soil

Gamma spectrometry performed on soil samples in 1994 indicated a range of cesium-137 from 21.9 picocuries/kilogram (pCi/kg) to 263 pCi/kg at the indicator stations. As shown in Table 5-1, and presented in Figures 5-14 and 5-15, the cesium-137 levels in the soil samples were well within the range observed during preoperational and previous operational sampling. The gamma spectrometry results for the soil samples did not indicate any impact from Plant 2 operations on the environment.

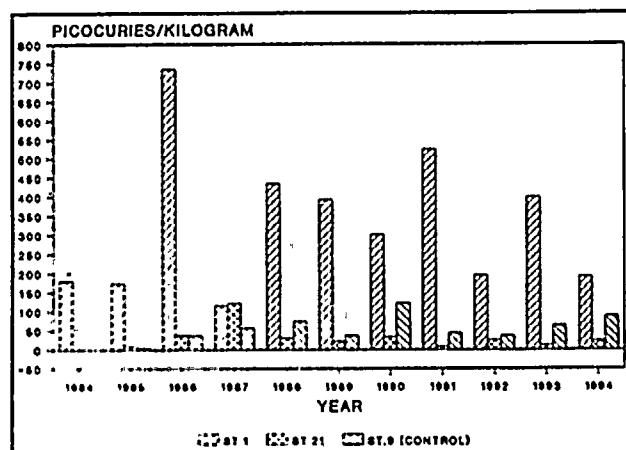


Figure 5-14 Cesium-137 in Soil 1984 - 1994, Stations 1, 21 and 9

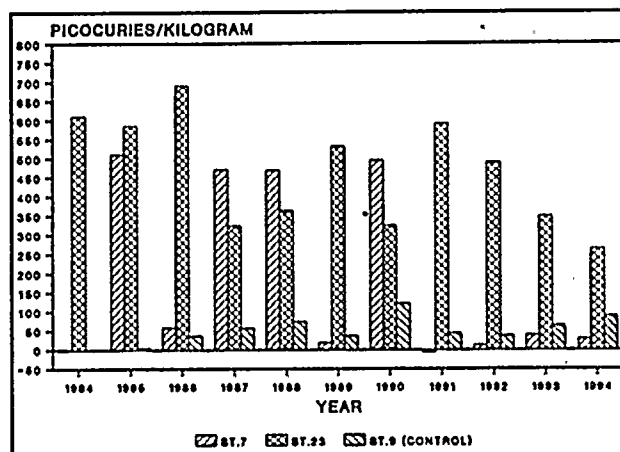


Figure 5-15 Cesium-137 in Soil 1984 - 1994, Stations 7, 23 and 9

No strontium analysis was required in 1994. Aside from cesium-137, the only radionuclides detected in the samples were potassium-40, radium-226 and thorium-228. These are part of the natural radioactivity typically found in soils.

No indication of impact of Plant 2 operations on the environment was evident in these results.

## 5.5 River Sediment

The results of gamma spectrometry of river sediment indicated that aside from the naturally-occurring radionuclides (potassium-40, radium-226 and thorium-228), only cobalt-60 and cesium-137 were detected downstream of the plant (Station 34). Cesium-137 was also detected in the control location (Station 33) sample. The cesium-137 concentrations upstream from the plant discharge were 76.7 and 92.0 pCi/kg dry weight; the concentrations downstream from the discharge were 225 and 235 pCi/kg dry weight. Downstream cobalt-60 concentrations were 18.1 and 37.4 pCi/kg dry weight. These are consistent with the concentrations observed previously.

Cesium-137 has been detected in preoperational samples and in samples taken since plant operation began. It has also been previously identified as a component of the Columbia River sediment originating from the operation of the old Hanford Reservation reactors.<sup>(16)</sup>

## 5.6 Fish

The gamma spectrometry results of fish samples collected in the vicinity of the Plant 2 discharge and at the control location on the Snake River were below detection limits, except for potassium-40, a naturally-occurring radionuclide. The cesium-137 levels for the fish taken from the Columbia River were low and within the range observed during the preoperational and previous operational periods and shown in Figure 5-16.

The cesium-137 results remained at the low levels observed in 1993.

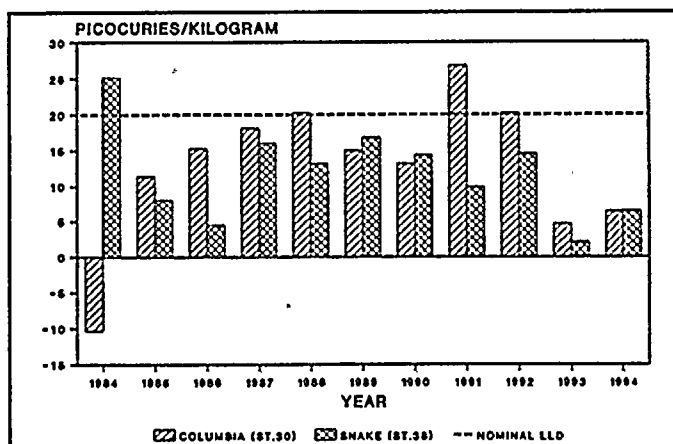


Figure 5-16 Cs-137 in All Fish 1984 - 1994

## 5.7 Milk

All the results of iodine-131 analyses of milk samples collected during 1994 were less than detection limits. All milk sample results for the indicator and control locations were less than the detection limits, except for potassium-40, which is naturally-occurring.

## 5.8 Garden Produce

The gamma isotopic analysis results for all root, fruit and leafy vegetables collected in 1994 were below detection limits other than potassium-40, which occurs naturally.

## 5.9 Special Interest Stations

None of the special interest stations, the storm drain pond, the Sanitary Waste Treatment Facility (SWTF), containerized storage area, were a part of the routine sampling program for the REMP until 1992. Until incorporated into the REMP, the sediment samples collected during previous years at the storm drain and SWTF were analyzed inhouse at the Support Services radiochemical laboratory. The storm drain and SWTF sediment samples were analyzed wet, so the results were in terms of wet weight instead of the dry weight concentrations determined by Teledyne. Consequently, direct comparison of the wet sample results with the dried sample results is difficult since the percent solids can vary from sample to sample.



### 5.9.1 Storm Drain Pond (Station 101)

The storm drain pond is a drain field designed to collect site effluents from several sources which include the plant building roof drains, the Service Building and the Diesel Generator Building floor drains, air handling unit drains, backwash from make up water treatment, and Turbine Building nonradioactive floor drains. The storm drain pond is located within the Plant 2 restricted area approximately 1500 feet northeast of Plant 2. Water is conveyed to the pond via a 18-inch diameter pipe which discharges into a 300-foot long earthen channel that leads to a 100-foot diameter pond. The pond is a shallow, unlined percolation/evaporation basin.

There is some soil contamination in the pond as a result of previous plant operations relating to the Turbine Building sumps and the discharge of low levels of radioactive materials from the sumps. This has necessitated that the pond be posted as a radioactive materials storage area. The initiating problem is described in detail in the Radiological Environmental Monitoring Program 1992 Annual Report.

REMP personnel collected water, sediment and vegetation samples at the outfall during 1994. Additional monthly water and sediment samples were taken from the pond area beginning in July of 1994. Flow proportional composite water samples were collected weekly from the outfall and sediment samples were collected weekly until August, when sampling was changed to a monthly schedule. Vegetation was sampled annually.

Tritium was the predominate isotope detected during 1994. Figure 5-17 shows the monthly average for 1993 versus the 1994 monthly average. Sample results for 1994 indicate the measures taken by the plant have resulted in significant curtailment of the release of tritium to the pond.

The range for positive tritium results at the outfall was from 150 pCi/l to 8800 pCi/l. At the pond, the range for positive results was 160 pCi/l to 820 pCi/l. Gross beta activity at the outfall averaged 4.7 pCi/l with a range of 0.25 to 21.0 pCi/l. Gross beta at the pond ranged from 1.8 pCi/l to 14 pCi/l and averaged 5.6 pCi/l.

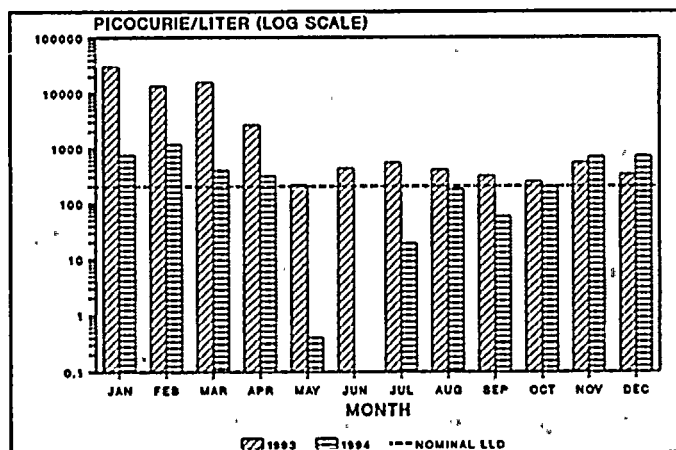


Figure 5-17 Average Monthly Tritium at Storm Drain Outfall - 1993-94

### 5.9.2 Sanitary Waste Treatment Facility (Station 102)

The Sanitary Waste Treatment Facility (SWTF), located approximately 0.3 mile south of Plant 2, processes the sanitary waste from Plant 2, the WNP-1 and WNP-4 sites and the Plant Support Facility (PSF). In April 1991, a sewer pipe was installed from the Department of Energy's Fast Flux Test Facility (FFTF) to the SWTF. The effluent pipe was to be isolated at the FFTF source to prevent flow of wastes until agreements for processing the wastes were finalized. The monitoring results related to an inadvertent discharge of FFTF waste in 1992 are included in the 1992 Annual REMP report. Discharge standards and monitoring requirements for the SWTF are established in EFSEC Resolution No. 259<sup>(17)</sup>. Until April 1992, the SWTF sediment was sampled semiannually and analyzed in the Support Services radiation laboratory and the radionuclide concentrations were given in terms of wet weight.

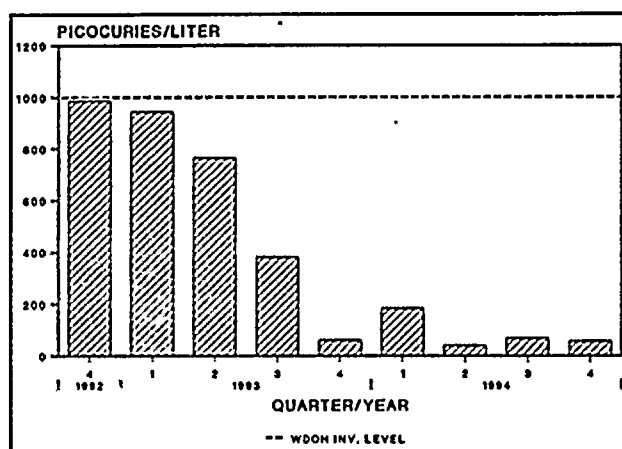


Figure 5-18 Average Tritium in Water 1992 - 1994, ST102C

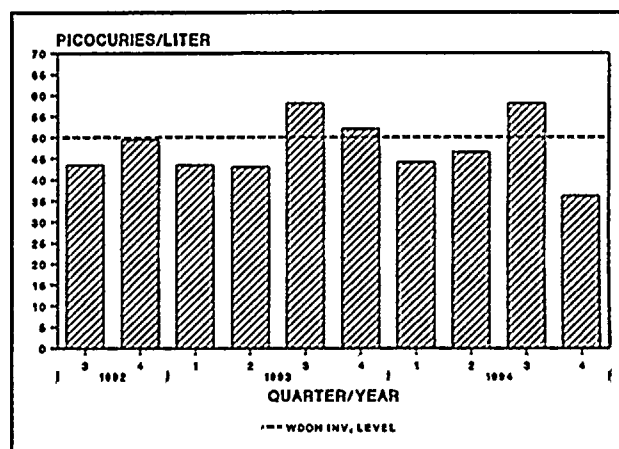


Figure 5-19 Average Gross Beta in Water 1992 - 1994, ST102C

The average quarterly tritium and gross beta results for water samples taken in the south stabilization pond prior to discharge (ST102C) for 1992 through 1994 are presented in Figure 5-18 and Figure 5-19. The pond had indicated gross beta levels at or above the 50 pCi/l EPA drinking water limit in several samples. An investigation into the source of gross beta indicated potassium-40, a natural isotope, was the major contributor. Other contributors to the beta appear to be natural isotopes and no fission or activation products were detected that would indicate Plant 2 as a source.

Gamma analysis of sediment collected from the north pond reveals detectable cobalt-60 and cesium-137 in addition to the naturally occurring radionuclide potassium-40, radium-226 and thorium-228. The range for the cobalt-60 during 1994 was 167 pCi/kg dry weight to 417 pCi/kg dry weight. Cesium-137 ranges were 106 pCi/kg dry weight to 218 pCi/kg dry weight. Both these are within ranges observed since 1992.

