

COLR 94-10, Revision 0
Controlled Copy No. _____

**WNP-2
Cycle 10
Core Operating Limits Report**

June 1994

Washington Public Power Supply System

WNP-2
Cycle 10
Core Operating Limits Report

LIST OF EFFECTIVE PAGES

<u>Page</u>	<u>Revision</u>
i	0
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0
32	0
33	0
34	0

WNP-2
Cycle 10
Core Operating Limits Report

LIST OF EFFECTIVE PAGES

<u>Page</u>	<u>Revision</u>
35	0
36	0
37	0



10/10/10

WNP-2
Cycle 10
Core Operating Limits Report

TABLE OF CONTENTS

	<u>Page</u>
1.0 <u>INTRODUCTION AND SUMMARY</u>	1
2.0 <u>AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)</u> <u>LIMITS FOR USE IN TECHNICAL SPECIFICATION 3.2.1</u>	2
3.0 <u>MINIMUM CRITICAL POWER RATIO (MCPR) LIMIT FOR USE IN</u> <u>TECHNICAL SPECIFICATION 3.2.3</u>	8
4.0 <u>LINEAR HEAT GENERATION RATE (LHGR) LIMIT FOR USE IN</u> <u>TECHNICAL SPECIFICATION 3.2.4</u>	29
5.0 <u>REFERENCES</u>	35



1.0 INTRODUCTION AND SUMMARY

This report provides the Average Planar Linear Heat Generation Rate (APLHGR) limits, the Minimum Critical Power Ratio (MCPR) limits; and the Linear Heat Generation Rate (LHGR) limits for WNP-2, Cycle 10 as required by Technical Specification 6.9.3.1. As required by Technical Specifications 6.9.3.2 and 6.9.3.3, these limits were determined using NRC-approved methodology and are established so that all applicable limits of the plant safety analysis are met. The thermal limits for SPC fuel given in this report are documented in the "Cycle 10 Plant Transient Analysis" (Reference 5.1.1) and the "Cycle 10 Reload Analysis" (Reference 5.1.2). The thermal limits determined through the approved methodology are modified for the GE11 and SVEA-96 LFAs as discussed below.

The WNP-2 Cycle 10 core includes four Siemens Power Corporation (SPC), four GE Nuclear Energy (GE), and four ABB Combustion Engineering Nuclear Operations (ABB CENO) Lead Fuel Assemblies (LFAs). The SPC LFAs were inserted during the reload for Cycle 5. The GE and ABB CENO LFAs were inserted at the beginning of Cycle 6 and were designed to be compatible with the reload fuel utilized in Cycle 6. The LFAs are loaded in core locations which analysis has shown to have sufficient thermal margin such that the LFAs are not expected to be the most limiting fuel assemblies on either a nodal or an assembly power basis. The GE Nuclear Energy GE11 LFAs are described in the "GE11 Lead Fuel Assembly Report for Washington Public Power Supply System Nuclear Project No. 2, Reload 5, Cycle 6" (Reference 5.3.1). This reference describes the design goals of the GE11 LFAs and provides support for monitoring the GE11 LFAs at thermal limits based on the SPC 8x8 reload fuel thermal limits. The ABB CENO SVEA-96 LFAs are described in the "Supplemental Lead Fuel Assembly Licensing Report—SVEA-96 LFAs for WNP-2—Summary" (Reference 5.3.2). The process for developing thermal limits for the SVEA-96 LFAs based upon the SPC 8x8 reload fuel thermal limits is described in References 5.3.2 through 5.3.4

The MAPLHGR limits for the GE11 LFAs are the same as for the SPC 8x8 reload fuel, except that a ratio $[(64-2)/(81-7)]$ is applied to account for the different number of fuel pins in the two designs. The MAPLHGR limits for the SVEA-96 LFAs are the same as for the SPC 8x8 reload fuel, except that a ratio $[(64-2)/(100-4)]$ is applied to account for the different number of fuel pins in the two designs. Furthermore, the MAPLHGR limits for the SVEA-96 LFAs are multiplied by the following constants: (a) 1.04 to account for a different estimation of the local power in the output from POWERPLEX compared to ABB CENO methods and (b) 1.02 to account for a different estimation of exposure in the output from POWERPLEX compared to ABB CENO methods.

The MCPR limit is the maximum of (a) the applicable exposure dependent, full power and full flow MCPR limit, (b) the applicable exposure and power dependent MCPR limit, and (c) the flow dependent MCPR limit specified in this report. This stipulation assures that the safety limit MCPR will not be violated throughout the WNP-2 operating regime. Full power MCPR limits are specified to define operating limits at rated power and flow. For the WNP-2 core, the Turbine Trip without Bypass event is limiting for operation at rated power and flow. Power dependent MCPR limits are specified to define operating limits at other than rated power conditions. For the WNP-2 core, the Feedwater Controller Failure event from reduced power

100

100

100

100

100

is calculated to be more severe than from full power conditions. A flow dependent MCPR is specified to define operating limits at other than rated flow conditions. The reduced flow MCPR limit provides bounding protection for the limiting Recirculation Flow Increase event.

The LHGR limits for the GE11 LFAs are the same as for the SPC 8x8 reload fuel, except that a ratio $([64-2]/[81-7])$ is applied to account for the different number of fuel pins in the two designs. The LHGR limits for the SVEA-96 LFAs are taken directly from Reference 5.3.2.

The reload licensing analyses for this cycle provide operating limits for Extended Load Line (ELLLA) operation which extends the power and flow operating regime for WNP-2 up to the 109% rod line which at full power corresponds to 87% of rated flow. The MCPR limits defined in this report are applicable up to 100% of rated thermal power along and below the 109% rod line. The minimum flow for operation at rated power is 87% of rated flow; the maximum is 106%. References 5.1.1 and 5.1.2 and the references in Section 5.4 document the analyses in support of ELLLA operation.

Preparation, review and approval of this report were performed in accordance with applicable Supply System procedures. The specific topical report revisions and supplements which describe the methodology utilized in this cycle specific analysis are referenced in Section 5.2.

2.0 AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR) LIMITS FOR USE IN TECHNICAL SPECIFICATION 3.2.1

The APLHGRs for use in Technical Specification 3.2.1 shall not exceed the limits shown in Figures 2.1, 2.2, 2.4, and 2.5 when in two-loop operation and in Figures 2.1, 2.3, 2.4, and 2.5 when in single loop operation. The limits for each fuel type as a function of Average Planar Exposure are provided for the SPC reload fuel, the SPC LFAs, the SVEA-96 LFAs, and the GE11 LFAs.



