UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 1, 2018

Mr. James J. Hutto
Regulatory Affairs Director
Southern Nuclear Operating Company, Inc.
P.O. Box 1295 / Bin – 038
Birmingham, AL 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS TO CORRECT NON-CONSERVATIVE TECHNICAL SPECIFICATION 3.7.1 (CAC NOS. MF9465, MF9466; EPID NO. L-2017-LLA-0181)

Dear Mr. Hutto:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 218 to Renewed Facility Operating License No. NPF-2 and Amendment No. 215 to Renewed Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant (FNP), Units 1 and 2, respectively. The amendments are in response to your application dated March 22, 2017.

The amendments revise Technical Specification (TS) 3.7.1, "Main Steam Safety Valves (MSSVs)". The change involves revision to Condition A and Condition B of TS 3.7.1 to replace the existing values for moderator temperature coefficient. These values were determined to be non-conservative and inconsistent with the plant licensing basis and TS Bases for FNP.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Shawn Williams

Shawn A. Williams, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:
1. Amendment No. 218 to NPF-2
2. Amendment No. 215 to NPF-8
3. Safety Evaluation

cc w/enclosures: Distribution via Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 218
Renewed License No. NPF-2

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Joseph M. Farley Nuclear Plant, Unit 1, (the facility), Renewed Facility Operating License No. NPF-2, filed by Southern Nuclear Operating Company, Inc. (the licensee), dated March 22, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-2 is hereby amended to read as follows:

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 218, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: March 1, 2018

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 215
Renewed License No. NPF-8

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Joseph M. Farley Nuclear Plant, Unit 2, (the facility), Renewed Facility Operating License No. NPF-8, filed by Southern Nuclear Operating Company, Inc. (the licensee), dated March 22, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment, and paragraphs 2.C.(2) of Renewed Facility Operating License No. NPF-8 is hereby amended to read as follows:

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 215, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License
and Technical Specifications

Date of Issuance: March 1, 2018

ATTACHMENT TO JOSEPH M. FARLEY NUCLEAR PLANTS

UNITS 1 AND 2

LICENSE AMENDMENT NO. 218

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

AND LICENSE AMENDMENT NO. 215

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Renewed Facility Operating Licenses and Appendix "A" Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License	License
NPF-2, page 4	NPF-2, page 4
NPF-8, page 3	NPF-8, page 3
TSs	TSs
3.7.1-1	3.7.1-1

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 218, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission.

a. Southern Nuclear shall not operate the reactor in Operational Modes 1 and 2 with less than three reactor coolant pumps in operation.

b. Deleted per Amendment 13

c. Deleted per Amendment 2

d. Deleted per Amendment 2

e. Deleted per Amendment 152

Deleted per Amendment 2

f. Deleted per Amendment 158

g. Southern Nuclear shall maintain a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

1) Identification of a sampling schedule for the critical parameters and control points for these parameters;

2) Identification of the procedures used to quantify parameters that are critical to control points;

3) Identification of process sampling points;

4) A procedure for the recording and management of data;

5) Procedures defining corrective actions for off control point chemistry conditions; and

(2) Alabama Power Company, pursuant to Section 103 of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location in Houston County, Alabama in accordance with the procedures and limitations set forth in this renewed license.

(3) Southern Nuclear, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(4) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 2775 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 215 are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

(3) Deleted per Amendment 144
(4) Deleted per Amendment 149
(5) Deleted per Amendment 144

3.7 PLANT SYSTEMS

3.7.1 Main Steam Safety Valves (MSSVs)

LCO 3.7.1 Five MSSVs per steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

----------NOTE----------
Separate Condition entry is allowed for each MSSV.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more steam generators with one MSSV inoperable and cycle burnup ≥ 14,000 MWD/MTU.	A.1 Reduce THERMAL POWER to ≤ 87% RTP.	4 hours
B. One or more steam generators with two or more MSSVs inoperable. OR One or more steam generators with one MSSV inoperable and cycle burnup <14,000 MWD/MTU.	B.1 Reduce THERMAL POWER to less than or equal to the Maximum Allowable % RTP specified in Table 3.7.1-1 for the number of OPERABLE MSSVs. AND	4 hours

(continued)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 218 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-2

AND

AMENDMENT NO. 215 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-8

SOUTHERN NUCLEAR OPERATING COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By application dated March 22, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17081A484), Southern Nuclear Operating Company, Inc., (SNC, the licensee) submitted a request to change the Joseph M. Farley Nuclear Plant (FNP or Farley), Units 1 and 2, Technical Specifications (TSs).