### **5.9.3 Containerized Storage Area (Station 118)**

Station 118, consists of twenty-nine large metal storage containers holding the low-pressure turbine rotor parts removed from the plant during the 1992 maintenance outage. Soil samples and ionization chamber readings were taken at Station 118. Beginning in September 1994, samples from different areas were composited and sent to Teledyne Brown for analysis.

Soil samples taken at Station 118 before and after storage of the low pressure turbine rotor parts contained no detectable radioactivity except that from naturally-occurring radionuclides, such as potassium-40 and radium-226.

### **5.10 1994 Sample Deviations**

Air sampler outages made up the majority of sample deviations for 1994. Problems ranged from pump failure to power outages. Sampler problems at ST101 account for the majority of deviation listed for water samples. Deviations are shown in Table 5-2.

TABLE 5-1

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM COMPARATIVE SUMMARY

MEDIA/ ANALYSIS	PREOPERATIONAL <sup>(a)</sup>		PREVIOUS OPERATIONAL <sup>(b)(c)</sup>		1994 <sup>(d)</sup>	
	MEAN	(RANGE)	MEAN	(RANGE)	MEAN	(RANGE)
Air: pCi/m³						
Gross Beta	<0.02	(<0.003 - 0.130)	0.02	(0.00 - 0.74)	0.02	(0.00 - 0.06)
I-131 <sup>(e)</sup>	<0.05	(<0.01 - 0.11)	0.00	(-0.07 - 0.82)	0.00	(-0.02 - 0.01)
Gamma						
Cs-134	<0.01	(<0.001 - 0.040)	0.0004	(-0.0021 - 0.0149)	0.0000	(-0.0002 - 0.0003)
Cs-137	<0.01	(<0.001 - 0.040)	0.0008	(-0.0011 - 0.0356)	0.0000	(-0.0003 - 0.0003)
Ru-103	Not Reported		0.0005	(-0.0008 - 0.0194)	0.0000	(-0.0005 - 0.0005)
River/Drinking Water: pCi/l						
Gross Beta	<3	(<1 - <6)	2	(-0.2 - 9.1)	1.8	(0.0 - 2.6)
Gamma						
Cs-134	<3.8	(<1 - <12)	0.3	(-5.4 - 5.2)	0	(-1.9 - 2.2)
Cs-137	<4.1	(<1 - <13)	1	(-5.7 - 6.2)	0.7	(-3.1 - 3.4)
Co-58	<5.1	(<1 - <25)	-0.1	(-3.3 - 2.9)	-0.1	(-1.7 - 0.9)
Co-60	<4.7	(<1 - <13)	0.9	(-4.9 - 7.1)	0.4	(-4.5 - 3.2)
Fe-59	<13.3	(<2 - <93)	0.7	(-8.9 - 6.9)	0.4	(-4.5 - 4.3)
Zn-65	<8.3	(<2 - <27)	-1.4	(-16.2 - 8.2)	0.4	(-5.2 - 5.8)
H-3	<481.7	(220 - <820)	130.9	(-500 - 596)	101.3	(9.5 - 220)
Sr-90	<2		0.4	(0.3 - 0.7)	Analysis Not Performed	
Groundwater: pCi/l						
Gamma						
Cs-134	<4	(<1 - <12)	0.4	(-4.1 - 5.4)	0.4	(-1.1 - 2.4)
Cs-137	<3.8	(0.8 - <8)	1.1	(-3.1 - 4.9)	0.8	(-4.1 - 4.1)
Co-58	<4.7	(<1 - <12)	-0.3	(-3.3 - 1.9)	-0.7	(-3.3 - 1.3)
Co-60	<4.1	(0.1 - <9)	1.1	(-2.4 - 8.4)	-0.1	(-2.3 - 1.2)
Fe-59	<11.6	(<2 - <33)	0.7	(-4.5 - 5.7)	1.1	(-0.8 - 3.8)
Zn-65	<8.6	(<2 - 17)	-1.3	(-46.8 - 15)	1.3	(-2.4 - 4.1)
H-3	<467.8	(<10 - 2600)	35.1	(-516 - 470)	4.7	(-77 - 160)
Sr-90	<0.4	(<0.1 - 0.7)	Analysis Not Performed		Analysis Not Performed	

(a) All stations, all years.

(b) Indicator stations only for the years 1984 to 1993. Some of the data means and ranges are biased high due to Chernobyl in 1986.

(c) The data used for these averages does not include the "less than" values reported in 1984.

(d) Indicator stations only.

(e) Charcoal cartridge results.

TABLE 5-1 (Cont.)

## RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM COMPARATIVE SUMMARY

MEDIA/ ANALYSIS	PREOPERATIONAL <sup>(a)</sup>		PREVIOUS OPERATIONAL <sup>(b)(c)</sup>		1994 <sup>(d)</sup>	
	MEAN	(RANGE)	MEAN	(RANGE)	MEAN	(RANGE)
Discharge Water: pCi/l						
Gross Beta	<2.8	(<1.9 - 4)	17.9	(0.6 - 56)	15.1	(3.7 - 26)
Gamma						
Cs-134	<3.7	(<1 - <8)	0.7	(-3.9 - 10.1)	0.3	(-1.4 - 2.9)
Cs-137	<4.7	(<1 - 16)	2	(-5.3 - 23.1)	1.9	(0.0 - 5.9)
Co-58	<1.4	(1 - 13)	0.0	(-2.6 - 4.6)	-0.3	(-1.4 - 1.2)
Co-60	<5.0	(<1.9 - <13)	5.9	(-8.7 - 57.6)	6.4	(1.1 - 17.5)
Fe-59	<11.9	(<3 - <38)	0.8	(-4.1 - 13)	0.9	(-2 - 3.6)
Zn-65	<8.6	(<2 - 27)	3.2	(-8.2 - 86.7)	1.6	(-5.4 - 5.4)
H-3	<420	(<80 - 700)	1993	(55 - 12000)	2585	(540 - 3500)
Sr-90	<3		0.8	(0.5 - 1.1)	Analysis Not Performed	
Storm Drain Water: pCi/l						
Gross Beta	Analysis Not Performed		38	(1.6 - 1100)	4.8	(0.3 - 21)
Gamma	Analysis Not Performed					
Cs-134			0.0	(-9.6 - 8.1)	0.3	(-6.2 - 2.6)
Cs-137			1.5	(-11 - 252)	1.7	(-3.2 - 10.5)
Co-58			-0.5	(-7.6 - 3.4)	-0.4	(-4.5 - 2.6)
Co-60			1.1	(-4.2 - 125)	0.5	(-2.1 - 6.7)
Fe-59			0.8	(-14 - 9.9)	0.4	(-3.9 - 4.4)
Zn-65			0.9	(-13 - 53)	0.2	(-8.3 - 12)
Mn-54			0.7	(-3.2 - 6.7)	0.6	(-6.2 - 2.5)
I-131			-0.2	(-17 - 21.1)	0.0	(-4 - 3.1)
Ce-141			0.3	(-19 - 707)	-9.1	(-441 - 5.1)
Sb-124			11.5		Analysis Not Performed	
Sb-125			20.8		Analysis Not Performed	
I-131 <sup>(e)</sup>			0.4	(-0.2 - 8.3)	Analysis Not Performed	
H-3	Analysis Not Performed		7360	(-170 - 270000)	573	(-160 - 8800)
Sr-89	Analysis Not Performed		0.9	(-0.1 - 1.8)	Analysis Not Performed	
Sr-90	Analysis Not Performed		0.4	(-0.1 - 0.9)	Analysis Not Performed	
Sanitary Waste Water: pCi/l						
Gross Alpha	Analysis Not Performed		0.3	(-0.4 - 1.5)	0.4	(-0.7 - 1.6)
Gross Beta	Analysis Not Performed		49	(35 - 61)	38.3	(13 - 60)
Cs-134			0.1	(-4.9 - 2.9)	0.3	(-2.6 - 4.9)
Cs-137			0.6	(-5.1 - 4.2)	1.5	(-2 - 4.2)
Co-58			-0.3	(-2.7 - 1.8)	-0.2	(-1.8 - 1.2)
Co-60			0.6	(-1.7 - 4)	0.1	(-12.9 - 3.1)
H-3	Analyses Not Performed		477	(-110 - 1700)	56	(-120 - 260)

(a) All stations, all years.

(b) Indicator stations only for the years 1984 to 1993. Some of the data means and ranges are biased high due to Chernobyl in 1986

(c) The data used for these averages does not include the "less than" values reported in 1984.

(d) Indicator stations only.

(e) Resin method

TABLE 5-1 (Cont.)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM COMPARATIVE SUMMARY

MEDIA/ ANALYSIS	PREOPERATIONAL <sup>(a)</sup>	PREVIOUS OPERATIONAL <sup>(b)(c)</sup>	1994 <sup>(d)</sup>	
	MEAN (RANGE)	MEAN (RANGE)	MEAN	(RANGE)
River Sediment:				
pCi/kg (dry)				
Gamma				
Cs-134	<112.5 (<50 - <150)	51.7 (7 - 172)	41.4	(28.7 - 54)
Cs-137	<287 (<50.0 - <560.0)	343.8 (153 - 1890)	229.6	(224.6 - 234.6)
Co-60	<254.6 (130 - 610)	42.7 (11 - 129)	27.7	(18.1 - 37.4)
Co-57	Not Reported	27.9 (-11.6 - 54)	6.9	(5.1 - 8.7)
Eu-152	Not Reported	122.8 (5.6 - 279)	135	(134 - 136)
Storm Drain Sediment:				
pCi/kg (dry)				
Gamma:	Analysis Not Performed <sup>(e)</sup>			
Cs-134		77.9 (4.1 - 1140)	39.7	(11.9 - 82.9)
Cs-137		193.9 (-3.6 - 2900)	122.5	(16.3 - 286.1)
Co-58		-0.3 (-27.0 - 58)	-3.7	(-10.4 - 4.8)
Co-60		1002.8 (-6.4 - 25400)	279.2	(69.7 - 868.2)
Fe-59		2 (-55 - 280)	-5.6	(-49.1 - 8.6)
Zn-65		180 (-25 - 4650)	22.2	(-2.1 - 64.2)
Mn-54		32.3 (-9.6 - 670)	6.1	(-2.8 - 18)
Ce-141		56.1 (-29 - 3740)	4	(-13.2 - 19.2)
Sanitary Waste Sediment:				
pCi/kg (dry)				
Gamma:	Analysis Not Performed <sup>(e)</sup>			
Cs-134		25.8 (-15.6 - 55.2)	26.2	(0.0 - 40.3)
Cs-137		143.9 (0.0 - 255)	157.8	(106.1 - 218.3)
Co-60		242.2 (-3.4 - 728)	263.3	(166.6 - 417)
Zn-65		3.4 (-106 - 77.4)	26.0	(10.6 - 47.5)
Mn-54		3.7 (-26 - 32.1)	7.2	(-13.3 - 19.2)
Sr-90	Analysis Not Performed	83.4 (26 - 130)	Analysis Not Performed	
Soil: pCi/kg (dry)				
Gamma				
Cs-134	<65.3 (<20 - <150)	27.1 (1 - 53.2)	21.9	(19.7 - 25.7)
Cs-137	364.3 (<20.0 - <1880.0)	280.4 (-7.3 - 735)	125.7	(21.9 - 262.5)
Sr-90	Analysis Not Performed	178.8 (0.2 - 455)	Analysis Not Performed	

(a) All stations all years.

(b) Indicator stations only for the years 1984 to 1993. Some of the data means and ranges are biased high due to Chernobyl in 1986.

(c) The data used for these averages does not include the "less the" values reported in 1984.

(d) Indicator stations only.

(e) Prior to February 1992, these samples were analyzed as wet weight. These numbers are for the samples analyzed as dry weight.