1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

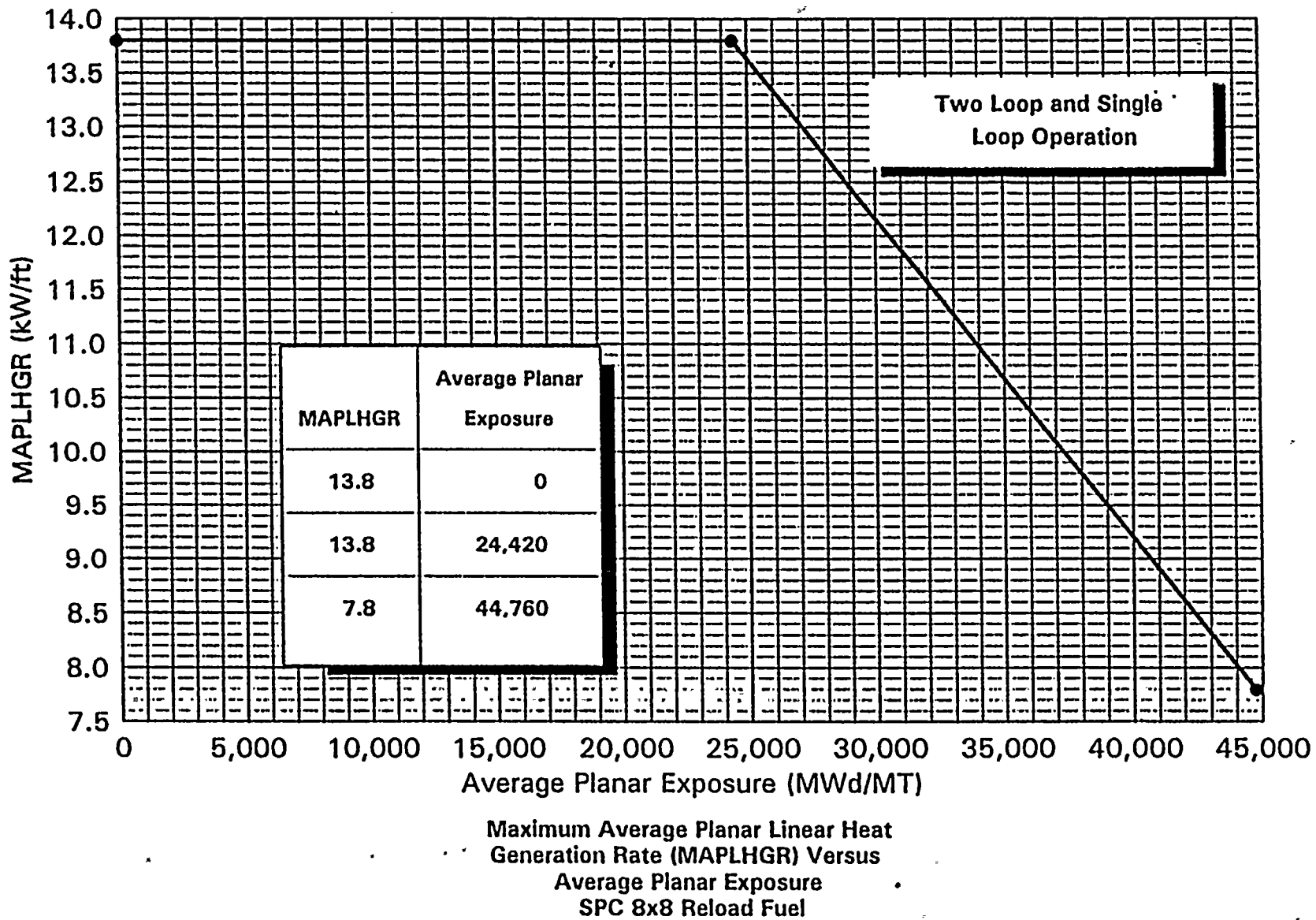
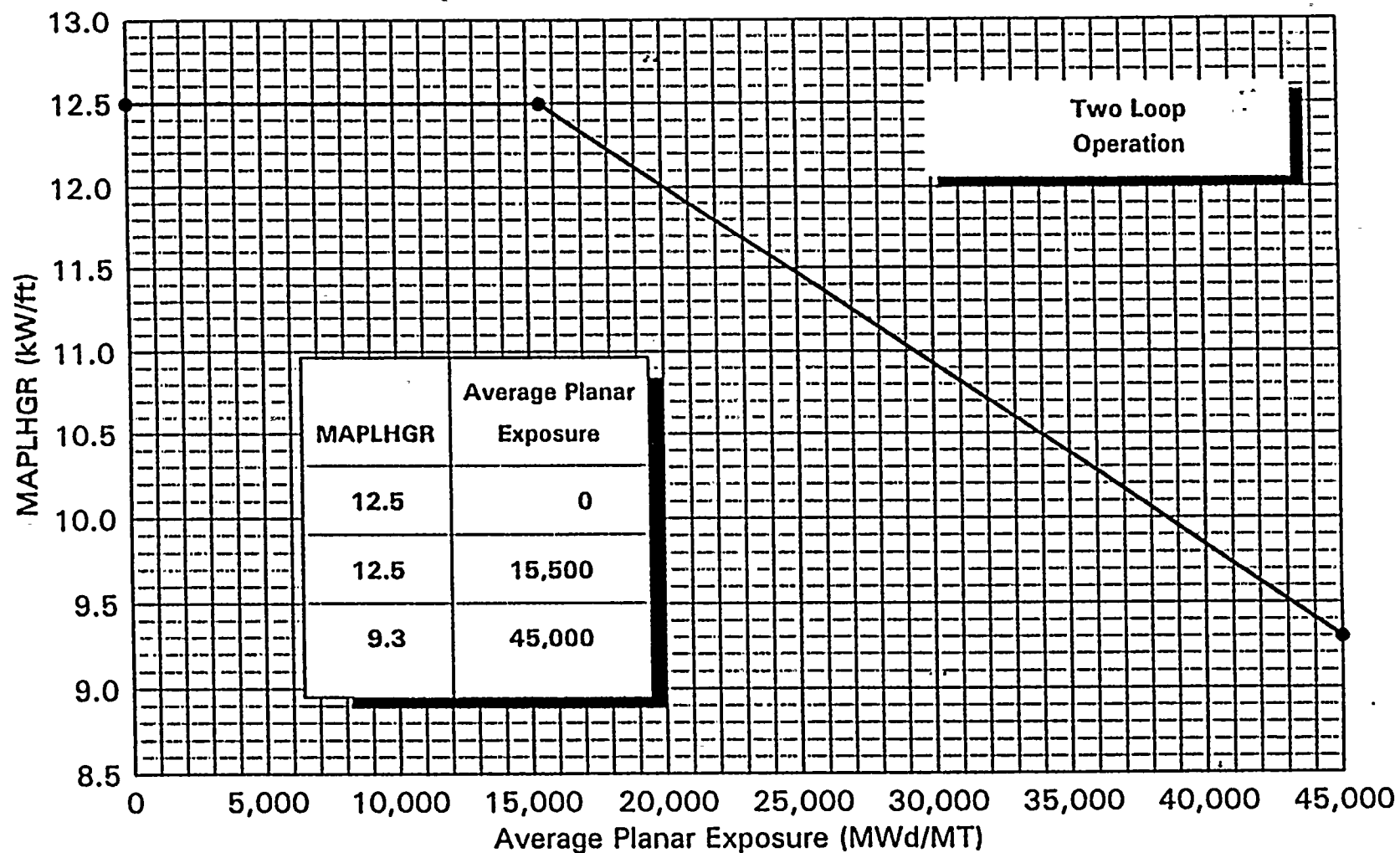
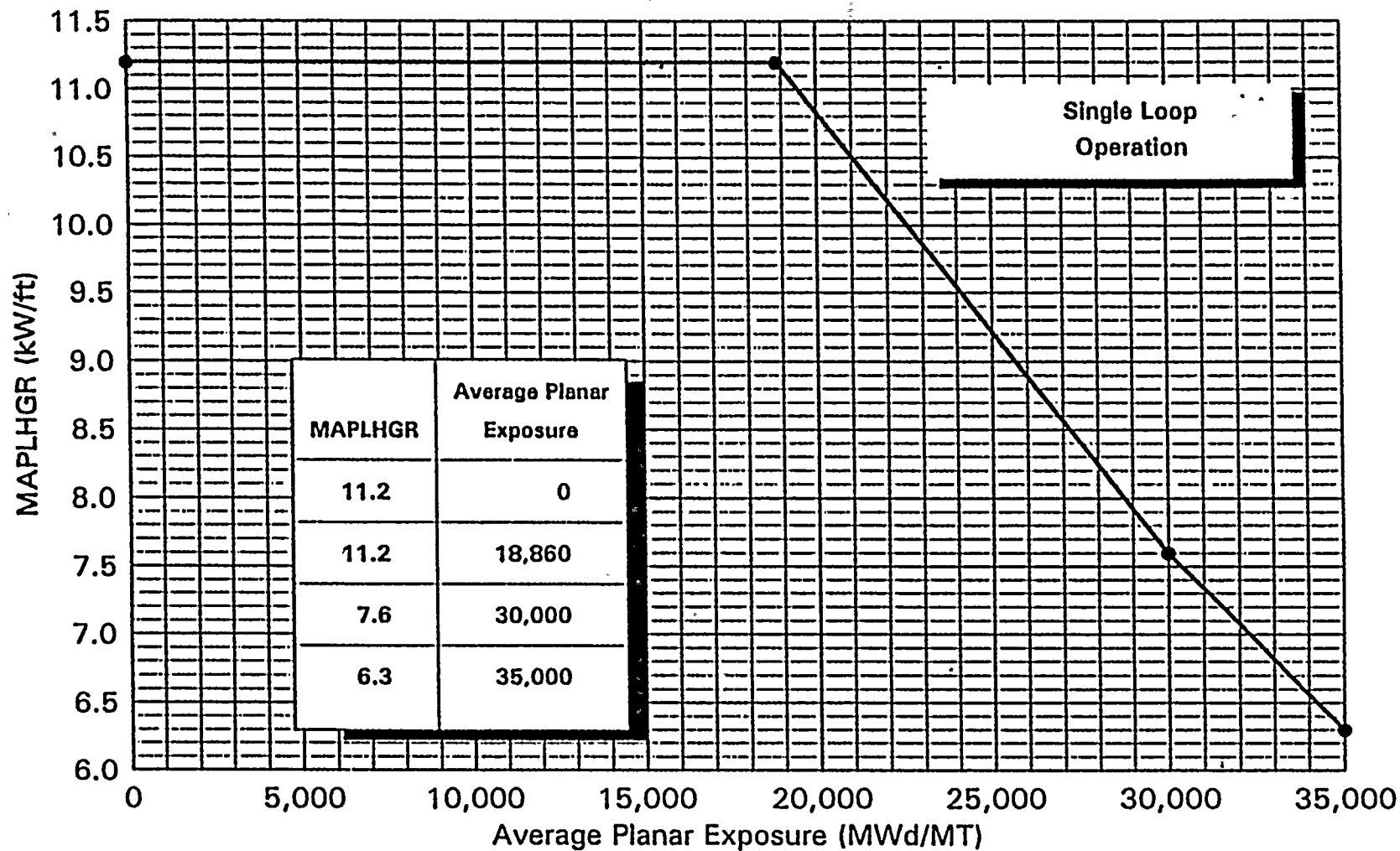