Specifically, TS 3.7.1, "Main Steam Safety Valves (MSSVs)" Condition A and Condition B are revised to eliminate the moderator temperature coefficient (MTC) values and replaced with Cycle Burnup values. The MTC values were determined to be non-conservative and inconsistent with the FNP Loss of Load/Turbine Trip (LOL/TT) safety analysis and TS Bases.

2.0 REGULATORY EVALUATION

2.1 Description of the System

Saturated steam generated in the three steam generators flows out through the containment wall in three 32-inch main steam lines to a common header. The MSSVs are located on the main steam lines outside the containment and upstream of the main steam isolation valves. MSSVs provide overpressure protection for the three steam generators during design basis transients occurring at 102 percent (%) Rated Thermal Power (RTP). Each of the three main steam lines is equipped with five spring-loaded MSSVs. To prevent chattering during operation of the MSSVs, each of the five MSSVs on a steam line is set at a different set pressure. The design basis for the MSSVs is that they must have sufficient capacity so that main steam pressure does not exceed 110% of the steam generator shell-side design pressure. Based on this requirement, the valves are sized to relieve 105% of the maximum calculated steam flow at an accumulation pressure not exceeding 110% of the steam generator shell design pressure. The MSSVs discharge to the atmosphere.

2.2 Proposed Changes

The MSSV TSs at FNP allow continued plant operation for a limited amount of time in certain situations when the MSSV LCO 3.7.1, "Five MSSVs per steam generator shall be OPERABLE," is not met. The situations during which continued operation is allowed are called out in CONDITION A, CONDTION B, and CONDTION C of the ACTIONS table. The licensee has proposed changes to the descriptions of CONDITION A and B to amend a non-conservative TS.

Current TS 3.7.1 Condition A states:

> One or more steam generators with one MSSV inoperable and the Moderator Temperature Coefficient (MTC) zero or negative at all power levels.

Revised TS 3.7.1 Condition A would state:

> One or more steam generators with one MSSV inoperable and cycle burnup $\geq$ 14,000 MWD/MTU. [megawatt days per metric ton of uranium]

Current TS 3.7.1 Condition B states:

> One or more steam generators with two or more MSSVs inoperable.
>
> OR
>
> One or more steam generators with one MSSV inoperable and the MTC positive at any power level.

Revised TS 3.7.1 Condition B would state:

> One or more steam generators with two or more MSSVs inoperable.
>
> OR
>
> One or more steam generators with one MSSV inoperable and cycle burnup < 14,000 MWD/MTU.

In addition, TS Bases wording was identified as requiring clarification, and was provided for information only. The term "core average burnup" was replaced with "cycle burnup" in all instances, which is consistent with the safety analysis calculations.

2.3 Regulatory Evaluation

The categories of items required to be in the TSs are provided in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36(c). As required by 10 CFR 50.36(c)(2)(i), the TSs will include LCOs, which are the lowest functional capability or performance levels of equipment required for safe operation of the facility. Per 10 CFR 50.36(c)(2)(i), when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TSs until the condition can be met. The regulation at 10 CFR 50.36(c)(3) requires TSs to include items in the category of SRs, which are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is

maintained, that facility operation will be within safety limits, and that the LCOs will be met. Also, 10 CFR 50.36(a)(1) states that a summary statement of the bases or reasons for such specifications, other than those covering administrative controls, shall also be included in the application, but shall not become part of the TSs.

3.0 TECHNICAL EVALUATION

3.1 Licensee Proposal

The licensee stated that use of MTC values in TS 3.7.1 CONDITION A and B are not consistent with the associated LOL/TT safety analysis and provided the following justification for the proposed changes:

> As reflected in LOL/TT safety analysis, TS 3.7.1 Bases states, in part, "The middle-of-life analysis assumes a -10 pcm/°F MTC and demonstrates that the maximum allowable power level associated with one inoperable MSSV per steam generator can be relaxed to 87% RTP when core average burnup is ≥ 14,000 MWD/MTU." A power reduction to 87%, rather than 60%, when MTC is less than zero but greater than -10 pcm/°F is not supported by the TS Bases and LOL/TT safety analysis calculations. This band, with MTC between zero and -10 pcm/°F, correlates to a cycle burnup of < 14,000 MWD/MTU, and should require a power reduction to 60% rather than allowing power to be reduced to 87%. Specifically, the current TS would result in a non-conservative condition post-accident when the plant was operating prior to middle-of life (cycle burnup <14,000 MWD/MTU) where MTC would be less negative than the analyzed -10 pcm/°F value that corresponds to the relaxed 87% RTP.
>
> This discrepancy was entered into the corrective action program, and a past operability review was conducted by reviewing narrative logs for the past three (3) years. During that timeframe, TS 3.7.1 Condition A was only entered once when burnup records indicated 21,092.66 MWD/MTU, and the MTC value was more negative than -10 pcm/°F. The burnup at which the MTC becomes more negative than -10 pcm/°F is confirmed to occur each cycle prior to 14,000 MWD/MTU cycle burnup per the Westinghouse reload safety analysis checklist. A standing order was issued by Operations to compensate for the currently written, non-conservative TS Condition A. The standing order requires operators to reduce power to 60%, when cycle burnup is < 14,000 MWD/MTU to ensure the actions are consistent with the current TS Bases and LOL/TT safety analysis.

The licensee stated that the non-conservatism in the current TS 3.7.1 was introduced during the conversion of FNP TSs.

3.2 NRC Staff Evaluation

The NRC staff reviewed the licensee's proposed changes, justification for the changes, FNP TSs prior to the TS conversion that were contained in Amendment No. 121, Unit 1, and No. 113, Unit 2, dated September 3, 1996 (ADAMS Accession No. ML013120487) and TS conversion to the Improved Standard Technical Specifications (ISTS) Conversion contained in Amendment No. 146, Unit 1, and No. 137, Unit 2, dated November 30, 1999 (ADAMS Accession Package No. ML993500022).

As described in the FNP Updated Final Safety Analysis Report (UFSAR), (ADAMS Package Accession No. ML17117A380) Section 10.3.7, five MSSVs are located on each main steam header, outside containment, upstream of the main steam isolation valves. Its purpose is to limit the secondary system pressure to ≤110% of design pressure for any anticipated operational occurrence (AOO) or accident considered in the Design Basis Accident and transient analysis. The events that challenge the relieving capacity of the MSSVs, and thus reactor coolant system (RCS) pressure, are those characterized as decreased heat removal events, which are described in the UFSAR, Section 15.2. Of these, the full power turbine trip without steam dump is typically the limiting AOO.

In addition to the decreased heat removal events, reactivity insertion events may also challenge the relieving capacity of the MSSVs. The UFSAR safety analyses assume that all of the MSSVs for each steam generator are OPERABLE. If there are inoperable MSSVs, it is necessary to limit the primary system power during steady state operation and AOOs to a value that does not result in exceeding the combined steam flow capacity of the turbine (if available) and the remaining OPERABLE MSSVs. The required limitation on primary system power necessary to prevent secondary system overpressurization may be determined by system transient analyses or conservatively arrived at by simple heat balance calculation.

When the MTC is positive, the reactor power may increase above the initial value during an RCS heatup event (e.g., turbine trip). Thus, for any number of inoperable MSSVs it is necessary to reduce the trip setpoint if a positive MTC may exist at partial power conditions, unless it is demonstrated by analysis that a specified reactor power reduction alone is sufficient to prevent overpressurization of the steam system. The maximum allowable power levels which are specified in TS Table 3.7.1-1 are overly conservative at middle and end-of-life conditions, when the MTC is not positive. Therefore, a specific analysis which credits a middle-of-life MTC was performed to relax the power reduction associated with one inoperable MSSV per steam generator. In addition, for the above case, no reduction in the Power Range Neutron Flux-High trip setpoint is required. The middle-of-life analysis assumes a -10 pcm (per cent mille = 10 -5) /degree of MTC and demonstrates that the maximum allowable power level associated with one inoperable MSSV per steam generator can be relaxed to 87% RTP when cycle burnup is ≥14,000 MWD/MTU. The MTC value at 14,000 MWD/MTU is verified to be more negative than -10 pcm/degree of for each reload cycle.

In its March 22, 2017, license amendment request (LAR), Section 2.0, the licensee stated that the MTC values for TS 3.7.1 Condition A and B are not consistent with the associated TS Bases and LOL/TT event safety analysis. The current TS 3.7.1 Condition A states, "One or more steam generators with one MSSV inoperable and the moderator temperature coefficient (MTC) zero or negative at all power levels." The corresponding Required Action A.1 states, "reduce THERMAL POWER to < 87% RTP". Condition B states, "One or more steam generators with two or more MSSVs inoperable OR One or more steam generators with one MSSV inoperable and the MTC positive at any power level." The corresponding Required Action B.1 states, "Reduce THERMAL POWER to less than or equal to the Maximum Allowable % RTP specified in Table 3.7.1-1 for the number of OPERABLE MSSVs." Thus according to TS Table 3.7.1-1, for one inoperable MSSV, power is reduced to 60% RTP.

