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 PARRISH, J.V. Washington Public Power Supply System
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SUBJECT: Submits "Inservice Insp Summary Rept For Refueling Outage
 RF94A, Spring 1994."

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

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October 19, 1994
GO2-94-238

Docket No. 50-397

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: **WNP-2, OPERATING LICENSE NPF-21
INSERVICE INSPECTION SUMMARY REPORT NIS-1
NIS-2, OWNER'S REPORTS**

Reference: 1) Letter, GO2-93-234, dated September 16, 1993, JV Parrish (SS)
to NRC, same subject
2) Letter, GO2-94-062, dated March 16, 1994, JV Parrish (SS) to
NRC, same subject

In accordance with the requirements of 10 CFR 50.55a(g) and IWA-6230 of ASME Section XI, as previously agreed, the Supply System hereby submits five (5) copies of the WNP-2 Inservice Inspection (ISI) Summary Report for the Spring 1994 refueling outage. Included in the Summary Report are the NIS-1 Owner's Data Report for inservice inspections performed during the Spring 1994 refueling outage, and NIS-2 Owner's Reports on repair or replacement completed since the filing of the last ISI Summary Report and Supplement No. 1 to the ISI Summary Report (see Reference 2).

Also enclosed are revised Relief Request ISI-2-003 and new Relief Request ISI-2-011 concerning the limited Code examination coverage detailed in Table I of the Spring, 1994, ISI Summary Report. The relief requests are for the first inspection interval and are submitted for your review and approval.

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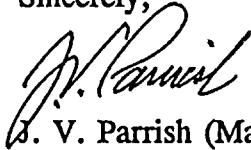
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**INSERVICE INSPECTION SUMMARY REPORT NIS-1/NIS-2, OWNER'S
REPORTS**

Should you have any questions or desire additional information regarding this matter,
please call me or Mr. P.R. Bemis at (509) 377-4027.

Sincerely,



J. V. Parrish (Mail Drop 1023)
Assistant Managing Director, Operations

JRH/ml
Attachments

cc: LJ Callan - NRC RIV
KE Perkins, Jr. - NRC RIV, Walnut Creek Field Office
NS Reynolds - Winston & Strawn
JW Clifford - NRC
DL Williams - BPA/399
NRC Sr. Resident Inspector - 927N

RELIEF REQUEST ISI-2-011

Component or System

ASME Section III Class 1 and 2, Section XI Categories B-K-1 and C-C integral attachment welds for piping, pumps, and valves.

Welds

<u>Code Class</u>	<u>Category</u>	<u>Item No.</u>	<u>Identification No.</u>	<u>ISI Diagram No.</u>
1	B-K-1	B10.10	RRC-HA-1(W)	RRC-101-1
1	B-K-1	B10.10	RRC-HB-1(W)	RRC-102-1
2	C-C	C3.40	RHR-77(W)	RHR-205-1
2	C-C	C3.40	RHR-410(W)	RHR-203-2

Welds RRC-HA-1(W) and RRC-HB-1(W) are SA 240 Type 304 material welded to SA 358 GR 304 CL 1 pipe. Welds RHR-77(W) and RHR-410(W) are SA36 material welded to SA 308 GR B pipe.

Code

All of the subject welds were designed and fabricated to ASME Section III Class 1 or 2. The inservice inspection is to be performed to the 1980 Edition, Winter 1980 Addenda of ASME Section XI.

Section XI requires examination of 100% of each category B-K-1 and C-C weld. All integral attachment welds require examination.

The following examinations are required for examination category B-K-1 and C-C:

<u>Item no</u>	<u>Description</u>	<u>Exam Method</u>
B10.10	Integrally welded attachments	Surface
C3.40	Integrally welded attachments	Surface

Basis for Requesting Relief

Relief is required from ASME Section XI examination requirements for the two item B10.10 welds on the basis of partial inaccessibility of the weld due to plant design and high dose required to prepare for and perform the examinations. The welds identified in this relief request require disassembly of a component support collar to gain access to perform a 100% Code examination.

Relief is required from ASME Section XI examination requirements for the two item C3.40 welds on the basis of inaccessibility of the welds due to their location in a pipe chase where access will place a hardship on the plant to gain access.

Alternative Examinations

The accessible portion of the item B10.10 welds, without removing the component support, of each weld will be examined per Section XI requirements.

No alternate examination is proposed for the item C3.40 welds.

Impact on Plant Quality and Safety

There will be no adverse impact on plant quality and safety by doing only a partial Code examination of the item B10.10 welds.

1. The attachment welds have passed dye penetrant examination in accordance with Section III.
2. Seventy-five (75) percent of weld RRC-HA-1(W) and fifty (50) percent of weld RRC-HB-1(W) are accessible without removing the component support collar
3. The percent of category B-K-1 welds examined during the first inspection interval exceeds 95% of total welds in this category. During the first inspection interval no unacceptable indications were found in this sample.
4. Other similar welds in this system and in the area of the subject welds have or will receive full Code surface examination coverage. The integrity of the pressure boundary can thus be verified by sampling.