Figure 2.1



Maximum Average Planar Linear Heat
Generation Rate (MAPLHGR) Versus
Average Planar Exposure
SPC 9x9-9X Reload Fuel and SPC 9x9 LFA's

Figure 2.2



Maximum Average Planar Linear Heat
Generation Rate (MAPLHGR) Versus
Average Planar Exposure
SPC 9x9-9X Reload Fuel and SPC 9x9 LFA's

Figure 2.3

10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

11

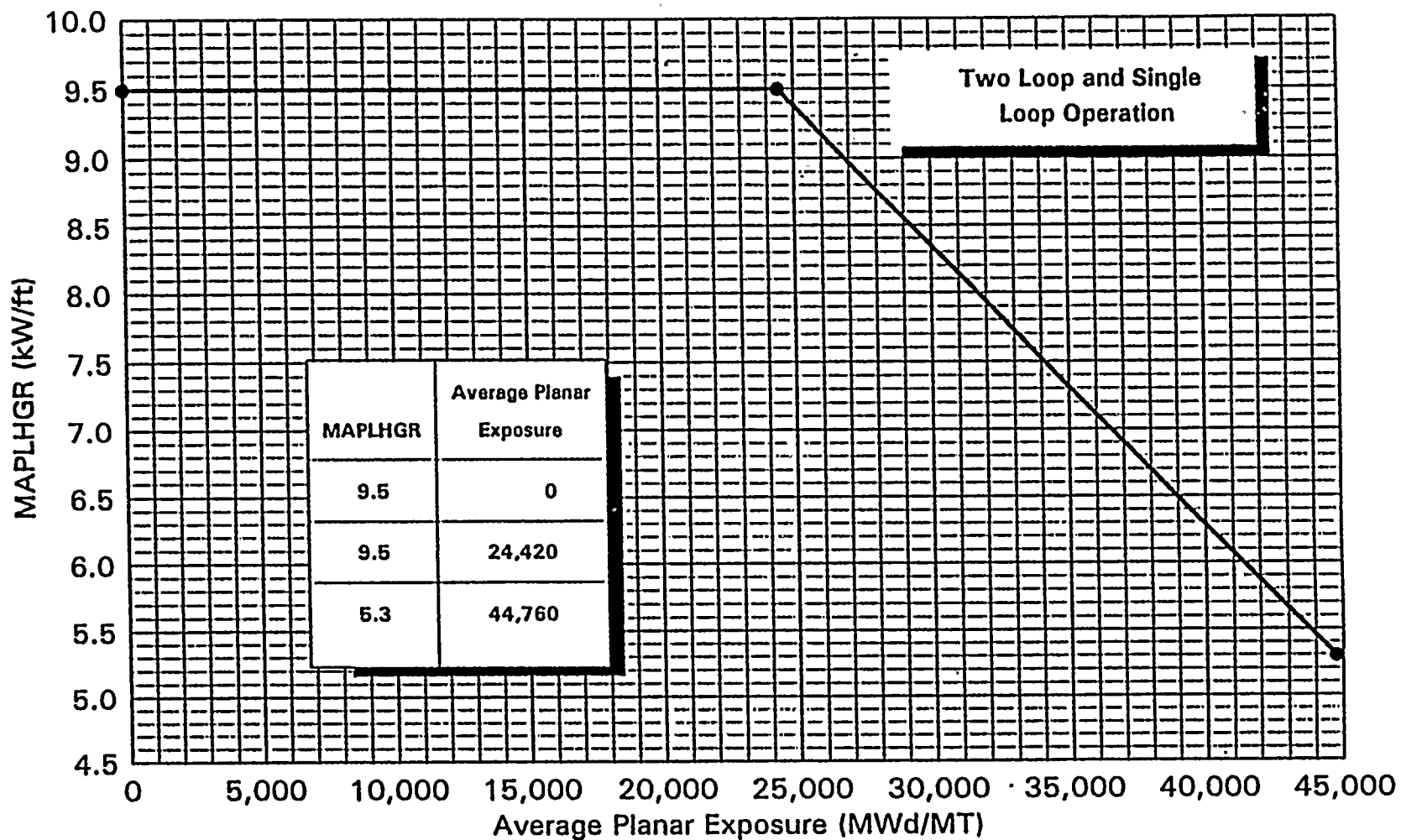
11

10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

11

11

10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100



Maximum Average Planar Linear Heat
Generation Rate (MAPLHGR) Versus
Average Planar Exposure
SVEA-96 Lead Fuel Assemblies

Figure 2.4

1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

101
102
103
104
105
106
107
108
109
110
111
112
113
114
115
116
117
118
119
120
121
122
123
124
125
126
127
128
129
130
131
132
133
134
135
136
137
138
139
140
141
142
143
144
145
146
147
148
149
150
151
152
153
154
155
156
157
158
159
160
161
162
163
164
165
166
167
168
169
170
171
172
173
174
175
176
177
178
179
180
181
182
183
184
185
186
187
188
189
190
191
192
193
194
195
196
197
198
199
200

201

202

203

204

205

206

207

208

209

210

211

212

213

214

215

216

217

218

219

220

221

222

223

224

225

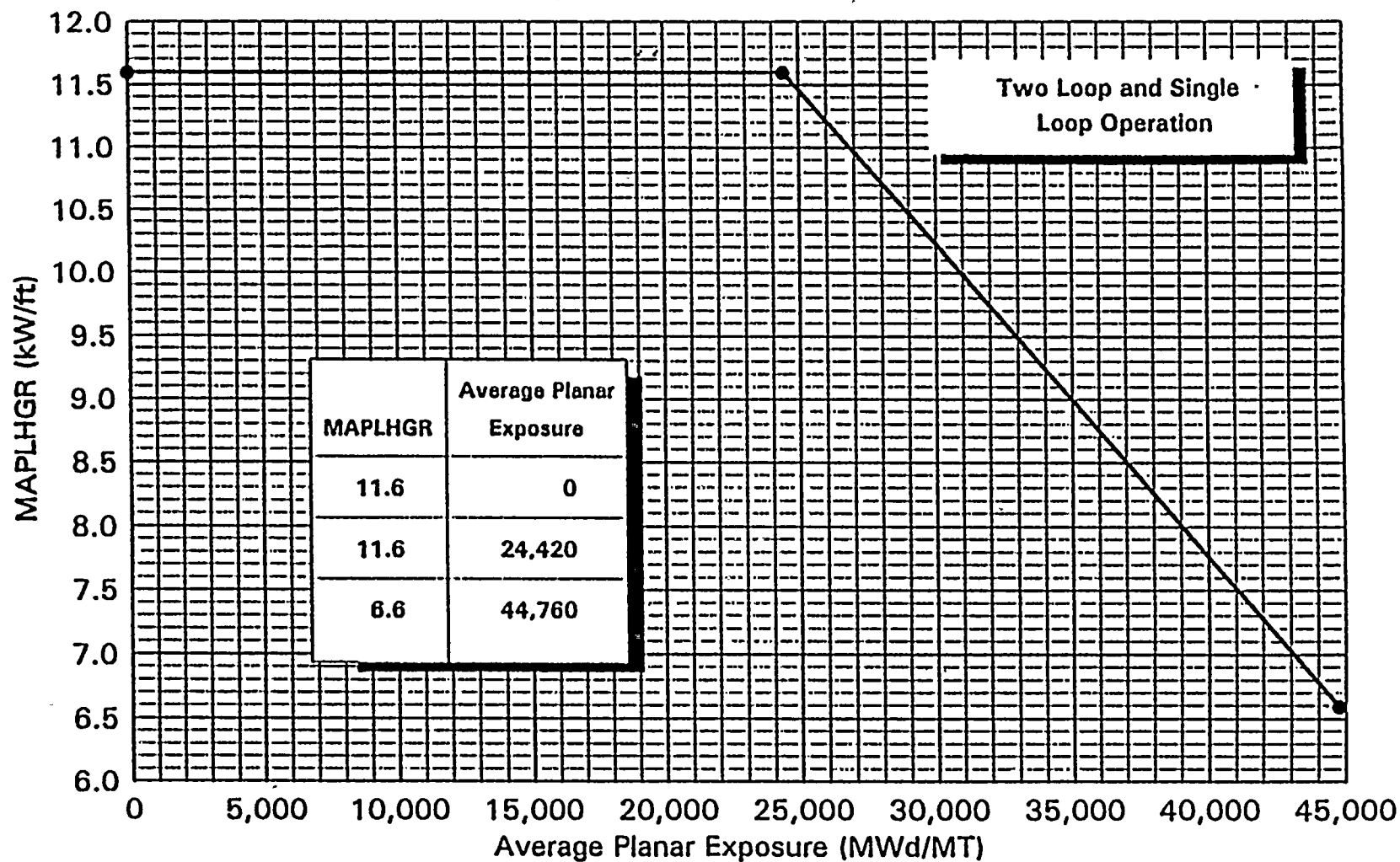
226

227

228

229

230



Maximum Average Planar Linear Heat
Generation Rate (MAPLHGR) Versus
Average Planar Exposure
GE11 Lead Fuel Assemblies

Figure 2.5

5

12

14

16

18

20

22

24

3.0 MINIMUM CRITICAL POWER RATIO (MCPR) LIMIT FOR USE IN TECHNICAL SPECIFICATION 3.2.3

The MCPR limit for use in Technical Specification 3.2.3 shall be:

Greater than or equal to the greater of the limits determined from Tables 3.1a and 3.1b and Figures 3.1 and 3.2a through 3.11b.