TABLE-5-1 (Cont.)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM COMPARATIVE SUMMARY

MEDIA/ ANALYSIS	PREOPERATIONAL <sup>(a)</sup>		PREVIOUS OPERATIONAL <sup>(b)(c)</sup>		1994 <sup>(d)</sup>	
	MEAN	(RANGE)	MEAN	(RANGE)	MEAN	(RANGE)
ST 118 Soil: pCi/kg (dry)						
Gamma	Analysis Not Performed					
Cs-134			22.4	(-2.9 - 45.6)	24.4	(19.1 - 31)
Cs-137			14.7	(-8.2 - 47.6)	14.4	(3.6 - 27.8)
Storm Drain Soil: pCi/kg (dry)						
Gamma	Analysis Not Performed					
Cs-134			19.6	(-1.4 - 38)	24.1	(20.1 - 31.4)
Cs-137			51	(24.6 - 77.3)	30	(12.5 - 45)
Sr-90	Analysis Not Performed		Analysis Not Performed		Analysis Not Performed	
Milk: pCi/l						
Gamma						
Cs-134	<3.7	(<0.9 - <14)	0.8	(-7.4 - 22.6)	0.0	(-8.7 - 3)
Cs-137	<3.8	(<1 - <12)	2.5	(-7.4 - 47.3)	0.9	(-3.8 - 2.7)
Ba-140	<72.1	(<6 - <2000)	0.2	(-44.3 - 55)	0.0	(-6.4 - 8.6)
La-140	<33.3	(<5 - 1000)	-0.4	(-24.2 - 9.7)	0.1	(-3.2 - 5.7)
I-131 <sup>(e)</sup>	<0.5	(<0.1 - <1)	0.9	(-0.8 - 143.6)	0.0	(-0.2 - 0.2)
Sr-90	Analysis Not Performed		1.9	(1.3 - 3.9)	Analysis Not Performed	
Fish: pCi/kg (wet)						
Gamma						
Cs-134	<61.2	(<6 - <130)	2	(-20.4 - 24)	-2.3	(-7.5 - 2.3)
Cs-137	<88.8	(<10 - <130)	15.2	(-35.1 - 57)	6.4	(-10 - 14.2)
Co-58	<87.7	(<9 - <130)	0.7	(-16.8 - 25.8)	-0.6	(-7.2 - 4.9)
Co-60	<80.6	(<9 - <130)	1.7	(-18.4 - 21)	-0.6	(-3.4 - 1.8)
Fe-59	<130	(<30 - <260)	-0.3	(-34.2 - 30)	2.9	(-7.2 - 10)
Mn-54	<88.3	(<8 - <130)	1.3	(-20 - 30.9)	2.8	(1.4 - 4.8)
Produce: pCi/kg (wet)						
Gamma						
Cs-134	<49.1	(<10 - <140)	1	(-24.8 - 19.8)	-1.1	(-8.3 - 2.5)
Cs-137	<69.8	(<10 - <140)	3.6	(-9.8 - 20.9)	1.7	(-0.8 - 4.3)
I-131	<105.6	(<10 - <1000)	-0.3	(-26 - 59)	-0.9	(-13.3 - 2.3)

(a) All stations, all years.

(b) Indicator stations only for the years 1984 to 1993. Some of the data means and ranges are biased high due to Chernobyl in 1986.

(c) The data used for these averages does not include the "less than" values reported in 1984.

(d) Indicator stations only.

(e) Resin method.

TABLE 5-1 (Cont.)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM COMPARATIVE SUMMARY

MEDIA/ ANALYSIS	<u>PREOPERATIONAL<sup>(a)</sup></u>		<u>PREVIOUS OPERATIONAL<sup>(b)(c)</sup></u>		<u>1994<sup>(d)</sup></u>	
	MEAN	(RANGE)	MEAN	(RANGE)	MEAN	(RANGE)
Storm Drain Vegetation <sup>(e)</sup> pCi/kg (wet)						
Gamma	Analysis Not Performed					
Mn-54			14.7	(-2 - 32.2)	-0.7	
Co-60			23.8	(-3.7 - 48.2)	-1.9	
Zn-65			34.6	(4.1 - 57.4)	-4.3	
Cs-134			9.1	(-6.5 - 45.8)	-2	
Cs-137			29.9	(3.9 - 93.5)	10.6	(10.6 - 10.6)
TLD: mR/day						
Quarterly	0.24	(0.11 - 0.32)	0.25	(0.16 - 0.35)	0.26	(0.23 - 0.32)
Annual	0.23	(0.20 - 0.31)	0.23	(0.18 - 0.32)	0.23	(0.20 - 0.28)

(a) All stations, all years.

(b) Indicator stations only for the years 1984 to 1993. Some of the data means and ranges are biased high due to Chernobyl in 1986.

(c) The data used for these averages does not include the "less than" values reported in 1984.

(d) Indicator Stations only.

(e) Routine samples from the outfall only.



TABLE 5-2

1994 SAMPLE DEVIATIONS

SAMPLE MEDIA	DATE	LOCATION	PROBLEM
Air Particulate/Iodine	02/07 - 02/14	Station 40	Unit Failure, Reduced hours
	02/28 - 03/07	Station 4	Unit Failure, Reduced hours
	05/02 - 05/09	Station 7	Power Outage, Reduced hours
	05/16 - 05/23	Station 48	Power Outage, Reduced hours
	06/06 - 06/13	Station 48	Pump Failure, Reduced hours
	06/13 - 06/20	Station 8	Power Outage, Reduced hours
	07/05 - 07/11	Station 4	Pump Failure, Reduced hours
	07/18 - 07/25	Station 7	Pump Failure, Reduced hours
	08/15 - 08/22	Station 40	Power Outage, Reduced Hours
	08/22 - 08/29	Station 5	Pump Failure, Acceptable Hours
	08/22 - 08/29	Station 23	Pump Failure, Reduced hours
	08/22 - 08/29	Station 40	Power Outage, Reduced Hours
	09/12 - 09/19	Station 5	Pump Failure, Reduced Hours
	09/12 - 09/19	Station 40	Pump Failure, Reduced Hours
	10/17 - 10/24	Station 5	Pump Failure, Reduced Hours
	10/31 - 11/07	Station 5	Pump Failure, Reduced Hours
	11/07 - 11/14	Station 40	Power Outage, Reduced Hours
	11/14 - 11/21	Station 40	Power Outage, Reduced Hours
	11/21 - 11/28	Station 40	Power Outage, Reduced Hours
	11/21 - 11/28	Station 57	Pump Failure, Reduced Hours
	12/19 - 12/27	Station 40	Pump Failure, Reduced Hours
Water	01/06 - 02/02	Station 27	Maintenance Being Performed, unit in timed mode.
	01/19 - 01/26	ST 101	Battery died, no sample.
	03/24 - 03/31	ST 101	Collection bottles capped, no individual samples collected.
	04/25 - 04/27	ST101	Sampler in for calibration, grab samples collected.
	05/11 - 06/21	Station 27	Timed Mode for Plant Outage Work.

TABLE 5-2 (Cont.)

1994 SAMPLE DEVIATIONS

SAMPLE MEDIA	DATE	LOCATION	PROBLEM
Water (cont.)	06/14 - 06/20	ST101	Intake tubing cracked, no samples.
	06/21 - 06/28	Station 27	Out of service for repair.
	07/12 - 08/01	ST101	Sample volume low; samples acceptable
	08/18 - 08/26	ST101	Sample volume calibration lost, sampler base flooded. No individual samples.
	09/12 - 09/16	ST101	Sampler in pause mode, no samples collected.
	10/09 - 10/19	ST101	Sample volume calibration lost, sampler base flooded. No individual samples.
	10/25 - 10/27	ST101	Sampler in pause mode, no samples collected.
	10/29 - 11/10	Station 26	Pump failure, grab samples taken; normal composite sample collected.
	11/23 - 11/28	ST101	Sample in manual mode, no samples collected.
	11/17 - 11/30	ST101	24 samples missed due to intake line freezing.
	12/13 - 12/16	ST101	Sampler brought in for calibration, grab sample taken 12/14.
Soil	12/94	Station 118	Sample not collected

TABLE 5-3

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**  
**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2**  
**HANFORD WASHINGTON**

DOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	<u>All Indicator Locations</u> Mean (Ratio) (a) (Range)	<u>Location With Highest Mean</u> Name Distance and Direction	Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
Air Particulates (pCi/m <sup>3</sup> )	Gross Beta	621 0.003	0.016(567/569) (0.004-0.063)	48 4.5 ml NE	0.017(52/52) (0.006-0.060)	0.015(52/52) (0.004-0.043)	0
	Gamma (Quarterly)	48					
	Be-7	0.01	0.130(44/44) (0.08-0.214)	48 4.5 ml NE	0.136(4/4) (0.093-0.192)	0.1274(4/4) (0.093-0.176)	0
	K-40	0.01	0.006(7/44) (0.005-0.007)	9A 30.0 ml WSW	0.017(2/4) (0.007-0.026)	0.017(2/4) (0.007-0.026)	0
Air Iodine (pCi/m <sup>3</sup> )	I-131	621 0.01	-(0/569)			-(0/52)	0
Soil (pCi/kg dry)	Gamma	5					
	K-40	700	12800(4/4) (11500-13600)	7 2.7 ml WNW	13600(1/1)	11700(1/1)	0
	Cs-137	40	126(4/4) (21.9-263)	23 1.3 ml S	263(1/1)	87.9(1/1)	0
	Ra-226	400	733(4/4) (621-887)	01 1.3 ml S	887(1/1)	810(1/1)	0
	Th-228	50	584(4/4) (418-696)	23 1.3 ml S	696(1/1)	529(1/1)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARYWASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2  
HANFORD WASHINGTONDOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)		<u>All Indicator Locations</u> Mean (Ratio) (a) (Range)	<u>Location With Highest Mean</u> Name Distance and Direction		Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
Water (River/Drinking) (pCi/liter)	Gross Beta	37	4	2.04(19/24) (1.2-2.9)	29		2.02(9/12) (1.2-2.9)	1.95(10/13) (1.1-3.1)	0
	Tritium	12	200	193(3/8) (170-220)	28	7.4 ml SSE	205(2/4) (190-220)	-(0/4)	0
	Gamma	36							
	K-40		200	-(0/24)				-(0/12)	0
Water (Discharge) (pCi/liter)	Gross Beta	12	12	15.1(12/12) (3.7-26.0)	27	3.2 ml E	15.1(12/12) (3.7-26.0)	-(0/0)	0
	Tritium	4	200	2575(4/4) (500-3500)	27	3.2 ml E	2575(4/4) (500-3500)	-(0/0)	0
	Gamma	12							
	K-40		10	46.8(1/12)	27	3.2 ml E	46.8(1/12)	-(0/0)	0
	Co-60		10	9.67(6/12) (4.63-17.5)	27	3.2 ml E	9.67(6/12) (4.63-17.5)	-(0/0)	0
	Cs-137		10	5.86(1/12)	27	3.2 ml E	5.86(1/12)	-(0/0)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**  
**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2**  
**HANFORD WASHINGTON**

DOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Ratio) (Range) (a)	Location With Highest Mean Name Distance and Direction	Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
Water (Ground) (pCi/liter)	Tritium 12	200	160(1/12)	52 0.1 ml N	160(1/4)	None	0
	Gamma 12						
	K-40		-(0/12)				0
Sediment (pCi/kg dry)	Gamma 4						
	K-40	700	15300(2/2) (14100-16500)	34 3.5 ml ESE	(15300(2/2) (14100-16500)	14400(2/2) (14200-14600)	0
	Co-60	30	27.8(2/2) (18.1-37.4)	34 3.5 ml ESE	27.8(2/2) (18.1-37.4)	-(0/2)	0
	Cs-137	40	230(2/2) (225-235)	34 3.5 ml ESE	230(2/2) (225-235)	84.4(2/2) (76.7-92.0)	0
	Ra-226	400	1310(2/2) (1020-1600)	33 3.6 ml ENE	1375(2/2) (1310-1440)	1375(2/2) (1310-1440)	0
	Th-228	50	921(2/2) (692-1150)	33 3.6 ml ENE	1064(2/2) (988-1140)	1064(2/2) (988-1140)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**  
**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2**  
**HANFORD WASHINGTON**

DOCKET NO. 50-397  
 JANUARY 1 to DECEMBER 31, 1994.

MEDIUM OR PATHWAY SAMPLED (UNIT OF MEASUREMENT)	ANALYSIS AND TOTAL NUMBER OF ANALYSES PERFORMED		LOWER LIMIT OF DETECTION (LLD)	ALL INDICATOR LOCATIONS	LOCATION WITH HIGHEST MEAN		CONTROL LOCATION MEAN (RATIO) (a) (RANGE)	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
				MEAN (RATIO) (a) (RANGE)	NAME DISTANCE AND DIRECTION	MEAN (RATIO) (a) (RANGE)		
Fish (pCi/kg wet)	Gamma	8						
	K-40		1000	3088(4/4) (2470-3920)	38 27 ml ESE	3213(4/4) (2970-3640)	3213(4/4) (2970-3640)	0
Milk (pCi/liter)	I-131	75	1	-(0/57)			-(0/18)	0
	Gamma	75						
	K-40	75	200	1369(57/57) (1180-1650)	36 7.2 ml ESE	1409(18/18) (1250-1520)	1398(18/18) (1190-1500)	0
Roots (pCi/kg wet)	Gamma	7		-(0/4)	N/A		-(0/3)	0
Fruits (pCi/kg wet)	Gamma	9		-(0/5)			-(0/4)	0
Vegetables (pCi/kg wet)	Gamma	11						
	Th-228	11		-(0/5)			-(0/6)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2  
HANFORD WASHINGTONDOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