There will be no adverse impact on plant quality and safety by doing only a partial Code examination of the item C3.40 welds.

1. The attachment welds have passed magnetic particle examination in accordance with Section III.

3. The percent of category C-C welds examined in the first inspection interval exceeds 96% of total welds in this category. During the first inspection interval no unacceptable indications were found in this sample.
4. Other similar welds in this system and in the area of the subject welds have or will receive full Code surface examination coverage. The integrity of the pressure boundary can thus be verified by sampling.

The sample sizes in these two categories are reasonably large and representative and assure continued plant quality and safety.

REQUEST FOR RELIEF NO. ISI-2-003 (revised)

Component or
System

Component Supports , ASME Section III Code Class 1, 2, and 3
Welded Attachments , ASME Section III Code Class 3

Code

All of the component supports were designed and fabricated to ASME Section III, NF. *All of the welded attachments were designed and fabricated to ASME Section III, ND.* The inservice inspection is to be performed to ASME Section XI 1980 Edition, Winter 1980 Addenda.

Number of
Component
Supports
items

<u>Category</u>	<u>Item No.</u>	<u>No. of Items</u>
F-B	F-1,F-2	16 32
F-C	F-1,F-2,F-3,F-4	1
D-B	D2.20	5
D-C	D3.20	1

Supports and welded attachments are listed at the end of this request for relief.

Section XI
Requirements

Section XI requires a visual examination (~~VT-3~~) of component supports and Class 3 welded attachments as follows:

Class 1, 2, 3 category F-B VT-3
Class 1, 2, 3 category F-C VT-3 and VT-4
Class 3 category D-B VT-3
Class 3 category D-C VT-3

Basis for
Requesting
Relief

The component supports and welded attachments are completely or partially inaccessible to examination. The component supports and welded attachments are within or close to wall penetrations which are ~~foam filled for fire protection barriers~~ or enclosed in cubicles or pipe chases. ~~The support is covered by the foam.~~ A loss of function of the component support is expected to be identified at adjacent supports which are examined. For supports within wall penetrations, it should also be noted that the pipe is completely surrounded by concrete with the metal support embedded in the concrete; the annulus between the pipe and concrete is foam filled. If any failure did occur, the concrete would perform a backup support function.

REQUEST FOR RELIEF NO. ISI-2-003

Alternate
Examinations

The component supports *and welded attachments* are completely or partially inaccessible to all examination techniques. *No alternate examinations are proposed.*

Impact of Plant
Quality and
Safety

There will be no adverse impact on plant quality and safety. Failure of these component supports *or welded attachments* will not prevent the reactor from being shut down. *During the first inspection period the following percent of items were examined:*

<u>Examination Category</u>	<u>Percent</u>
D-B	> 97%
D-C	> 90%
F-B	> 93%
F-C	> 99%

No unacceptable indications were found during these examinations.

Later editions of ASME Section XI define the sample size for category F-B (category F-A in later Code editions) as 10% of the Class 3 piping supports. The percent of Class 3 component supports examined during the first interval (> 93% of rigid type supports for F-B) exceeds the minimum sample size of 10% required in later Code editions.

Later editions of ASME Section XI define the sample size for category F-C (category F-A in later Code editions) as 15% of the Class 2 piping supports. The percent of Class 2 component supports examined during the first interval (> 93% of spring type supports for F-B) exceeds the minimum sample size of 15% required in later Code editions.

The Class 3 welded attachments in categories D-B and D-C examined during the first inspection interval represent greater than 96% of the total welded attachments in these two categories.

The sample sizes in these four categories are reasonably large and representative and assure continued plant quality and safety.