100

101

102

103

104

105

106

107

108

109

110

111

112

113

114

115

116

117

118

119

120

121

122

123

124

125

126

127

128

129

130

131

132

Table 3.1a
WNP-2 Cycle 10 MCPR Operating Conditions
Cycle Exposures ≤ 4500 MWd/MTU

Condition Limit		SLMCPR = 1.07 ⁽²⁾			
		SPC 8x8 GE11 LFA	SPC 9x9 LFA	SPC 9x9 LFA	SVEA-96
NSS ⁽¹⁾					
	Full Power	1.24 ⁽³⁾	1.24 ⁽³⁾	1.28	1.44 ⁽³⁾
	Flow Dependent	Figure 3.1			
	Power Dependent ⁽⁴⁾	Fig. 3.2a	Fig. 3.3a	Fig. 3.3a	Fig. 3.2a
TSSS ⁽¹⁾					
	Full Power	1.26	1.24	1.35	1.48
	Flow Dependent	Figure 3.1			
	Power Dependent ⁽⁴⁾	Fig. 3.4a	Fig. 3.5a	Fig. 3.5a	Fig. 3.4a
NSS ⁽¹⁾					
RPT	Full Power	1.28	1.27	1.43	1.51
Inoperable	Flow Dependent	Figure 3.1			
	Power Dependent ⁽⁴⁾	Fig. 3.10a	Fig. 3.11a	Fig. 3.11a	Fig. 3.10a
SLO ⁽²⁾ NSS					
	Full Power	1.56	1.36	1.36	1.98
	Flow Dependent	None			
	Power Dependent ⁽⁴⁾	Fig. 3.2a	Fig. 3.3a	Fig. 3.3a	Fig. 3.2a
SLO ⁽²⁾ TSSS					
	Full Power	1.56	1.36	1.36	1.98
	Flow Dependent	None			
	Power Dependent ⁽⁴⁾	Fig. 3.4a	Fig. 3.5a	Fig. 3.5a	Fig. 3.4a
SLO ⁽²⁾ NSS					
RPT	Full Power	1.56	1.36	1.36	1.98
Inoperable	Flow Dependent	None			
	Power Dependent ⁽⁴⁾	Fig. 3.10a	Fig. 3.11a	Fig. 3.11a	Fig. 3.10a

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

Table 3.1b
WNP-2 Cycle 10 MCPR Operating Conditions
Cycle Exposures > 4500 MWd/MTU

Condition Limit		SLMCPR = 1.07 ^(a)				SLMCPR = 1.07 ^(a) FFTR			
		SPC 8x8 GE11 LFA	SPC 9x9	SPC 9x9 LFA	SVEA-96	SPC 8x8 GE11 LFA	SPC 9x9 LFA	SPC 9x9 LFA	SVEA-96
NSS ^(b)									
	Full Power	1.30	1.27	1.44	1.54	1.32	1.29	1.46	1.58
	Flow Dependent	Figure 3.1				Figure 3.1			
	Power Dependent ^(c)	Fig. 3.2b	Fig. 3.3b	Fig. 3.3b	Fig. 3.2b	Fig. 3.6	Fig. 3.7	Fig. 3.7	Fig. 3.6
TSSS ^(b)									
	Full Power	1.33	1.30	1.49	1.60	1.35	1.32	1.51	1.63
	Flow Dependent	Figure 3.1				Figure 3.1			
	Power Dependent ^(c)	Fig. 3.4b	Fig. 3.5b	Fig. 3.5b	Fig. 3.4b	Fig. 3.8	Fig. 3.9	Fig. 3.9	Fig. 3.8
NSS ^(b)						Not Analyzed			
RPT	Full Power	1.38	1.35	1.61	1.68				
Inoperable	Flow Dependent	Figure 3.1							
	Power Dependent ^(c)	Fig. 3.10b	Fig. 3.11b	Fig. 3.11b	Fig. 3.10b				
SLO ^(a) NSS									
	Full Power	1.56	1.36	1.36	1.98	1.56	1.36	1.36	1.98
	Flow Dependent	None				None			
	Power Dependent ^(c)	Fig. 3.2b	Fig. 3.3b	Fig. 3.3b	Fig. 3.2b	Fig. 3.6	Fig. 3.7	Fig. 3.7	Fig. 3.6
SLO ^(a) TSSS									
	Full Power	1.56	1.36	1.36	1.98	1.56	1.36	1.36	1.98
	Flow Dependent	None				None			
	Power Dependent ^(c)	Fig. 3.4b	Fig. 3.5b	Fig. 3.5b	Fig. 3.4b	Fig. 3.8	Fig. 3.9	Fig. 3.9	Fig. 3.8
SLO ^(a) NSS						Not Analyzed			
RPT	Full Power	1.56	1.36	1.36	1.98				
Inoperable	Flow Dependent	None							
	Power Dependent ^(c)	Fig. 3.10b	Fig. 3.11b	Fig. 3.11b	Fig. 3.10b				



D Notes for Tables 3.1a and 3.1b

Note 1: The scram insertion times must meet the requirements of Technical Specification 3.1.3.4. The NSS MCPR values are based on the SPC transient analysis performed using the control rod insertion times shown below (defined as normal scram speed: NSS). In the event that Surveillance 4.1.3.2 shows these scram insertion times have been exceeded, the MCPR limit shall be determined from the applicable Technical Specification Scram Speed (TSSS) MCPR limits in Tables 3.1a and b.

Position Inserted From Fully Withdrawn	Slowest measured average control rod insertion times to specified notches for all operable control rods for each group of four control rods arranged in a two-by-two array (seconds)
Notch 45	0.380
Notch 39	0.720
Notch 25	1.600
Notch 5	2.950

Note 2: For Single Loop Operation (SLO), the SLMCPR increases by 0.01. The increase is included in the MCPR limits for SLO.

Note 3: For the noted full power MCPR limits, the control rod withdrawal error (CRWE) event is limiting. The turbine trip without bypass (TTNB) event is limiting for the remaining full power limits. CRWE analysis was performed with a nominal rod block monitor (RBM) setpoint of 1.06. Use of the nominal setpoint is in accordance with the methodology described in Reference 5.2.6, consistent with approved industry practice.

Note 4: Power dependent MCPR limits are provided for core thermal powers greater than or equal to 25% of rated power at all core flows. The power dependent MCPR limits for core thermal powers less than or equal to 30% of rated power are subdivided by core flow. Limits are provided for core flows greater than 50% of rated flow and less than or equal to 50% of rated flow, respectively. A step change in the power dependent MCPR limits occurs at 30% of rated power because direct scram on turbine throttle valve closure is automatically bypassed per Technical Specification 3.3.1.