MEDIUM OR PATHWAY SAMPLED (UNIT OF MEASUREMENT)	ANALYSIS AND TOTAL NUMBER OF ANALYSES PERFORMED	LOWER LIMIT OF DETECTION (LLD)	ALL INDICATOR LOCATIONS MEAN (RATIO) <sup>(a)</sup> (RANGE)	LOCATION WITH HIGHEST MEAN		CONTROL LOCATION MEAN (RATIO) <sup>(a)</sup> (RANGE)	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
				NAME DISTANCE AND DIRECTION	MEAN (RATIO) <sup>(a)</sup> (RANGE)		
Direct Radiation Quarterly TLD's (mR/day)	TLD 224	-	0.26(220/220) (0.23-0.32)	46 5.0 ml NE	0.31(4/4) (0.30-0.32)	0.24(4/4) (0.23-0.25)	0
Direct Radiation Annual TLD's (mR/day)	TLD 56	-	0.23(55/55) (0.20-0.28)	46 5.0 ml NE	0.28(1/1)	0.21(1/1)	0
Storm Drain Water Station 101 (pCi/liter)	Gr-Beta 44		5.93(30/44) (2.90-21.0)	101 0.3 ml ENE	5.93(30/44) (2.90-21.0)	-(0/0)	0
	Tritium 56	200	1023(31/56) (150-8800)	101 0.3 ml ENE	1023(31/56) (150-8800)	-(0/0)	0
	Gamma 56						
	K-40	200	72.1(6/56) (50.6-105)	101 0.3 ml ENE	72.1(6/56) (50.6-105)	-(0/0)	0
	Cs-137	10	10.3(1/56)	101 0.3 ml ENE	10.3(1/56)	-(0/0)	0
	Th-228	10	15.5(1/56)	101 0.3 ml ENE	15.5(1/56)	-(0/0)	0
Storm Drain Water Station 101E (pCi/liter)	Gr-Beta 5		5.56(5/5) (1.8-14.0)	101 0.3 ml ENE	5.56(5/5) (1.8-14.0)	-(0/0)	0
	Tritium 5	200	537(3/5) (160-820)	101 0.3 ml ENE	537(3/5) (160-820)	-(0/0)	0
	Gamma 5		-(0/5)			-(0/0)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**  
**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2**  
**HANFORD WASHINGTON**

DOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Ratio) (a) (Range)	Location With Highest Mean Name Distance and Direction	Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
<b>Storm Drain Sediment Station 101-Outfall (pCi/kg)</b>							
	Gamma	36					
	K-40	700	9070(36/36) (1890-13900)	101 0.3 ml ENE	9070(36/36) (1890-13900)	-(0/0)	0
	Mn-54	40	15.4(2/36) (12.7-18.0)	101 0.3 ml ENE	15.4(2/36) (12.7-18.0)	-(0/0)	0
	Co-60	30	286(36/36) (133-868)	101 0.3 ml ENE	286(36/36) (133-868)	-(0/0)	0
	Cs-134	30	49.7(23/36) (23.7-82.9)	101 0.3 ml ENE	49.7(23/36) (23.7-82.9)	-(0/0)	0
	Cs-137	40	133(36/36) (25.5-286)	101 0.3 ml ENE	133(36/36) (25.5-286)	-(0/0)	0
	Ra-226	400	861(36/36) (524-1260)	101 0.3 ml ENE	861(36/36) (524-1260)	-(0/0)	0
	Th-228	50	446(36/36) (310-793)	101 0.3 ml ENE	446(36/36) (310-793)	-(0/0)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.



TABLE 5-3 (Cont.)

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2  
HANFORD WASHINGTONDOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Ratio) (a) (Range)	Location With Highest Mean Name Distance and Direction	Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
<b>Storm Drain Sediment Station 101C-Pond (pCi/kg)</b>							
	Gamma	5					
	K-40	700	14920(5/5) (13500-16700)	101 0.3 ml ENE	14920(5/5) (13500-16700)	-(0/0)	0
	Co-60	30	232(5/5) (69.7-504)	101 0.3 ml ENE	232(5/5) (69.7-504)	-(0/0)	0
	Cs-137	40	65.4(3/5) (42.8-107)	101 0.3 ml ENE	65.4(3/5) (42.8-107)	-(0/0)	0
	Ra-226	400	1306(5/5) (1140-1500)	101 0.3 ml ENE	1306(5/5) (1140-1500)	-(0/0)	0
	Th-228	50	678(5/5) (582-783)	101 0.3 ml ENE	678(5/5) (582-783)	-(0/0)	0
<b>Storm Drain Soil (pCi/kg)</b>							
	Gamma	8					
	K-40	700	15250(8/8) (13800-17300)	101 0.3 ml ENE	15250(8/8) (13800-17300)	-(0/0)	0
	Cs-137	40	36.4(7/8) (22.1-45.0)	101 0.3 ml ENE	36.4(7/8) (22.1-45.0)	-(0/0)	0
	Ra-226	400	785(8/8) (595-923)	101 0.3 ml ENE	785(8/8) (595-923)	-(0/0)	0
	Th-228	50	503(8/8) (411-553)	101 0.3 ml ENE	503(8/8) (411-553)	-(0/0)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

**RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARY**  
**WASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2**  
**HANFORD WASHINGTON**

DOCKET NO. 50-397  
 JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Ratio) (a) (Range)	Location With Highest Mean Name Distance and Direction	Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
<b>Storm Drain Vegetation</b>							
(pCi/kg wet)	Gamma	1					
	Be-7		583(1/1)	101 0.3 mi ENE	583(1/1)	-(0/0)	0
	K-40		3180(1/1)	101 0.3 mi ENE	3180(1/1)	-(0/0)	0
<b>Sanitary Waste Treatment Facility Water (pCi/l)</b>	Gross Alpha	10	3			-(0/0)	0
	Gross Beta	31	1			-(0/0)	0
	Tritium	37	200			-(0/0)	0
	Gamma	37					
	K-40		73.9(5/37) (44.9-116)	102C 0.3 mi ENE	73.9(5/37) (44.9-116)	-(0/0)	0
<b>Sanitary Waste Treatment Facility Sediment (pCi/kg)</b>	Gamma	8					
	K-40		700	102 0.3 mi ENE	8760(8/8) (6040-15000)	-(0/0)	0
	Co-60		30	102 0.3 mi ENE	264(8/8) (167-417)	-(0/0)	0
	Cs-137		40	102 0.3 mi ENE	158(8/8) (106-218)	-(0/0)	0
	Ra-226		400	102 0.3 mi ENE	1336(8/8) (842-1700)	-(0/0)	0
	Th-228		50	102 0.3 mi ENE	489(8/8)	-(0/0)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-3 (Cont.)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM SUMMARYWASHINGTON PUBLIC POWER SUPPLY SYSTEM WNP-2  
HANFORD WASHINGTONDOCKET NO. 50-397  
JANUARY 1 to DECEMBER 31, 1994

Medium Or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Ratio) (a) (Range)	Location With Highest Mean Name Distance and Direction	Mean Ratio(a) (Range)	Control Location MEAN (RATIO) (a) (RANGE)	Number of Nonroutine Reported Measurements
Station 118 Soil (pCi/kg dry)							
	Gamma	15					
	K-40	700	12813(15/15) (11700-14400)	118 0.3 ml S	12813(15/15) (11700-14400)	-(0/0)	0
	Cs-137	40	17.3(8/15) (12.4-27.8)	118 0.3 ml S	17.3(8/15) (12.4-27.8)	-(0/0)	0
	Ra-226	700	790(15/15) (622-922)	118 0.3 ml S	790(15/15) (622-922)	-(0/0)	0
	Th-228	50	522(15/15) (441-662)	118 0.3 ml S	522(15/15) (441-662)	-(0/0)	0

(a) The ratio of positive results above the LLD to the number of samples analyzed for the parameter of interest.

TABLE 5-4  
MEAN QUARTERLY TLD DATA SUMMARY FOR THE PREOPERATIONAL  
AND OPERATIONAL PERIODS  
 Results in mR/day

STATION	<u>PREOPERATIONAL</u>		<u>1984 - 1993 OPERATIONAL</u>		<u>1994 OPERATIONAL</u>	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN	STANDARD ERROR	MEAN	STANDARD ERROR
1	0.24	0.02	0.25	0.01	0.26	0.01
2	0.23	0.02	0.25	0.01	0.25	0.01
3	0.22	0.01	0.24	0.01	0.25	0.01
4	0.22	0.02	0.23	0.01	0.24	0.01
5	0.23	0.01	0.23	0.01	0.24	0.01
6	0.22	0.01	0.23	0.01	0.24	0.01
7	0.23	0.01	0.24	0.01	0.26	0.00
8	0.26	0.01	0.27	0.01	0.28	0.01
9	0.22	0.01	0.23	0.01	0.24	0.01
10	0.23	0.01	0.24	0.01	0.25	0.01
11	0.24	0.01	0.24	0.01	0.26	0.01
12	0.25	0.01	0.26	0.01	0.27	0.01
13	0.24	0.01	0.25	0.01	0.26	0.01
14	0.24	0.02	0.25	0.01	0.25	0.01
15	0.25	0.01	0.26	0.01	0.27	0.01
16	0.24	0.01	0.25	0.01	0.26	0.01
17	0.25	0.01	0.25	0.01	0.26	0.01
18	0.24	0.01	0.25	0.01	0.26	0.01
19	0.24	0.01	0.25	0.01	0.25	0.01
20	0.24	0.01	0.25	0.01	0.26	0.00
21	0.23	0.01	0.23	0.01	0.24	0.01
22	0.24	0.01	0.25	0.01	0.26	0.01
23	0.24	0.01	0.24	0.01	0.25	0.01
24	0.24	0.01	0.25	0.01	0.26	0.01
25	0.25	0.01	0.26	0.01	0.27	0.01
40	0.22	0.01	0.23	0.01	0.23	0.00
41	0.26	0.02	0.26	0.01	0.27	0.01
42	0.25	0.01	0.25	0.01	0.26	0.01
43	0.25	0.01	0.26	0.01	0.27	0.01
44	0.23	0.01	0.24	0.01	0.25	0.01
45	0.23	0.01	0.24	0.01	0.25	0.01
46	0.28	0.02	0.30	0.01	0.31	0.01
47	0.22	0.02	0.23	0.01	0.24	0.01
49	0.24	0.00	0.25	0.01	0.26	0.01
50	0.22	0.00	0.25	0.01	0.25	0.01
51	0.23	0.01	0.24	0.01	0.25	0.01

(a) This preoperational mean is for the 1982-1983 data only.

TABLE 5-4 (Cont.)

MEAN QUARTERLY TLD DATA SUMMARY FOR THE PREOPERATIONAL  
AND OPERATIONAL PERIODS

Results in mR/day

STATION	<u>PREOPERATIONAL</u>		<u>1984 - 1993 OPERATIONAL</u>		<u>1994 OPERATIONAL</u>	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN	STANDARD ERROR	MEAN	STANDARD ERROR
53	0.27	0.00	0.27	0.01	0.27	0.02
54	0.26	0.00	0.26	0.01	0.26	0.01
55	0.23	0.00	0.24	0.01	0.25	0.01
56	0.24	0.00	0.25	0.01	0.27	0.01
61	(b)		0.27	0.01	(b)	
71(1S)	0.24	0.02	0.28	0.01	0.30	0.02
72(2S)	0.25	0.01	0.27	0.01	0.28	0.01
73(3S)	0.23	0.01	0.24	0.01	0.25	0.01
74(4S)	0.26	0.01	0.27	0.01	0.28	0.01
75(5S)	0.22	0.02	0.25	0.01	0.27	0.01
76(6S)	0.24	0.01	0.25	0.01	0.26	0.01
77(7S)	0.25	0.01	0.25	0.01	0.27	0.00
78(8S)	0.25	0.01	0.25	0.01	0.25	0.01
79(9S)	0.25	0.01	0.25	0.01	0.26	0.01
80(10S)	0.24	0.01	0.24	0.01	0.25	0.01
81(11S)	0.24	0.02	0.25	0.01	0.26	0.01
82(12S)	0.25	0.02	0.25	0.01	0.27	0.01
83(13S)	0.25	0.01	0.26	0.01	0.27	0.01
84(14S)	0.24	0.01	0.25	0.01	0.26	0.01
85(15S)	0.26	0.01	0.26	0.01	0.27	0.01
86(16S)	0.25	0.01	0.28	0.01	0.30	0.02
All	0.24	0.00	0.25	0.01	0.26	0.00

(a) This preoperational mean is for 1982-1983 data only.

(b) Station 61 was added in 1989 and discontinued in 1992.