REQUEST FOR RELIEF NO. ISI-2-003

List of Inaccessible Component Supports
and Class 3 Welded Attachments

Identification Number	Drawing	Description	Category	Item No.
SW-69 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
SW-67 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
SW-72 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
SW-317 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
SW-152 <i>see note 1</i>	SW-303	Rigid	F-B	F-1, F-2
SW-431 <i>see note 1</i>	SW-303	Rigid	F-B	F-1, F-2
SW-137 <i>see note 1</i>	SW-303	Rigid	F-B	F-1, F-2
SW-438 <i>see note 1</i>	SW-303	Rigid	F-B	F-1, F-2
SW-203 <i>see note 1</i>	SW-303	Rigid	F-B	F-1, F-2
SW-77 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
SW-34 <i>see note 1</i>	SW-305	Rigid	F-B	F-1, F-2
SW-142 <i>see note 1</i>	SW-303	Rigid	F-B	F-1, F-2
SW-60 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
SW-916N <i>see note 1</i>	SW-307	Rigid	F-B	F-1, F-2
SW-75 <i>see note 1</i>	SW-301	Rigid	F-B	F-1, F-2
RCIC-18 <i>see note 1</i>	RCIC-205	Rigid	F-B	F-1, F-2
FPC-64 <i>see note 1</i>	FPC-301	Box	F-B	F-1, F-2
FPC-64(W) <i>see note 1</i>	FPC-301	Welded Attach	D-C	D3.20
FPC-98 <i>see note 1</i>	FPC-304	Rigid	F-B	F-1, F-2
FPC-114 <i>see note 2</i>	FPC-304	Rigid	F-B	F-1, F-2
FPC-203 <i>see note 1</i>	FPC-304	Box	F-B	F-1, F-2
LPCS-19 <i>see note 1</i>	LPCS-202	Anchor	F-B	F-1, F-2
RHR-53 <i>see note 3</i>	RHR-207	Spring	F-C	F-1, F-2, F-3, F-4

Identification Number	Drawing	Description	Category	Item No.
<i>RHR-99 see note 1</i>	<i>RHR-210</i>	<i>Anchor</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>RHR-174 see note 2</i>	<i>RHR-201</i>	<i>Box</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>RHR-605 see note 2</i>	<i>RHR-201</i>	<i>Strut</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>RHR-606 see note 2</i>	<i>RHR-201</i>	<i>Strut</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SLC-4453-57 see note 1</i>	<i>SLC-101</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SW-90 see note 1</i>	<i>SW-307</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SW-90(W) see note 1</i>	<i>SW-307</i>	<i>Welded Attach</i>	<i>D-B</i>	<i>D2.20</i>
<i>SW-123 see note 1</i>	<i>SW-301</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SW-123(W) see note 1</i>	<i>SW-301</i>	<i>Welded Attach</i>	<i>D-B</i>	<i>D2.20</i>
<i>SW-439 see note 1</i>	<i>SW-303</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SW-439(W) see note 1</i>	<i>SW-303</i>	<i>Welded Attach</i>	<i>D-B</i>	<i>D2.20</i>
<i>SW-946N see note 1</i>	<i>SW-314</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SW-946N(W) see note 1</i>	<i>SW-314</i>	<i>Welded Attach</i>	<i>D-B</i>	<i>D2.20</i>
<i>SW-951N see note 1</i>	<i>SW-315</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<i>SW-951N(W) see note 1</i>	<i>SW-315</i>	<i>Welded Attach</i>	<i>D-B</i>	<i>D2.20</i>
<i>SW-950N see note 1</i>	<i>SW-315</i>	<i>Rigid</i>	<i>F-B</i>	<i>F-1, F-2</i>
<p><i>Notes</i></p> <p><i>1 Covered or contained within fire barrier</i></p> <p><i>2 In enclosed cubicle or pipe chase</i></p> <p><i>3 Limited examination due to surrounding interferences</i></p>				

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*See
Reports*

SUBJECT: "Primary Reactor Containment 1994 Integrated Leakage Rate
Test Final Rept." W/941021 ltr.

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Docket No. 50-397

October 21, 1994
GO2-94-244

U.S. Nuclear Regulatory Commission
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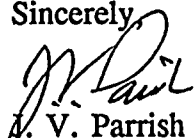
Subject: **WNP-2, OPERATING LICENSE NPF-21
REACTOR CONTAINMENT BUILDING
INTEGRATED LEAK RATE TEST**

- References:
- 1) WNP-2 Final Safety Analysis Report, Washington Public Power Supply System
 - 2) Primary Reactor Containment Leakage Testing for Water Cooled Power Reactors, Code of Federal Regulations, Title 10, Part 50, Appendix J
 - 3) Leakage Rate Testing Containment Structures for Nuclear Reactors, American National Standards Institute, Inc., New York, NY; ANSI N45.4-1972
 - 4) Letter, dated April 29, 1987, DM Crutchfield (NRC) to GC Sorensen (SS), "Issuance of Exemption to a Provision of Appendix J and Amendment No. 41 to Facility Operating License No. NPF-21, WPPSS Nuclear Project No. 2 (TAC No. 60740)"

In accordance with the reporting requirements stipulated in Reference 2, and in compliance with the testing commitments, regulations and guidelines specified in References 1, 2, 3, and 4, the Reactor Containment Building Integrated Leak Rate Test, 1994 is submitted.

Should you have any questions or desire additional information regarding this matter, please call me or D. A. Swank at (509) 377-4563.

Sincerely,



J. V. Parrish (Mail Drop 1023)
Assistant Managing Director, Operations

Attachment

cc: LJ Callan - NRC RIV
NS Reynolds - Winston & Strawn
JW Clifford - NRC
KE Perkins, Jr. - NRC RIV, Walnut Creek Field Office

DL Williams - BPA/399
NRC Sr. Resident Inspector - 927N

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