100

100

100

100

100

100

100

100

100

100

100

100

100



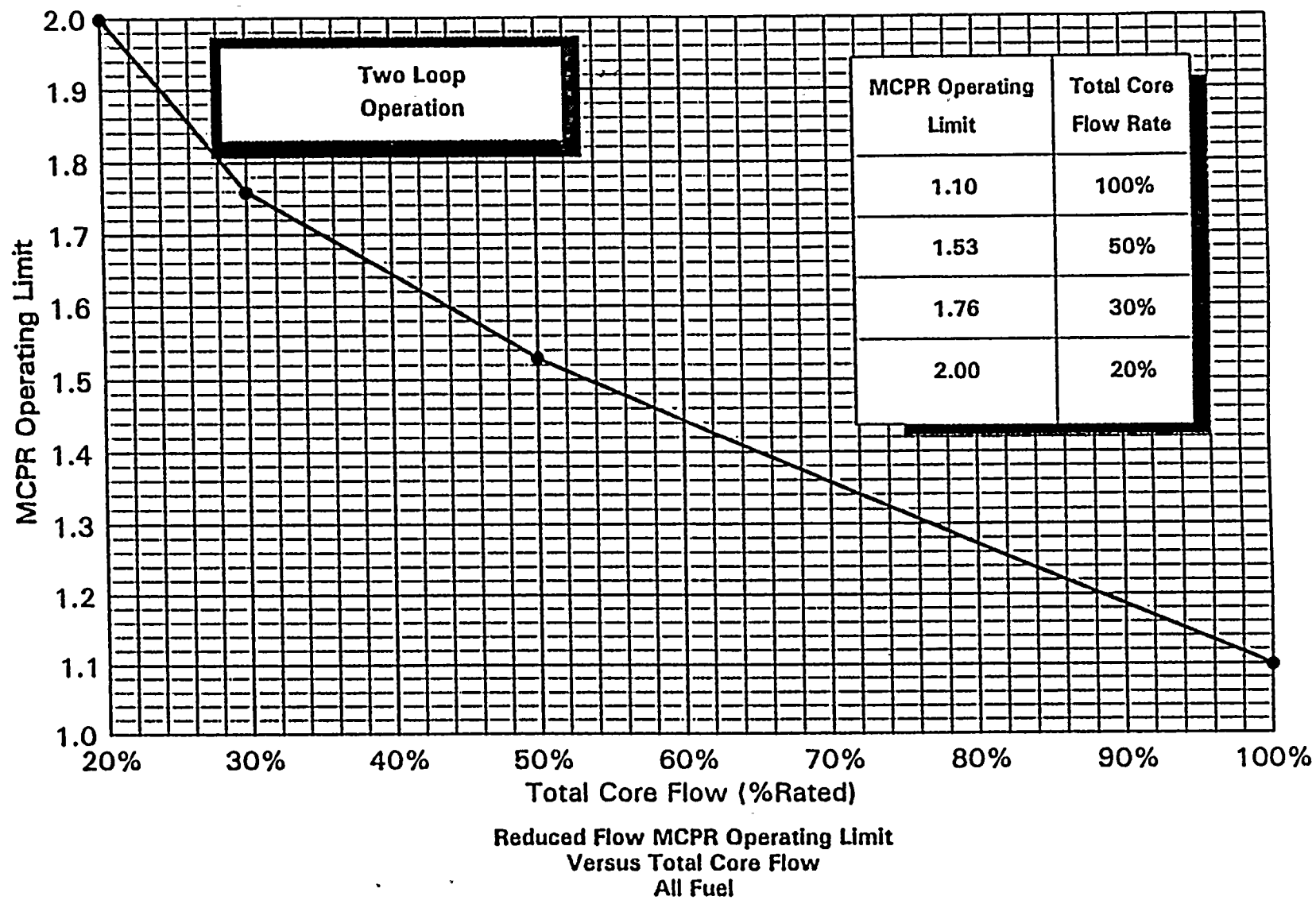


Figure 3.1

10

11

12

13

14

15

16

17

18

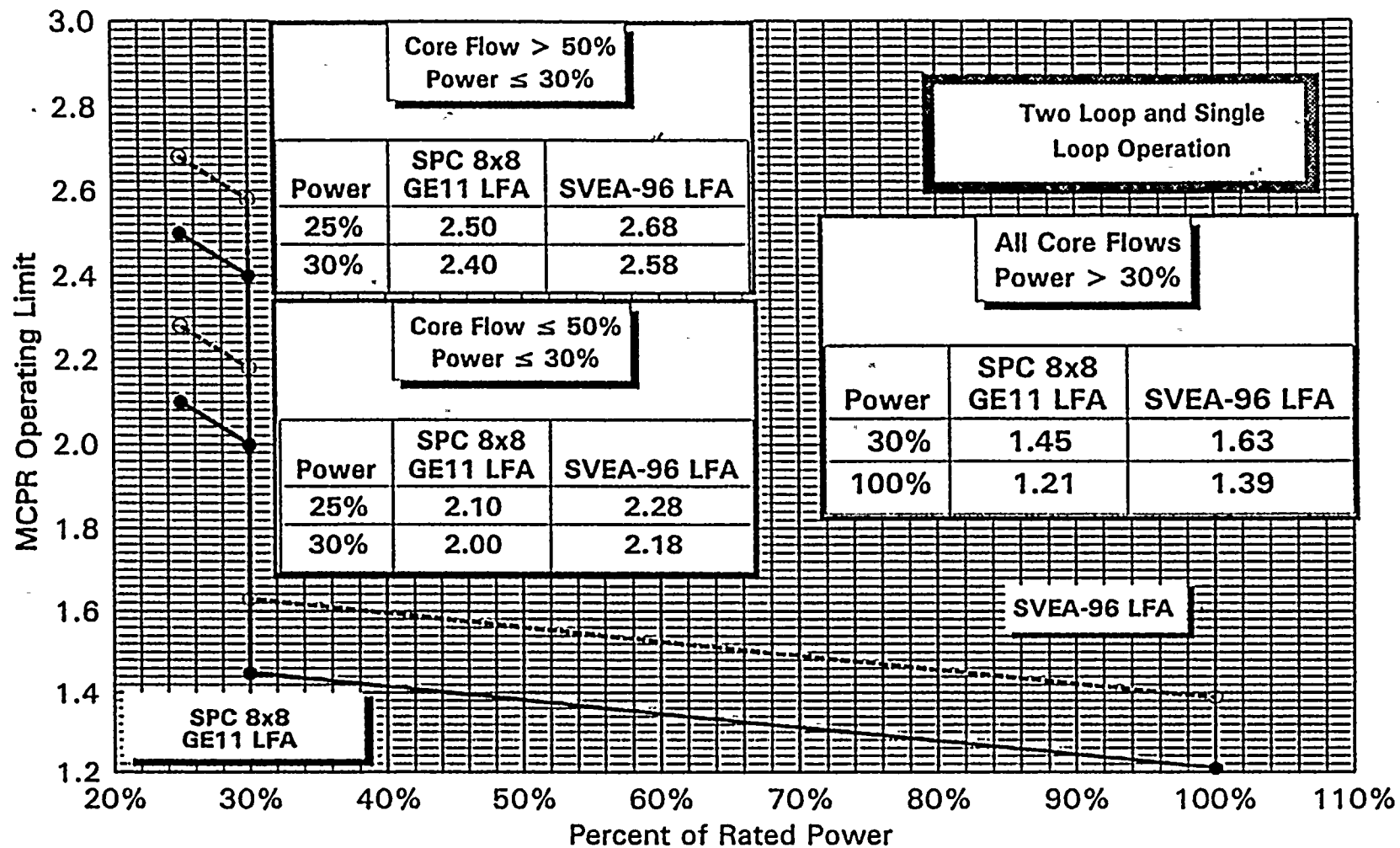
19

20

21

22

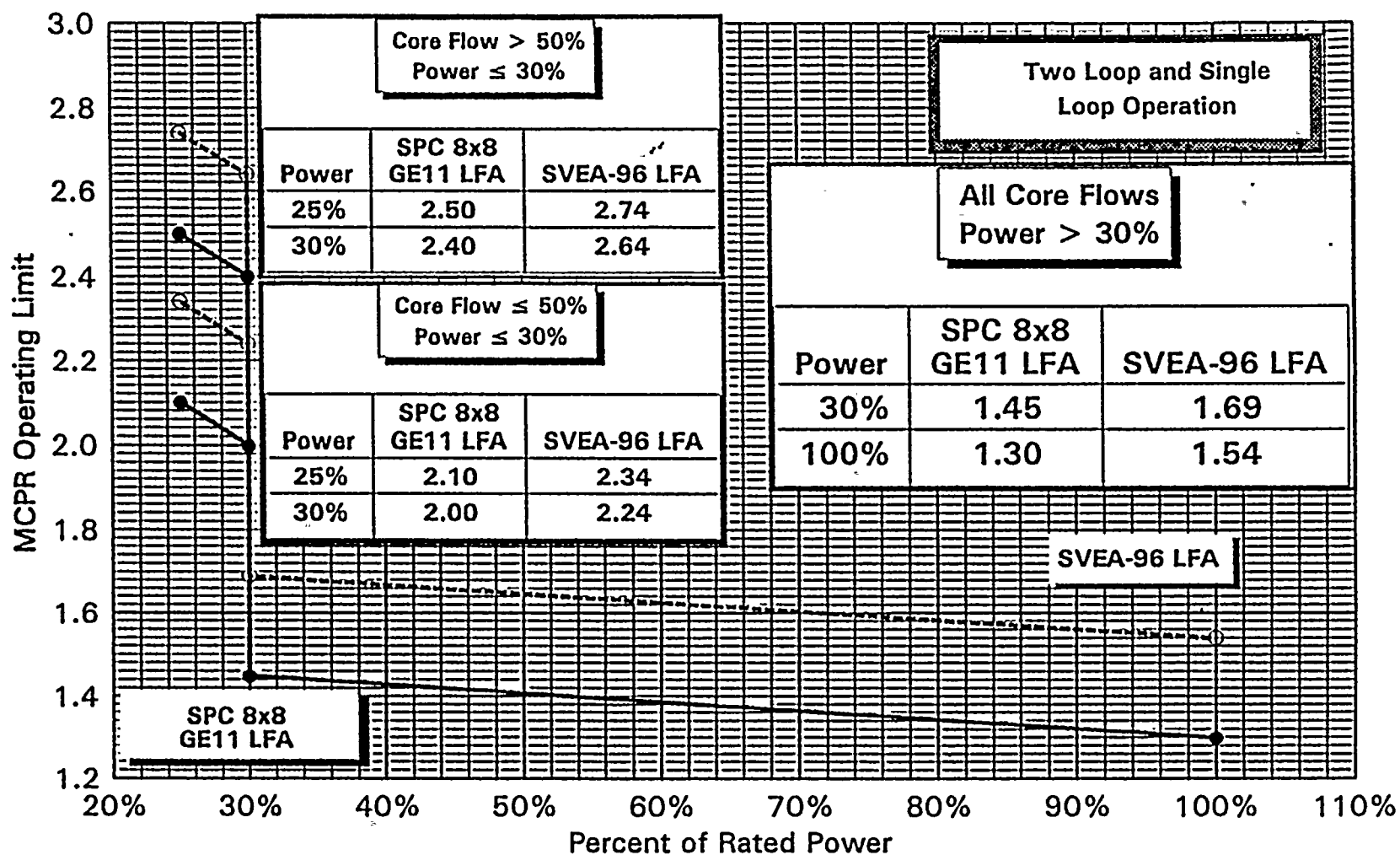




Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Operable
SPC 8x8, GE11 LFA, SVEA-96 LFA
Cycle Exposures ≤ 4500 MWd/MT

Figure 3.2a

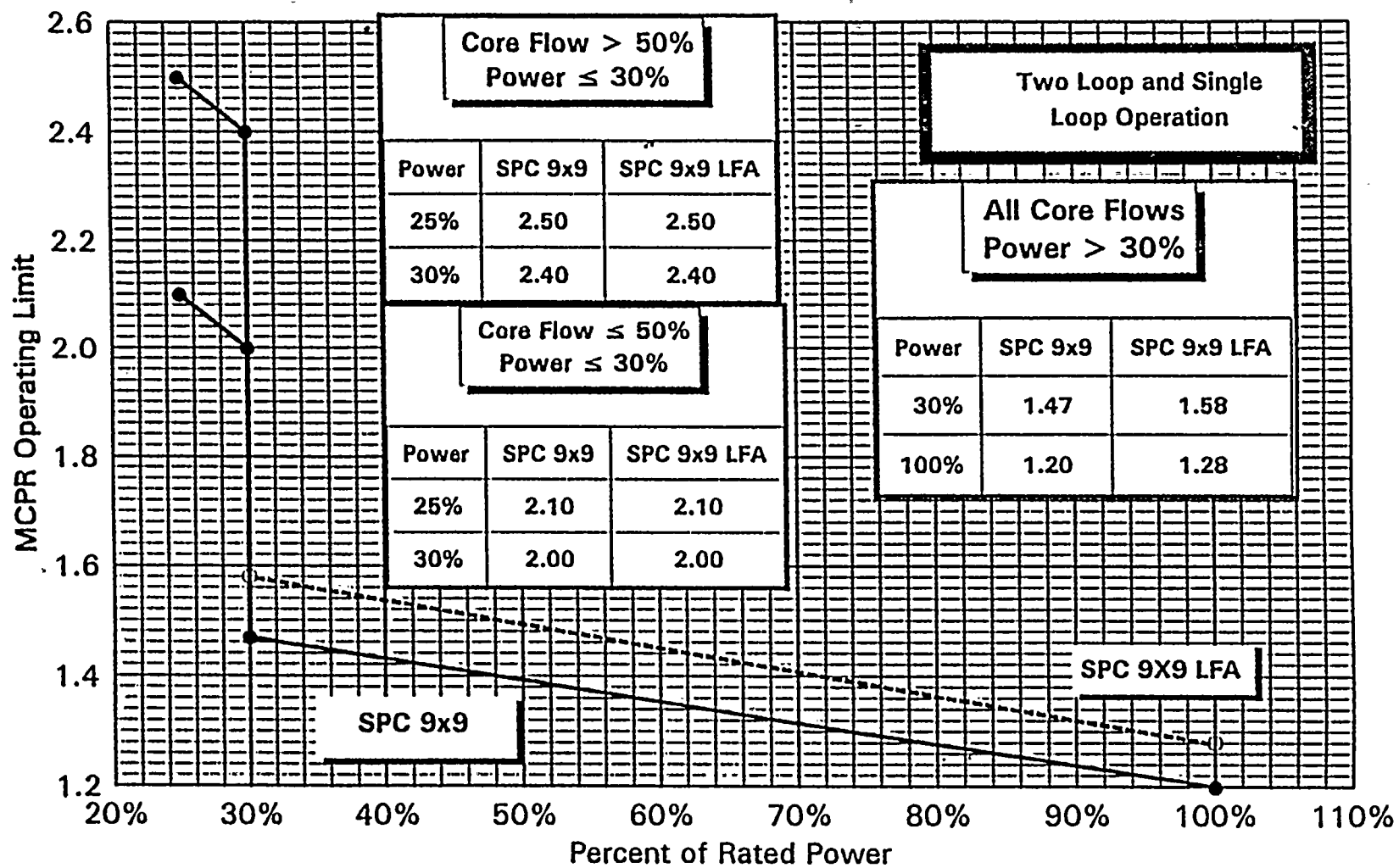




Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Operable
SPC 8x8, GE11 LFA, SVEA-96 LFA
Cycle Exposures > 4500 MWd/MT