TABLE 5-5

ANNUAL TLD DATA SUMMARY FOR THE PREOPERATIONAL  
AND OPERATIONAL PERIODS

Results in mR/day

STATION	<u>PREOPERATIONAL</u>		<u>1984 - 1993 OPERATIONAL</u>		<u>1994 OPERATIONAL</u>	
	MEAN(a)	STANDARD ERROR	MEAN	STANDARD ERROR	MEAN	STANDARD DEVIATION(2 $\sigma$ )
1	0.23	0.04	0.24	0.01	0.23	0.01
2	0.22	0.04	0.23	0.01	0.23	0.01
3	0.21	0.04	0.22	0.01	0.22	0.01
4	0.22	0.06	0.21	0.01	0.22	0.00
5	0.22	0.04	0.21	0.02	0.21	0.01
6	0.21	0.04	0.21	0.02	0.22	0.01
7	0.22	0.02	0.23(b)	0.01	0.23	0.01
8	0.25	0.04	0.25(b)	0.02	0.26	0.01
9	0.20	0.02	0.21	0.02	0.21	0.01
10	0.22	0.04	0.22	0.02	0.22	0.01
11	0.22	0.04	0.23	0.02	0.23	0.01
12	0.24	0.04	0.24	0.02	0.25	0.01
13	0.22	0.04	0.23	0.01	0.23	0.01
14	0.22	0.04	0.22	0.02	0.23	0.01
15	0.23	0.06	0.24	0.02	0.24	0.01
16	0.23	0.04	0.23	0.02	0.24	0.01
17	0.23	0.02	0.24(b)	0.02	0.23	0.01
18	0.25	0.02	0.23	0.02	0.24	0.01
19	0.22	0.04	0.23	0.01	0.23	0.01
20	0.23	0.04	0.23	0.02	0.23	0.01
21	0.23	0.02	0.21	0.02	0.21	0.01
22	0.22	0.02	0.23	0.02	0.23	0.01
23	0.24	0.02	0.23	0.02	0.23	0.01
24	0.22	0.02	0.23	0.02	0.23	0.01
25	0.24	0.02	0.25	0.02	0.24	0.01
40	0.21(c)	0.02	0.22	0.02	0.20	0.01
41	0.26(c)	0.04	0.24	0.02	0.23	0.01
42	0.24(c)	0.02	0.23	0.02	0.25	0.01
43	0.24(c)	0.02	0.24	0.02	0.25	0.01
44	0.24	0.02	0.22	0.02	0.22	0.01
45	0.24	0.01	0.23	0.02	0.23	0.01
46	0.29	0.01	0.28	0.02	0.28	0.01
47	0.22(c)	0.03	0.22	0.02	0.22	0.00
49	(d)		0.23	0.01	0.23	0.01

(a) This preoperational mean is for 1982 - 1983 data only.

(b) 1985 TLD missing.

(c) There was only one annual exchange during the preoperational period.

(d) Stations 49-56 were first monitored during Fourth Quarter 1983. Station 61 was added in 1989 and discontinued in 1992.

TABLE 5-5 (Cont.)

**ANNUAL TLD DATA SUMMARY FOR THE PREOPERATIONAL  
AND OPERATIONAL PERIODS**

Results in mR/day

STATION	PREOPERATIONAL		1984 - 1993 OPERATIONAL		1994 OPERATIONAL	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN	STANDARD ERROR	MEAN	STANDARD DEVIATION(2 $\sigma$ )
50	(d)		0.23	0.02	0.22	0.00
51	(d)		0.22	0.02	0.23	0.01
53	(d)		0.25	0.02	0.25	0.01
54	(d)		0.24	0.02	0.24	0.01
55	(d)		0.23	0.02	0.22	0.01
56	(d)		0.23	0.02	0.23	0.01
61	(d)		0.26 <sup>(d)</sup>	0.02	(e)	
71	0.24 <sup>(a)</sup>	0.02	0.26	0.02	0.27	0.01
72	0.25 <sup>(a)</sup>	0.02	0.25	0.02	0.27	0.01
73	0.23 <sup>(a)</sup>	0.01	0.22	0.02	0.23	0.01
74	0.24 <sup>(a)</sup>	0.01	0.25	0.02	0.25	0.01
75	0.24 <sup>(a)</sup>	0.01	0.23	0.02	0.24	0.01
76	0.24 <sup>(a)</sup>	0.02	0.23	0.02	0.23	0.01
77	0.25 <sup>(a)</sup>	0.02	0.23	0.02	0.23	0.01
78	0.25 <sup>(a)</sup>	0.04	0.23	0.02	0.22	0.01
79	0.25 <sup>(a)</sup>	0.02	0.23	0.02	0.23	0.01
80	0.23 <sup>(a)</sup>	0.05	0.22	0.02	0.22	0.00
81	0.23 <sup>(a)</sup>	0.02	0.23	0.02	0.23	0.01
82	0.25 <sup>(a)</sup>	0.03	0.24	0.01	0.24	0.01
83	0.25 <sup>(a)</sup>	0.02	0.24	0.02	0.25	0.01
84	0.23 <sup>(a)</sup>	0.02	0.23	0.02	0.24	0.01
85	0.26 <sup>(a)</sup>	0.02	0.25	0.02	0.25	0.01
86	0.24	0.02	0.26	0.02	0.27	0.01
All	0.23	0.02	0.23	0.00	0.23	0.03

(a) This preoperational mean is for 1982 - 1983 data only.

(b) 1985 TLD missing.

(c) There was only one annual exchange during the preoperational period.

(d) Stations 49-56 were first monitored during Fourth Quarter 1983. Station 61 was added in 1989.

(e) Station 61 discontinued on June 29, 1992

TABLE 5-6

1994 MEAN QUARTERLY VERSUS ANNUAL TLD DATA

Results in mR/day

STATION	QUARTERLY TLDs		ANNUAL TLDs	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN <sup>(b)</sup>	STANDARD DEVIATION(2 $\sigma$ )
1	0.26	0.01	0.23	0.01
2	0.25	0.01	0.23	0.01
3	0.25	0.01	0.22	0.01
4	0.24	0.01	0.22	0.00
5	0.24	0.01	0.21	0.01
6	0.24	0.01	0.22	0.01
7	0.26	0.00	0.23	0.01
8	0.28	0.01	0.26	0.01
9	0.24	0.01	0.21	0.01
10	0.25	0.01	0.22	0.01
11	0.26	0.01	0.23	0.01
12	0.27	0.01	0.25	0.01
13	0.26	0.01	0.23	0.01
14	0.25	0.01	0.23	0.01
15	0.27	0.01	0.24	0.01
16	0.26	0.01	0.24	0.01
17	0.26	0.01	0.23	0.01
18	0.26	0.01	0.24	0.01
19	0.25	0.01	0.23	0.01
20	0.25	0.00	0.23	0.01
21	0.24	0.01	0.21	0.01
22	0.26	0.01	0.23	0.01
23	0.25	0.01	0.23	0.01
24	0.26	0.01	0.23	0.01
25	0.27	0.01	0.24	0.01
40	0.23	0.00	0.20	0.01
41	0.27	0.01	0.23	0.01
42	0.26	0.01	0.25	0.01
43	0.27	0.01	0.25	0.01
44	0.25	0.01	0.22	0.01
45	0.25	0.01	0.23	0.01
46	0.31	0.01	0.28	0.01
47	0.24	0.01	0.22	0.00
49	0.26	0.01	0.23	0.01
50	0.25	0.01	0.22	0.00

(a) Mean of the quarterly results.

(b) Mean of four readout areas on each TLD.

(c) TLD missing.



TABLE 5-6 (Cont.)

1994 MEAN QUARTERLY VERSUS ANNUAL TLD DATA  
Results in mR/day

STATION	<u>QUARTERLY TLDS</u>		<u>ANNUAL TLDS</u>	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN <sup>(b)</sup>	STANDARD DEVIATION(2 $\sigma$ )
51	0.25	0.01	0.23	0.01
53	0.27	0.02	0.25	0.01
54	0.26	0.01	0.24	0.01
55	0.25	0.01	0.22	0.01
56	0.27	0.01	0.23	0.01
61	(c)		(c)	0.01
71 (1S)	0.30	0.02	0.27	0.01
72 (2S)	0.28	0.01	0.27	0.01
73 (3S)	0.25	0.01	0.23	0.01
74 (4S)	0.28	0.01	0.25	0.01
75 (5S)	0.27	0.01	0.24	0.01
76 (6S)	0.26	0.01	0.23	0.01
77 (7S)	0.27	0.00	0.23	0.01
78 (8S)	0.25	0.01	0.22	0.01
79 (9S)	0.26	0.01	0.23	0.01
80 (10S)	0.25	0.01	0.22	0.00
81 (11S)	0.26	0.01	0.23	0.01
82 (12S)	0.27	0.01	0.24	0.01
83 (13S)	0.27	0.01	0.25	0.01
84 (14S)	0.26	0.01	0.24	0.01
85 (15S)	0.27	0.01	0.25	0.01
86 (16S)	0.30	0.02	0.27	0.01

(a) Mean of the quarterly mean results.

(b) Mean of four readout areas on each TLD.

(c) Station 61 was discontinued on June 29, 1992.

## 6.0 QUALITY ASSURANCE AND QUALITY CONTROL

The REMP is designed to meet the quality assurance and quality control criteria of Regulatory Guide 4.15<sup>(4)</sup>. To accomplish this, the REMP requires that its analytical contractors meet these criteria also. In-depth audits are performed of the REMP records and activities and the records and activities of its support organizations at least annually by the Supply System Quality Assurance group.

Quality assurance and technical audits of the analytical contractor (Teledyne Brown Engineering) are also conducted annually to verify their compliance to regulatory and contractual requirements. The adequacy of their quality assurance program is also assessed during the audits.

Intercomparison programs, which involve the comparison of Supply System analytical results for samples containing known concentrations of various radionuclides with the known values and with the results reported by other monitoring programs, are also a major component of the quality assurance activities of the REMP. The program participates in the Environmental Protection Agency (EPA) and Environmental Measurements Laboratory (EML) intercomparison programs. It also participates in local and regional intercomparison studies. The following sections summarize the quality assurance and quality control aspects of the TLD and analytical components of the REMP.

### 6.1 Quality Control For the Supply System Environmental TLD Program

The Quality Control Program includes the preparation, processing and evaluation of environmental TLDs. QA dosimeters, which are annealed just prior to being given a known exposure to cesium-137 gamma radiation and processed among the field dosimeters, serve as indicators that the readout, calibration and evaluation of the field dosimeters were properly performed. The number of QA dosimeters used during each processing is generally 10% of the number of field dosimeters. Since 1987, the same QA dosimeters have been used repeatedly throughout the year in order to track their sensitivity and to provide consistency from run to run.

If the mean QA dosimeter results are greater than  $\pm 5\%$  (or 5 mR) of the given exposure, an investigation into the source of the discrepancy is initiated. Evaluation of the 1994 QA dosimeter results indicated only small biases for the four quarters.

Control dosimeters (trip controls) are used for each set of field dosimeters to monitor the contribution of the exposure received by the field TLDs while in transit. The radiation background in the storage area is also monitored by a separate set of control dosimeters (building controls). If the trip control results are significantly greater than the building control results, the difference between the two is subtracted from the field dosimeters.

Spiked dosimeters, which are exposed to known levels of radiation below the 100 mR given to the QA dosimeters, were processed with the field dosimeters during each run to verify the accuracy of the environmental TLD evaluations. All results were within  $\pm 5\%$  of the given exposure and are provided in Table 6-1.

During each environmental TLD processing, individual calibration factors are determined for each TLD by exposing the TLDs to 100 mR from an encapsulated cesium-137 source. The calibration exposure is determined from the exposure duration and the assessed source strength at 55 centimeters. As a quality control check for each calibration exposure, an NIST-traceable ionization chamber is also used to determine the calibration exposure. If the ionization chamber measurement does not agree with the calibration exposure to within  $\pm 5\%$  of the calculated value, an investigation is performed into the discrepancy and the TLD calibration is repeated, if necessary.

## **6.2 Quality Control For the Analytical Program**

Quality control for the analytical program involves two components: the quality control activities performed by the Supply System and the quality control program of the analytical contractor, Teledyne Brown Engineering. Both of these components are described in the following sections.

### **6.2.1 Supply System Quality Control Activities**

The Supply System has participated in the U.S. Department of Energy's Environmental Measurements Laboratory (EML) Quality Assessment Program since 1987. In general, the Teledyne Brown results agreed with the EML values as seen in Table 6-2.

In addition to participating in the EML Intercomparison Program, the Supply System participates in the Environmental Radiation Quality Assurance Task Force of the Pacific Northwest (QATF) intercomparison program. The results of the 1994 QATF intercomparison are listed in Table 6-3.

Duplicate samples were also submitted to Teledyne Brown for analysis during 1994. These duplicates consisted of two sets of milk samples and one set of air filters from EML. The milk duplicates were marked Station 37 and were submitted for analysis at the same time as the milk samples from Station 36.

### **6.2.2 Teledyne Brown Engineering Quality Control Program**

The goal of the quality control program at Teledyne Brown Engineering is to produce analytical results which are accurate, precise and supported by adequate documentation. The program is based on the requirements of 10CFR50, Appendix B, Regulatory Guide 4.15 and the implemented program, as described in Teledyne's Quality Assurance Manual IWL-0032-395 and Quality Control Manual IWL-0032-365.

All measuring equipment is calibrated for efficiency at least annually using standard reference material traceable to the National Institute of Standards and Technology (NIST). For alpha and beta counting, check sources are prepared and counted every day the counter is in use. Control charts are maintained with three-sigma limits specified. Backgrounds are usually measured at least once per week.

The efficiency of the gamma spectrometers is determined annually with a NIST-traceable standard reference material selected to cover the energy range of the nuclides to be monitored and to include all of the geometries measured. Backgrounds are determined every other week and check sources are counted weekly. The energy resolution and efficiency are plotted at two energy levels (125.1 and 1274.4 KeV) on charts and held within three-sigma control limits.