FNP TS 3.7.1 Bases states, in part, "The middle-of-life analysis assumes a -10 pcm/°F MTC and demonstrates that the maximum allowable power level associated with one inoperable MSSV per steam generator can be relaxed to 87% RTP when core average burnup is > 14,000 MWD/MTU." The licensee further states that a power reduction to 87%, rather than 60%, when

MTC is less than zero but greater than -10 pcm/°F is not supported by the TS Bases and LOL/TT safety analysis calculations. This band, with MTC between zero and -10 pcm/°F, correlates to a cycle burnup of < 14,000 MWD/MTU, and should require a power reduction to 60% rather than allowing power to be reduced to 87%. Therefore, the NRC staff finds that the current TS would result in a non-conservative condition post-accident when the plant was operating prior to middle-of-life (cycle burnup < 14,000 MWD/MTU) where MTC would be less negative than the analyzed -10 pcm/°F value that corresponds to the relaxed 87% RTP.

This discrepancy was entered into the licensee's corrective action program, and a past operability review was conducted by reviewing narrative logs for the past three (3) years. During that timeframe, TS 3.7.1 Condition A was only entered once when burnup records indicated 21,092.66 MWD/MTU, and the MTC value was more negative than -10 pcm/°F. The burnup at which the MTC becomes more negative than -10 pcm/°F is confirmed to occur each cycle prior to 14,000 MWD/MTU cycle burnup per the Westinghouse reload safety analysis checklist. It was stated in the LAR that a standing order was issued by Operations to compensate for the currently written, non-conservative TS Condition A. The standing order requires operators to reduce power to 60%, when cycle burnup is < 14,000 MWD/MTU to ensure the actions are consistent with the current TS Bases and LOL/TT safety analysis. The proposed amendment of TS 3.7.1 Condition A and Condition B will rectify this inconsistency between TS and TS Bases and plant licensing basis making the TS more conservative than as it is currently written.

In the LAR, the licensee provided clarification that the error introducing the incorrect MTC value was executed inadvertently during the ISTS conversion. The licensee is requesting to revert Conditions A and Condition B back to the approved value used prior to the ISTS conversion, which specified actions based on cycle burnup.

3.3 NRC Staff Conclusion

Based on the above, the NRC staff concludes that the proposed amendment to revise Condition A and Condition B of TS 3.7.1 to specify actions based on cycle burnup (14,000 MWD/MTU) is more conservative than the current values for MTC specified in the TS. In addition, the amendment will make the TS consistent with TS Bases and related safety analysis assumptions in plant licensing basis. Therefore the staff concludes that the proposed changes are acceptable because the FNP TSs, as amended, will be derived from the analyses and evaluation included in the safety analysis report and will remain consistent with the facility operation that will be within safety limits, and that the LCOs will be met consistent with 10 CFR Section 50.36(c)(2).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments on December 28, 2017. On December 28, 2017, the State official confirmed that the State of Alabama had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the Federal Register on May 9, 2017 (82 FR 21562). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:

M. Hamm, NRR
M. Razzaque, NRR

Date: March 1, 2018

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS TO CORRECT NON-CONSERVATIVE TECHNICAL SPECIFICATION 3.7.1 (CAC NOS. MF9465, MF9466; EPID NO. L-2017-LLA-0181) DATED MARCH 1, 2018

DISTRIBUTION:
PUBLIC
RidsRgn2MailCenter Resource
RidsNrrLAKGoldstein Resource
RidsNrrDssStsb Resource
RidsNrrDeEvib Resource
RidsACRS_MailCTR Resource
RidsNrrPMFarley Resource
MRazzaque, NRR
MHamm, NRR

ADAMS Accession No.: ML17291A781

OFFICE	NRR/DORL/LPL2-1/PM	NRR/DORL/LPL2-1/LA	NRR/DSS/STSB/ABC	NRR/DSS/SRXB/BC
NAME	SWilliams	KGoldstein	JWhitman	EOesterle
DATE	12/28/2017	12/18/2017	10/25/2017	10/10/2017
OFFICE	OGC - NLO	NRR/DORL/LPL2-1/BC	NRR/DORL/LPL2-1/PM	
NAME	JGillespie	MMarkley	SWilliams	
DATE	1/9/2018	2/26/2018	3/1/2018	

OFFICIAL RECORD COPY