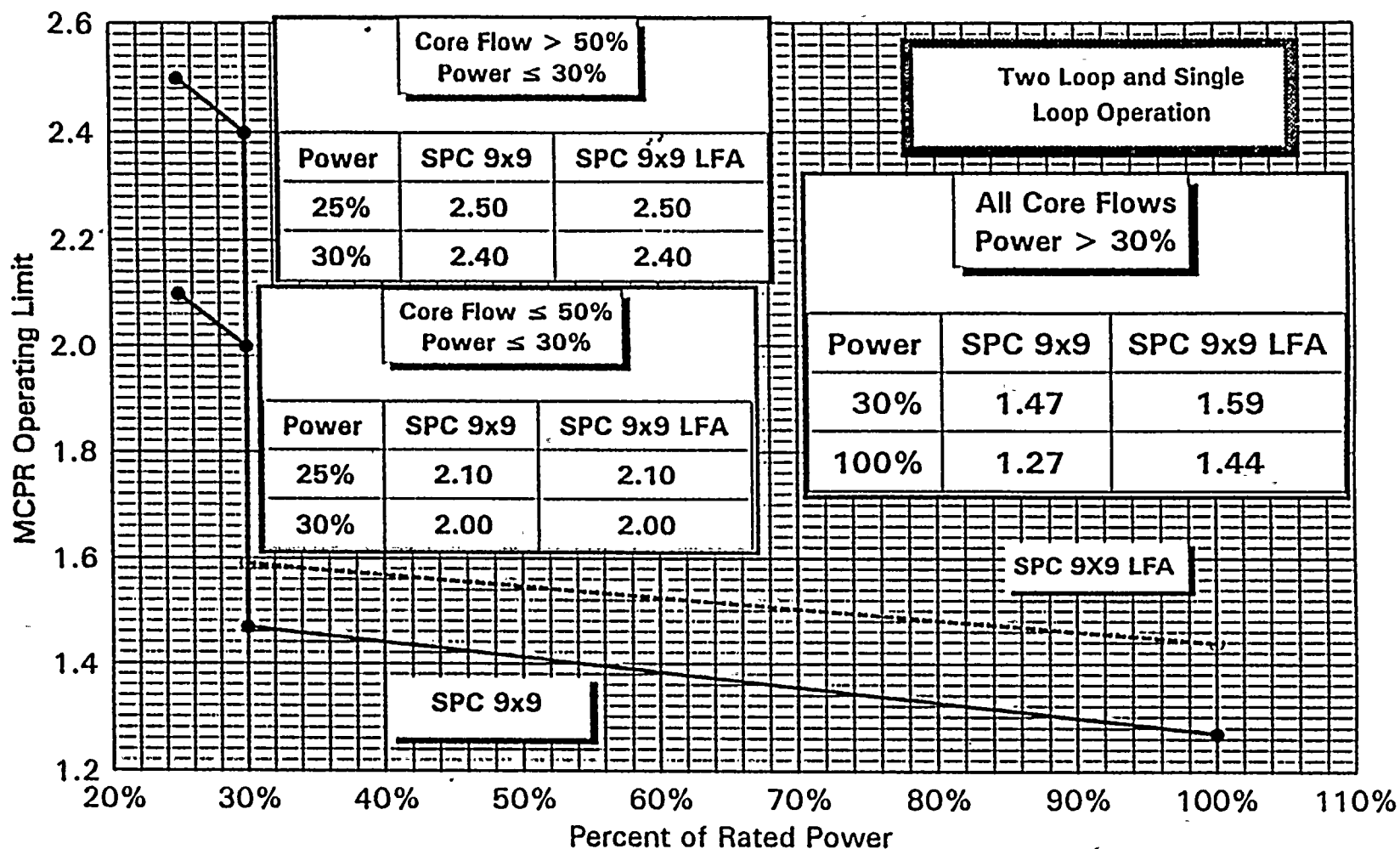
Figure 3.2b





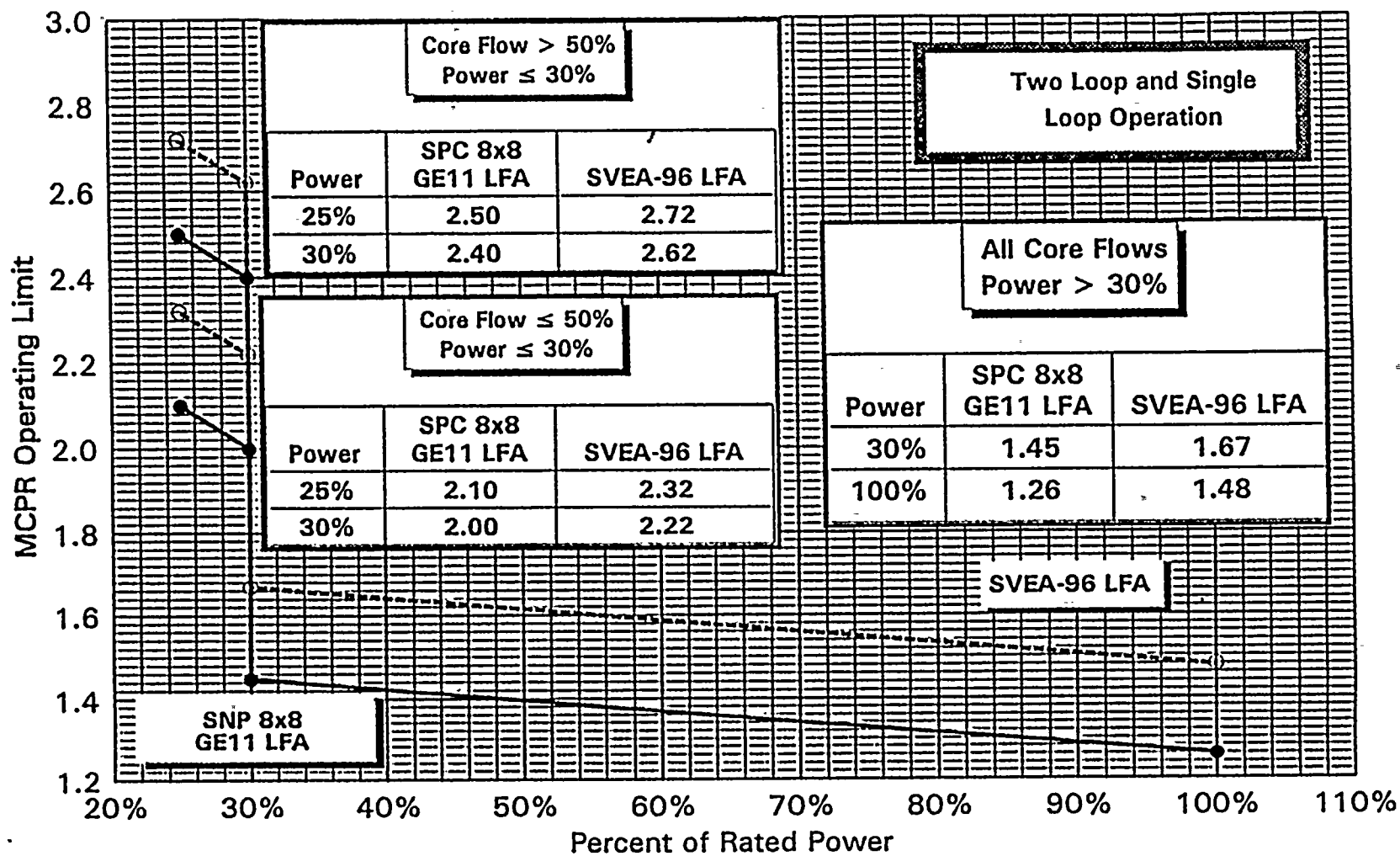
Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Operable
SPC 9x9, SPC 9x9 LFA
Cycle Exposures ≤ 4500 MWd/MT

Figure 3.3a



Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Operable
SPC 9x9, SPC 9x9 LFA
Cycle Exposures > 4500 MWd/MT

Figure 3.3b



Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
TSSS, RPT Operable
SPC 8x8, GE11 LFA, SVEA-96 LFA
Cycle Exposures ≤ 4500 MWd/MT