The efficiency of the tritium liquid scintillations was determined at least annually by counting a NIST-traceable standard which had been diluted in a known amount of distilled water so that the activity level loaded into the counter is sufficient to provide good counting statistics. If the efficiency measurement does not agree within two standard deviations of the average of the previous six measurements, corrective action is taken. The background of each counter is measured for each batch of samples. A control chart is maintained for the background and check source measurements as a stability check.

Control of sample identity is maintained by the assignment of a unique five-digit number which is maintained throughout the sample's history from sample receipt, log-in number assignment, and designation of analysis to reporting the final analytical result. Besides ensuring sample accountability, Teledyne procedures also ensure the maintenance of sample integrity by controlling sample storage and taking steps to prevent contamination of samples.

Results are reviewed for reasonableness before being entered into the data system. Any results which are suspect, i.e., which are higher or lower than results in the past, are returned to the laboratory for recount. If a longer count, decay check, recount on another system or recalculation does not give acceptable results based on experience, a new aliquot is analyzed. The complete information about the sample is contained on the worksheets accompanying the sample results.

Teledyne Brown also participates in the EPA intercomparison program. Table 6-4 presents the results of the 1994 intercomparison as reported to the Supply System...Footnotes in the table refer to investigations of problems encountered in a few cases and the steps taken by Teledyne Brown to prevent reoccurrence.

### **Iodine-131 Cartridges**

One blank charcoal cartridge was analyzed on a weekly basis with each set of cartridges, for a total of fifty-two blanks. The average activity of the blanks was  $-3.2 \pm 8.4 \text{ E-01 total pCi}$ . Activities were calculated without considering detection limits.

### Gross-Beta - Filters

One blank filter was measured with each set of filters analyzed. Fifty-two blanks were counted. The average activity of the blanks in total pCi was  $1.1 \pm 0.2 \text{ E}+00$ .

### I-131 - Milk

A blank milk was analyzed with each group of samples analyzed. The results showed that there was no contamination in the laboratory or counting area. Eighteen blank samples were counted in 1994 with an average activity of  $1.6 \pm 5.5 \text{ E}-02 \text{ pCi/liter}$  without considering detection limits. An additional 52 blanks were analyzed as part of the Teledyne Brown Engineering Quality Assurance program. The average result for blanks in 1994 was  $1.6 \pm 6.2 \text{ E}-02 \text{ pCi/liter}$ .

### Sr-90 - Milk and Water

Blank water samples were analyzed on a weekly basis. The average activity of the fifty-two samples in 1994 was  $5.8 \pm 20.0 \text{ E}-02 \text{ pCi/liter}$ . Fifty-two spiked water samples were analyzed with an average value of  $3.7 \pm 0.2 \text{ E}+01 \text{ pCi/l}$  versus a spike value of  $3.8 \pm 0.6 \text{ E}+01 \text{ pCi/l}$ . A total of twenty-four spiked milk samples were analyzed in 1994. The average value of the samples was  $3.8 \pm 0.3 \text{ E}+01 \text{ pCi/liter}$  compared to a spike value of  $3.8 \pm 0.6 \text{ E}+01 \text{ pCi/l}$ . Twenty-four blank milk samples were analyzed with an average activity of  $1.0 \pm 0.3 \text{ E}+00 \text{ pCi/liter}$  of Sr-90, which is the natural content of milk.

### Gross Beta - Water

Fifty-two blanks were prepared from distilled water. The average result was  $2.9 \pm 1.5 \text{ E}-01$  without considering detection limits. In 1994, fifty-two spike samples were also analyzed with an average result of  $2.1 \pm 0.2 \text{ E}+01 \text{ pCi/liter}$ . The spike level was  $2.2 \pm 0.5 \text{ E}+01 \text{ pCi/l}$ . No contamination was indicated and the background level was low and stable.

### Tritium in Water

Fifty-two blank samples were analyzed by liquid scintillation. The average result, without considering detection levels, was  $-1.1 \pm 35.0 \text{ E}+00 \text{ pCi/l}$ . Fifty-two spiked samples with a level of  $1.3 \pm 0.2 \text{ E}+03 \text{ pCi/l}$  were analyzed. The average result was  $1.4 \pm 0.1 \text{ E}+03 \text{ pCi/l}$ .

### Gamma Spectroscopy

A blank water sample was analyzed on a weekly basis in the gamma spectroscopy laboratory. All nuclides were below the normal level of detection. Weekly spike samples were analyzed using the Cs-137 peak at 662 KeV. The spike value was  $2.2 \pm 0.3 \text{ E}+04 \text{ pCi/l}$  and the average result of the fifty-two spike samples was  $2.2 \pm 0.03 \text{ E}+04 \text{ pCi/l}$ .

TABLE 6-1

1994 ENVIRONMENTAL SPIKED DOSIMETER RESULTS

DISTRIBUTION PERIOD	GIVEN EXPOSURE (mR)	REPORTED EXPOSURE (mR)	BIAS (%)
First Quarter	30.0	30.6	2.0
		30.6	2.0
		30.3	1.0
Second Quarter	25.0	25.4	1.6
		27.2	0.7
		29.6	2.0
Third Quarter	29.0	29.5	1.7
		28.6	-1.4
		28.6	-1.4
Fourth Quarter	26.0	26.4	1.5
		27.2	4.4
		26.5	1.9
Annual	80.0	81.0	1.2
		78.2	-2.3
		84.1	4.9

TABLE 6-2

1994 ENVIRONMENTAL MEASUREMENTS LABORATORY (EML)  
QUALITY ASSESSMENT PROGRAM RESULTS

DATE	SAMPLE TYPE	NUCLIDE	REPORTED RESULT	PERCENT ERROR	EML VALUE	RATIO REPORTED/EML
03/94	Air (Bq/filter) <sup>1</sup>	Mn-54	2.77E+01	2	3.35E+01	0.83
		Co-57	8.61E+00	2	1.25E+01	0.69
		Co-60	5.92E+01	2	7.02E+01	0.84
		Sb-125	2.14E+01	5	2.33E+01	0.92
		Cs-137	3.48E+01	2	4.00E+01	0.87
		Ce-144	8.38E+01	2	1.28E+02	0.65
03/94	Soil (Bq/kg)	K-40	3.54E+02	5	3.37E+02	1.05
		Sr-90	1.20E+01	21	8.79E+00	1.37
		Cs-137	1.61E+02	2	1.41E+02	1.14
03/94	Vegetation (Bq/kg)	K-40	1.01E+03	3	9.23E+02	1.09
		Co-60	4.03E+01	5	3.40E+01	1.19
		Cs-137	5.31E+02	1	4.61E+02	1.15
03/94	Water (Bq/l)	H-3	1.90E+02	3	1.87E+02	1.02
		Mn-54	9.08E+01	3	9.82E+01	0.92
		Co-60	1.01E+02	3	1.01E+02	1.00
		Sr-90	3.30E+01	3	2.86E+01	1.15
		Cs-134	1.49E+02	2	1.54E+02	0.97
		Cs-137	1.00E+02	2	9.37E+01	1.07
09/94	Soil (Bq/kg)	K-40	3.91E+02	10	4.28E+02	0.91
		Sr-90	2.80E+00	29	3.30E+00	0.85
		Cs-137	2.96E+02	10	2.80E+02	1.06
09/94	Vegetation (Bq/kg)	K-40	7.71E+02	10	8.08E+02	0.95
		Sr-90	5.20E+02	7	5.35E+02	0.97
		Co-60	1.22E+01	22	1.07E+01	1.14
09/94	Water (Bq/l)	H-3	1.10E+02	7	1.13E+02	0.97
		Mn-54	9.83E+01	10	1.08E+02	0.91
		Co-60	3.38E+02	10	3.17E+02	1.07
		Sr-90	8.50E+01	4	6.86E+01	1.24
		Cs-137	5.08E+01	10	4.66E+01	1.09

<sup>1</sup> Bq = becquerel; the EML results are reported in becquerels instead of picocuries. One picocurie equals 0.037 becquerel.

TABLE 6-3

1994 ENVIRONMENTAL RADIATION QUALITY ASSURANCE TASK FORCE  
OF THE PACIFIC NORTHWEST INTERCOMPARISON RESULTS

SAMPLE/ NUCLIDE	WA	OR	PNL	WHC	SS	ID	USE
Reference Sediment (pCi/g dry)							
Sr-90	282 ± 4	—	108 ± 1.3	241 ± 2.4	240 ± 10	—	239 ± 4.1
	267 ± 4	—	117 ± 1.3	239 ± 2.4	240 ± 10	220 ± 20	240 ± 3.5
	269 ± 4	—	109 ± 1.3	—	240 ± 10	—	—
Cs-137	85.9 ± 0.3	83.2 ± 1.2	85.1 ± 0.7	87.5 ± 0.5	85.6 ± 8.6	78.7 ± 7.1	103.7 ± 0.2
	84.1 ± 0.3	—	82.5 ± 0.7	87.0 ± 0.5	86.2 ± 8.6	—	100.2 ± 0.3
	85.3 ± 0.3	—	73.1 ± 0.7	—	84.1 ± 8.4	—	—
Hanford Groundwater 100-N-56 (pCi/l)							
H-3	11800 ± 300	10200 ± 200	11100 ± 325	7791 ± 78	11000 ± 1000	11710 ± 340	11951 ± 320
	11300 ± 300	—	11100 ± 326	8287 ± 83	11000 ± 1000	11990 ± 350	11595 ± 316
	11300 ± 300	—	10800 ± 321	8712 ± 87	12000 ± 1000	11800 ± 350	11719 ± 317
Sr-90	232 ± 13	—	329 ± 4.5	212 ± 9.3	230 ± 10	240 ± 20	197.3 ± 4.2
	235 ± 13	—	243 ± 4.6	207 ± 6.8	240 ± 10	—	221.4 ± 4.8
	242 ± 14	—	243 ± 4.4	204 ± 6.3	230 ± 10	—	199.7 ± 4.5
Co-60	2.41 ± 0.87	12.8 ± 0.4	-3.25 ± 8.61	5.9 ± 1.9	<1.9 ± 1.3	2.3 ± 0.96	2.6 ± 0.8
	2.36 ± 1.16	13.0 ± 0.4	-5.57 ± 9.10	5.5 ± 2.3	<1.3 ± 1.8	1.2 ± 0.94	2.9 ± 0.9
	1.66 ± 0.81	—	1.24 ± 7.57	7.7 ± 2.5	<2.0 ± 2.7	1.3 ± 84.0	2.7 ± 1.2

WA = Washington State Department of Health  
 PNL = Battelle Pacific Northwest Laboratory  
 WHC = Westinghouse Hanford Company  
 USE = US Ecology

OR = State of Oregon Health Division  
 SS = Washington Public Power Supply System  
 ID = Idaho State University



TABLE 6-3 (Cont.)

1994 ENVIRONMENTAL RADIATION QUALITY ASSURANCE TASK FORCE  
OF THE PACIFIC NORTHWEST INTERCOMPARISON RESULTS

SAMPLE/ NUCLIDE	WA	OR	PNL	WHC	SS	ID	USE
Hanford Groundwater 399-1-16A (pCi/l)							
H-3	10600 ± 200	10200 ± 200	10300 ± 313	9878 ± 99	10000 ± 1000	10790 ± 340	10588 ± 303
	10700 ± 200	—	10100 ± 312	8861 ± 89	10000 ± 1000	10770 ± 340	10772 ± 305
	10400 ± 200	—	10200 ± 313	10732 ± 107	10000 ± 1000	11210 ± 340	10815 ± 306
U-234	58.7 ± 1.4	—	68.2 ± 1.1	—	67 ± 8	62 ± 14	66.9 ± 0.8
	58.1 ± 1.3	—	76.5 ± 1.5	—	67 ± 6	—	65.3 ± 0.8
	58.7 ± 1.3	—	75.5 ± 1.2	—	66 ± 8	—	66.9 ± 0.8
U-235	2.9 ± 0.3	—	4.21 ± 0.49	—	2.7 ± 0.9	2.7 ± 1.8	3.5 ± 0.2
	3.3 ± 0.3	—	1.53 ± 0.26	—	3.0 ± 0.8	—	3.7 ± 0.2
	3.3 ± 0.3	—	1.69 ± 0.17	—	2.6 ± 0.9	—	3.6 ± 0.2
U-238	49.1 ± 1.3	—	56.9 ± 1.0	—	52 ± 6	53 ± 12	54.9 ± 0.7
	46.3 ± 1.2	—	57.6 ± 1.3	—	54 ± 5	—	53.1 ± 0.7
	58.7 ± 1.3	—	58.8 ± 1.0	—	51 ± 6	—	55.1 ± 0.7
Total U	114 ± 12	—	93.1 ± 27.2	178 ± 8.9	122 ± 10	120 ± 40	125.3 ± 1.7
	111 ± 2	—	92.3 ± 26.9	177 ± 8.9	124 ± 7.9	—	122.1 ± 1.7
	115 ± 2	—	97.8 ± 28.6	177 ± 8.9	120 ± 10	—	125.6 ± 1.7

WA = Washington State Department of Health  
 PNL = Battelle Pacific Northwest Laboratory  
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 USE = US Ecology

OR = State of Oregon Health Division  
 SS = Washington Public Power Supply System  
 ID = Idaho State University

1994 REMP ANNUAL REPORT

TABLE 6-4

## 1994 EPA INTERCOMPARISON PROGRAM RESULTS

ISOTOPE	COLLECTION DATE	TI RESULTS (a)		EPA RESULTS (b)		OTHER LABS (c)	
<u>MEDIUM - WATER (pCi/liter)</u>							
Sr-89	01/14/94	24.00 ±	1.00	25.0 ±	5.0	23.74 ±	4.01
Sr-90	"	15.67 ±	1.53	15.0 ±	5.0	14.59 ±	2.17
Sr-89	04/19/94	19.00 ±	1.00	20.0 ±	5.0	18.49 ±	3.69
Sr-90	"	13.00 ±	0.00	14.0 ±	5.0	14.13 ±	2.14
Sr-89	07/15/94	26.00 ±	1.73	30.0 ±	5.0	28.84 ±	6.06
Sr-90	"	19.00 ±	0.00	20.0 ±	5.0	18.80 ±	2.80
Gr-Beta	01/28/94	72.33 ±	3.79	62.0 ±	10.0	56.14 ±	14.15 (d)
Gr-Beta	04/19/94	102.67 ±	6.43	117.0 ±	18.0	106.86 ±	15.47
Gr-Beta	07/22/94	16.00 ±	0.00	10.0 ±	5.0	14.91 ±	3.74 (e)
Gr-Beta	10/28/94	25.33 ±	1.53	23.0 ±	5.0	27.16 ±	5.23
I-131	02/04/94	110.33 ±	0.00	119.0 ±	12.0	120.99 ±	10.18
Ra-226	02/11/94	21.00 ±	1.00	19.9 ±	3.0	19.39 ±	3.51
Ra-228	"	15.67 ±	1.53	14.7 ±	3.7	14.09 ±	3.07
Co-60	04/19/94	23.67 ±	3.21	20.0 ±	5.0	20.12 ±	1.83
Cs-134	"	34.00 ±	1.73	34.0 ±	5.0	31.45 ±	2.72
Cs-137	"	34.00 ±	2.65	29.0 ±	5.0	31.17 ±	2.40
Ra-226	"	15.67 ±	1.53	20.0 ±	3.0	19.72 ±	4.23 (f)
Ra-228	"	15.33 ±	0.58	20.1 ±	5.0	20.07 ±	3.58
Co-60	06/10/94	43.00 ±	2.00	50.0 ±	5.0	49.77 ±	3.82 (g)
Zn-65	"	13.33 ±	0.58	134.0 ±	13.0	140.62 ±	9.59 (h)
Ru-106	"	201.33 ±	9.29	252.0 ±	25.0	216.56 ±	28.52 (i)
Cs-134	"	29.33 ±	3.79	40.0 ±	5.0	36.99 ±	3.14 (j)
Cs-137	"	49.67 ±	1.53	49.0 ±	5.0	52.38 ±	3.61
Ba-133	"	85.00 ±	3.00	98.0 ±	10.0	86.46 ±	8.31 (k)
Ra-226	06/17/94	15.33 ±	0.58	15.0 ±	2.3	14.95 ±	1.49
Ra-228	"	16.33 ±	1.53	15.4 ±	3.9	15.39 ±	2.84
Ra-226	09/16/94	10.67 ±	0.58	10.0 ±	1.5	9.92 ±	1.31
Ra-228	"	9.70 ±	0.52	10.2 ±	2.6	10.47 ±	1.87
I-131	10/07/94	71.00 ±	3.00	79.0 ±	8.0	79.89 ±	6.79
Co-60	11/04/94	52.00 ±	0.00	59.0 ±	5.0	58.87 ±	4.59 (l)
Zn-65	"	81.33 ±	7.02	100.0 ±	10.0	104.68 ±	7.95 (l)
Cs-134	"	19.67 ±	2.52	24.0 ±	5.0	22.95 ±	2.27
Cs-137	"	54.33 ±	2.31	49.0 ±	5.0	51.92 ±	3.61
Ba-133	"	58.33 ±	2.89	73.0 ±	7.0	70.81 ±	6.13 (l)
H-3	03/04/94	4833.33 ±152.75		4936.0 ± 494.0		4844.97 ± 477.67	
H-3	08/05/94	9700.00 ±100.04		9951.0 ± 995.0		9651.86 ± 696.62	
<u>MEDIUM - AIR FILTERS (pCi/F)</u>							
Gr-Beta	08/26/94	59.33 ±	3.21	56.0 ±	10.0	59.08 ±	7.23
Sr-90	"	18.00 ±	1.00	20.0 ±	5.0	19.57 ±	2.76
Cs-137	"	17.00 ±	1.73	15.0 ±	5.0	16.59 ±	2.42
<u>MEDIUM - MILK (pCi/liter)</u>							
Sr-89	09/30/94	24.33 ±	2.52	25.0 ±	5.0	22.19 ±	5.11
Sr-90	"	17.67 ±	1.53	15.00 ±	5.0	15.15 ±	2.48
I-131	"	81.67 ±	5.86	75.0 ±	8.0	74.89 ±	5.58
Cs-137	"	70.33 ±	4.62	59.0 ±	5.0	62.39 ±	3.72 (m)
K	"	1740.00 ±	153.95	1715.0 ±	86.0	1700.90 ±	109.00

TABLE 6-4 (Cont.)

Footnotes:

- (a) Teledyne Results - Average  $\pm$  one sigma. Units are pCi/liter for water and milk except for potassium (K) which is in mg/liter. Units are total pCi for air particulate filters.
- (b) EPA Results - Expected laboratory precision (1  $\sigma$ ). Units are pCi/l for water, and milk except that potassium is in mg/l. Units are total pCi for air particulate filters.
- (c) Average concentration plus or minus one sigma, based on range of values encountered.
- (d) There appears to be variation in self-absorption matrix. The EPA confirms that the composition of their tap water from Lake Mead, varies seasonally which can cause variation in alpha, beta results. Results are within  $\pm 3 \sigma$  control limits.
- (e) EPA results for gross beta in water were corrected for 20% crosstalk into the beta channel from the Th-230 alpha spike. Measurements show that the crosstalk can be much higher (37% for Tennelec Counter #3 and 54% for Gamma Products Counter #1). The normalized deviation from the grand average was only 0.38.
- (f) No specific or apparent reason found. Data sheets verified and detector efficiencies calibrated. Teledyne will check future samples to see if a pattern develops.
- (g) A second aliquot was analyzed, paying particular attention to volume aliquoted. The result, 52 pCi/l, was in good agreement with the EPA. The three original results, each counted on a different detector, showed good precision. The measurement of Co-60 has not been a problem.
- (h) The average value of three analyses on the "Report of Analysis" was 133 pCi/l which is in good agreement with the EPA. Apparently, incorrect results were entered into the EPA computer.
- (i) EPA was experiencing problems with the Ruthenium-106 used in Performance Evaluation Studies.
- (j) The first aliquot, prepared according to EPA dilution instructions was counted on four detectors in the 1-liter Marinelli geometry with Cs-134 results (based on the 796-KeV peak) in pCi/l of 32.0, 25.1, 31.7, and 30.8. The 31.7 result was not reported. Had that been reported instead of 25.1, the average would have been 31.5 and the normalized deviation would have been -2.94 instead of -3.70. A second aliquot was prepared and a single measurement was made with the result of 31.1 pCi/l. An undiluted aliquot was measured in a 150-ml geometry with the result of 33.5 pCi/l. That result is comparable with the Marinelli results. Thus, sample preparation (dilution, volume determination, maintaining correct pH, etc.), sample geometry, or detector efficiency do not seem to be the cause of the low results.
- (k) There is no apparent reason for the low result, however the average value, 85 pCi/l is in good agreement to the grand average (86.46).
- (l) The results are being investigated.
- (m) The milk sample was counted four times. The reported Cs-137 values were based on one aliquot of 1 liter volume and an aliquot of 0.865 liter counted two times. It is suspected that the 0.865 liter volume was incorrectly determined. If 1 liter (the usual volume for counting milk samples) is used in the calculation, the average of three results equals 63.6 pCi/l which gives a normalized deviation to the known of 1.59. The fourth count (a 1-liter aliquot) had a Cs-137 equal to 64.2 pCi/l which is in good agreement with the average of the other three.

## 7.0 REFERENCES

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5. U.S. Nuclear Regulatory Commission, "Performance, Testing and Procedural Specifications For Thermoluminescence Dosimetry-Environmental Applications", Regulatory Guide 4.13, Revision 1, July 1977.
6. Energy Facility Site Evaluation Council, Resolution No. 260, January 1992.
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8. WNP-2 Offsite Dose Calculation Manual (ODCM).
9. Code of Federal Regulations, Title 10 Part 20, "Standards For Protection Against Radiation".
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11. Washington Administrative Code 246-290, "Group A Public Water Systems".
12. Washington Administrative Code 173-200, "Water Quality Standards for Ground Water of the State of Washington".
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14. "Teledyne Isotopes TLD System For Personnel and Environmental Monitoring", Teledyne Isotopes, Westwood, New Jersey, February 2, 1981.
15. Washington Public Power Supply System, Radiological Environmental Monitoring Program Annual Report Plant 2, 1985, April 1986.
16. Robertson, D. E., and J. J. Fix, Association of Hanford Origin Radionuclides With Columbia River Sediment, BNWL-2305, August 1977.
17. Energy Facility Site Evaluation Council, Resolution 259, amended November 1994.

## 8.0 1993 REMP REPORT ERRATA

Corrections to errors found in the 1993 Radiological Environmental Monitoring Program Annual Report are listed below. Corrections to 1993 annual TLD results listed in Tables 5-5 and 5-6 are included in Tables 8-1 and 8-2.

Page 4-16, Table 4-2: SE sector, Station 40 sample types should be AP/AI/TLD.

Page 4-16, Table 4-2: SW sector, Station 96 sample type should be MI, not TLD.

Page 5-3, Figure 5-5: Caption should be TLD Frequency Distribution 1984-1992.

Page 5-13, Table 5-1: Storm Drain Water - Previous Operational Cs-134 range should be (-4.8 - 4.8).

Page 5-13, Table 5-1: Storm Drain Water - Previous Operational H-3 mean should be 12486.9 pCi/l.

Page 5-13, Table 5-1: Storm Drain Water - 1993 H-3 mean should be 5650.5 pCi/l.

Page 5-13, Table 5-1: Sanitary Waste Water - Previous Operational H-3 average should be 736.5 pCi/l.

Page 5-13, Table 5-1: Sanitary Waste Water - 1993 H-3 mean should be 317.2 pCi/l.

Page 5-14, Table 5-1: Storm Drain Sediment - Previous Operational Fe-59 mean and range should be 5.7 (-55.0 - 280.0) pCi/kg.

Page 5-14, Table 5-1: Sanitary Waste Sediment - Previous Operational Cs-137 mean should be 128.2 pCi/kg.

Page 5-14, Table 5-1: Sanitary Waste Sediment - Previous Operational Co-60 mean should be 170.7 pCi/kg.

Page 5-14, Table 5-1: Sanitary Waste Sediment - Previous Operational Zn-65 range should be (-65.0 - 69.0) pCi/kg.

Page 5-14, Table 5-1: Sanitary Waste Sediment - Previous Operational Mn-54 mean and range should be 6.2 (-26.0 - 32.1) pCi/kg.

Table A-2.1, Gross Beta on Air Particulate Filters: The exponent for the overall uncertainty starting 930927 to 931004 and ending at 931220 to 931227 for all stations should be E-03, not E-01.

Page 5-3, Figure 5-3: Chart incorrect, correct chart shown.

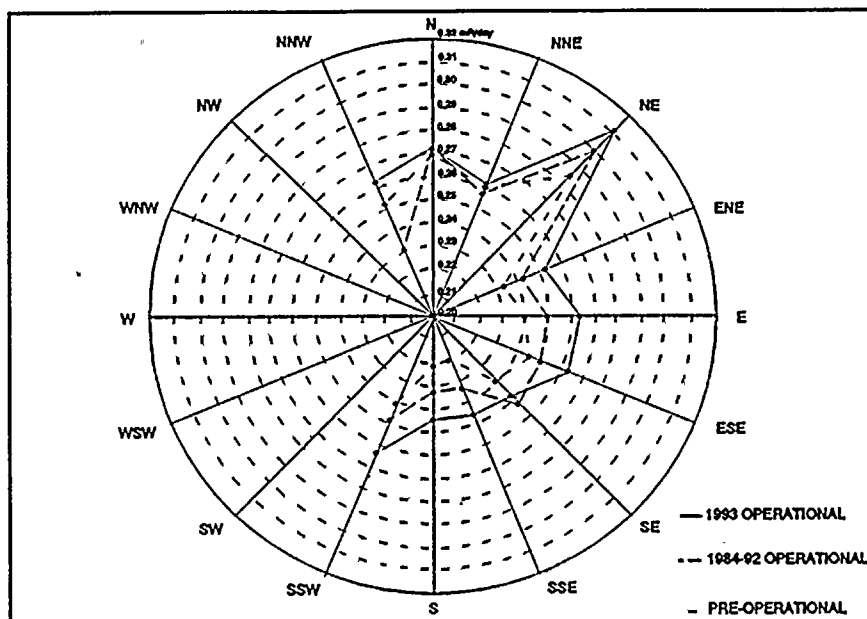


Figure 5-3 Remote TLDs

Page 5-6, Figure 5-12: Chart incorrect, correct chart shown.

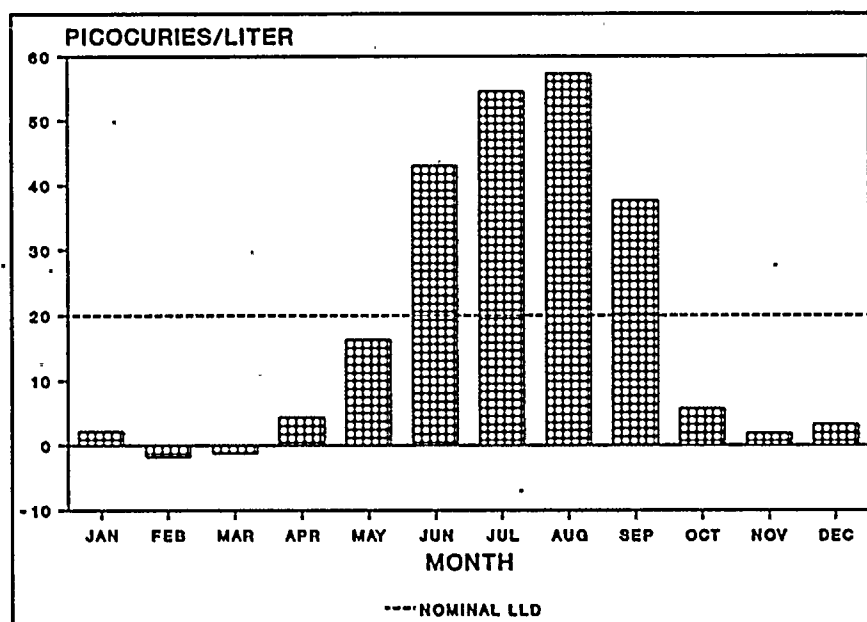


Figure 5-12 Zn-65 in Discharge Water - 1993

TABLE 8-1

**ANNUAL TLD DATA SUMMARY FOR THE PREOPERATIONAL  
AND OPERATIONAL PERIODS - CORRECTIONS FOR 1993 ANNUAL REPORT**

Results in mR/day

STATION	PREOPERATIONAL		1984 - 1992 OPERATIONAL		1993 OPERATIONAL	
	MEAN(a)	STANDARD ERROR	MEAN	STANDARD ERROR	MEAN	STANDARD DEVIATION(2σ)
1	0.23	0.04	0.23	0.02	0.25	0.02
2	0.22	0.04	0.23	0.02	0.24	0.01
3	0.21	0.04	0.22	0.02	0.23	0.01
4	0.22	0.06	0.20	0.02	0.22	0.01
5	0.22	0.04	0.21	0.02	0.22	0.01
6	0.21	0.04	0.21	0.02	0.22	0.01
7	0.22	0.02	0.23(b)	0.01	0.24	0.01
8	0.25	0.04	0.25(b)	0.02	0.26	0.01
9	0.20	0.02	0.21	0.02	0.22	0.00
10	0.22	0.04	0.22	0.02	0.23	0.01
11	0.22	0.04	0.22	0.02	0.24	0.01
12	0.24	0.04	0.24	0.02	0.25	0.01
13	0.22	0.04	0.23	0.01	0.24	0.01
14	0.22	0.04	0.22	0.02	0.24	0.01
15	0.23	0.06	0.24	0.02	0.26	0.01
16	0.23	0.04	0.23	0.02	0.24	0.01
17	0.23	0.02	0.23(b)	0.02	0.25	0.01
18	0.25	0.02	0.23	0.02	0.24	0.01
19	0.22	0.04	0.23	0.01	0.22	0.00
20	0.23	0.04	0.23	0.02	0.24	0.01
21	0.23	0.02	0.21	0.02	0.23	0.01
22	0.22	0.02	0.23	0.02	0.24	0.01
23	0.24	0.02	0.22	0.02	0.23	0.01
24	0.22	0.02	0.23	0.02	0.24	0.01
25	0.24	0.02	0.24	0.02	0.26	0.00
40	0.21(c)	0.02	0.22	0.02	0.21	0.00
41	0.26(c)	0.04	0.24	0.02	0.25	0.01
42	0.24(c)	0.02	0.23	0.02	0.24	0.00
43	0.24(c)	0.02	0.24	0.02	0.26	0.00
44	0.24	0.02	0.22	0.02	0.24	0.01
45	0.24	0.01	0.22	0.02	0.24	0.01
46	0.29	0.01	0.28	0.02	0.29	0.01
47	0.22(c)	0.03	0.21	0.02	0.24	0.01
49	(d)		0.22	0.01	0.24	0.01

(a) This preoperational mean is for 1982 - 1983 data only.

(b) 1985 TLD missing.

(c) There was only one annual exchange during the preoperational period.

(d) Stations 49-56 were first monitored during Fourth Quarter 1983. Station 61 was added in 1989 and discontinued in 1992.

TABLE 8-1 (Cont.)

ANNUAL TLD DATA SUMMARY FOR THE PREOPERATIONAL  
AND OPERATIONAL PERIODS - CORRECTIONS FOR 1993 ANNUAL REPORT  
Results in mR/day

STATION	<u>PREOPERATIONAL</u>		<u>1984 - 1992 OPERATIONAL</u>		<u>1993 OPERATIONAL</u>	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN	STANDARD ERROR	MEAN	STANDARD DEVIATION(2 $\sigma$ )
50	(d)		0.23	0.02	<u>0.24</u>	<u>0.01</u>
51	(d)		0.22	0.02	<u>0.24</u>	0.01
53	(d)		0.25	0.02	<u>0.25</u>	0.01
54	(d)		0.24	0.02	<u>0.25</u>	0.01
55	(d)		0.22	0.02	<u>0.24</u>	0.01
56	(d)		0.23	0.02	0.24	0.01
61	(d)		0.27 <sup>(d)</sup>	0.02	(e)	
71	0.24 <sup>(a)</sup>	0.02	0.25	0.02	<u>0.28</u>	<u>0.00</u>
72	0.25 <sup>(a)</sup>	0.02	0.25	0.02	0.27	<u>0.00</u>
73	0.23 <sup>(a)</sup>	0.01	0.22	0.02	0.23	0.01
74	0.24 <sup>(a)</sup>	0.01	0.24	0.02	0.26	0.01
75	0.24 <sup>(a)</sup>	0.01	0.23	0.02	<u>0.25</u>	<u>0.00</u>
76	0.24 <sup>(a)</sup>	0.02	0.23	0.02	<u>0.25</u>	0.01
77	0.25 <sup>(a)</sup>	0.02	0.23	0.02	<u>0.25</u>	<u>0.00</u>
78	0.25 <sup>(a)</sup>	0.04	0.23	0.02	<u>0.24</u>	0.00
79	0.25 <sup>(a)</sup>	0.02	0.23	0.02	0.24	0.01
80	0.23 <sup>(a)</sup>	0.05	0.22	0.02	<u>0.24</u>	0.01
81	0.23 <sup>(a)</sup>	0.02	0.23	0.02	0.24	<u>0.01</u>
82	0.25 <sup>(a)</sup>	0.03	0.24	0.01	<u>0.25</u>	0.01
83	0.25 <sup>(a)</sup>	0.02	0.24	0.02	<u>0.26</u>	<u>0.02</u>
84	0.23 <sup>(a)</sup>	0.02	0.23	0.02	0.25	0.01
85	0.26 <sup>(a)</sup>	0.02	0.24	0.02	0.26	<u>0.00</u>
86	0.24	0.02	0.26	0.02	<u>0.28</u>	0.01
All	0.23	0.02	0.23	0.01	0.24	0.01

(a) This preoperational mean is for 1982 - 1983 data only.

(b) 1985 TLD missing.

(c) There was only one annual exchange during the preoperational period.

(d) Stations 49-56 were first monitored during Fourth Quarter 1983. Station 61 was added in 1989.

(e) Station 61 discontinued on June 29, 1992



TABLE 8-2

1993 MEAN QUARTERLY VERSUS ANNUAL TLD DATA -  
CORRECTIONS FOR 1993 ANNUAL REPORT  
 Results in mR/day

STATION	<u>QUARTERLY TLDs</u>		<u>ANNUAL TLDs</u>	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN <sup>(b)</sup>	STANDARD DEVIATION(2 $\sigma$ )
1	0.26	0.01	<u>0.25</u>	<u>0.02</u>
2	0.26	0.01	0.24	0.01
3	0.24	0.01	0.23	0.01
4	0.24	0.01	0.22	0.01
5	0.24	0.01	0.22	<u>0.01</u>
6	0.24	0.01	<u>0.22</u>	0.01
7	0.25	0.01	0.24	0.01
8	0.28	0.01	0.26	0.01
9	0.24	0.01	0.22	<u>0.00</u>
10	0.25	0.01	0.23	0.01
11	0.25	0.00	0.24	0.01
12	0.27	0.01	<u>0.25</u>	0.01
13	0.26	0.01	0.24	0.01
14	0.25	0.00	<u>0.24</u>	0.01
15	0.27	0.01	0.26	0.01
16	0.26	0.01	<u>0.24</u>	0.01
17	0.26	0.01	0.25	0.01
18	0.26	0.01	0.24	0.01
19	0.25	0.01	<u>0.22</u>	<u>0.00</u>
20	0.25	0.01	<u>0.24</u>	0.01
21	0.23	0.01	<u>0.23</u>	0.01
22	0.25	0.01	0.24	0.01
23	0.25	0.01	0.23	0.01
24	0.26	0.01	0.24	0.01
25	0.27	0.01	<u>0.26</u>	<u>0.00</u>
40	0.23	0.01	0.21	0.00
41	0.27	0.01	0.25	0.01
42	0.26	0.01	<u>0.24</u>	0.00
43	0.27	0.01	<u>0.26</u>	<u>0.00</u>
44	0.25	0.01	<u>0.24</u>	0.01
45	0.25	0.01	<u>0.24</u>	0.01
46	0.31	0.01	<u>0.29</u>	<u>0.01</u>
47	0.24	0.01	<u>0.24</u>	0.01
49	0.26	0.01	0.24	0.01
50	0.25	0.01	<u>0.24</u>	<u>0.01</u>

(a) Mean of the quarterly results.

(b) Mean of four readout areas on each TLD.

TABLE 8-2 (Cont.)

1993 MEAN QUARTERLY VERSUS ANNUAL TLD DATA -  
CORRECTIONS FOR 1993 ANNUAL REPORT  
 Results in mR/day

STATION	<u>QUARTERLY TLDS</u>		<u>ANNUAL TLDS</u>	
	MEAN <sup>(a)</sup>	STANDARD ERROR	MEAN <sup>(b)</sup>	STANDARD DEVIATION(2 $\sigma$ )
51	0.25	0.01	<u>0.24</u>	0.01
53	0.29	0.02	<u>0.25</u>	0.01
54	0.26	0.01	<u>0.25</u>	0.01
55	0.25	0.01	<u>0.24</u>	0.01
56	0.26	0.01	0.24	0.01
61	(c)		(c)	
71 (1S)	0.29	0.02	<u>0.28</u>	<u>0.00</u>
72 (2S)	0.28	0.01	0.27	<u>0.00</u>
73 (3S)	0.25	0.01	0.23	0.01
74 (4S)	0.28	0.01	0.26	0.01
75 (5S)	0.26	0.02	<u>0.25</u>	<u>0.00</u>
76 (6S)	0.25	0.01	<u>0.25</u>	0.01
77 (7S)	0.26	0.01	<u>0.25</u>	<u>0.00</u>
78 (8S)	0.25	0.01	<u>0.24</u>	0.00
79 (9S)	0.26	0.01	0.24	0.01
80 (10S)	0.25	0.01	<u>0.24</u>	0.01
81 (11S)	0.25	0.01	0.24	<u>0.01</u>
82 (12S)	0.26	0.01	<u>0.25</u>	0.01
83 (13S)	0.27	0.01	<u>0.26</u>	<u>0.02</u>
84 (14S)	0.26	0.01	0.25	0.01
85 (15S)	0.27	0.01	0.26	<u>0.00</u>
86 (16S)	0.29	0.02	<u>0.28</u>	0.01

(a) Mean of the quarterly mean results.

(b) Mean of four readout areas on each TLD.

(c) Station 61 was discontinued on June 29, 1992.