Figure 3.4a

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

100

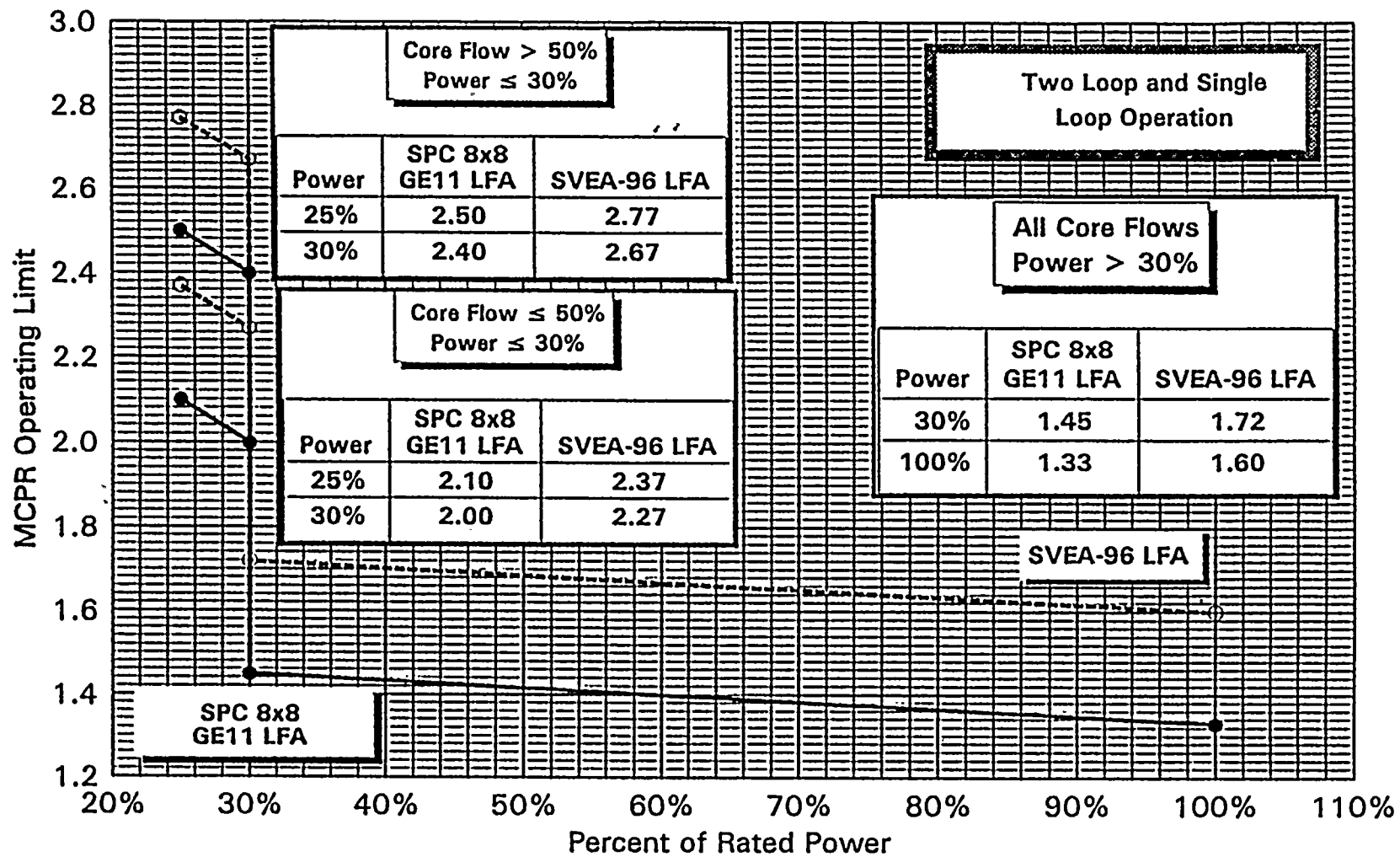
100

100

100

100

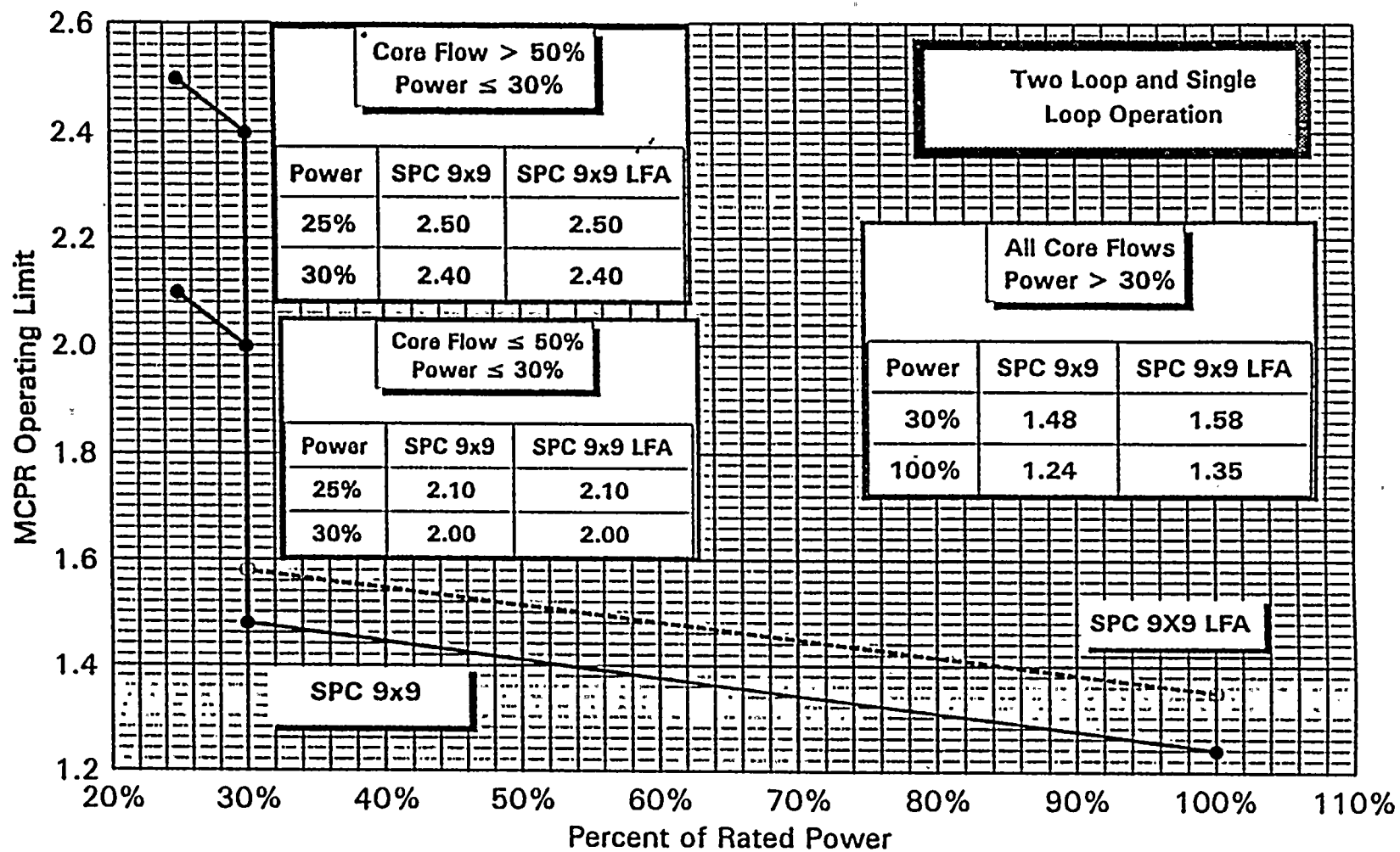
100



Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
TSSS, RPT Operable
SPC 8x8, GE11 LFA, SVEA-96 LFA
Cycle Exposures > 4500 MWd/MT

Figure 3.4b

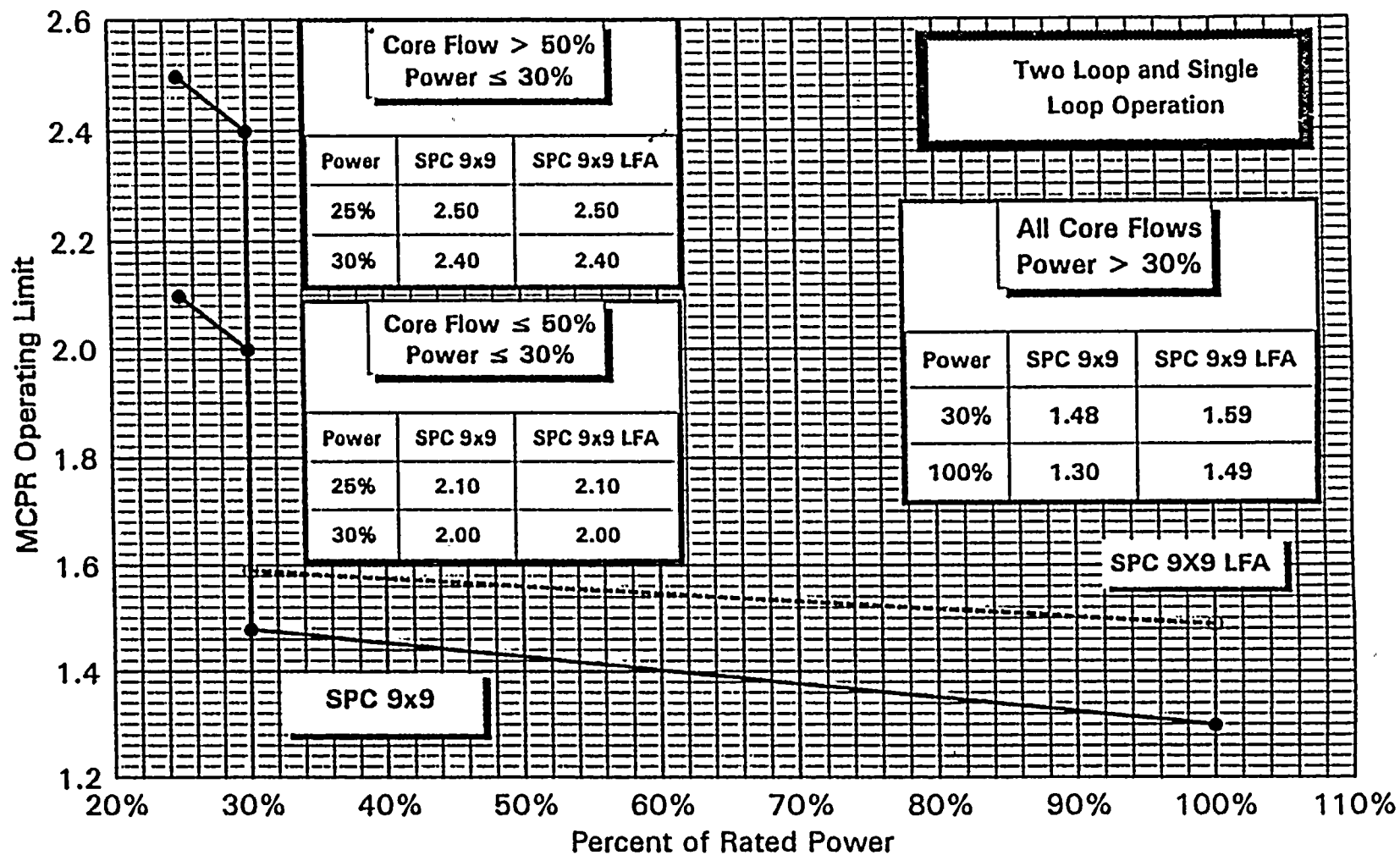




Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
TSSS, RPT Operable
SPC 9x9, SPC 9x9 LFA
Cycle Exposures ≤ 4500 MWd/MT

Figure 3.5a





Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
TSSS, RPT Operable
SPC 9x9, SPC 9x9 LFA
Cycle Exposures > 4500 MWd/MT

Figure 3.5b



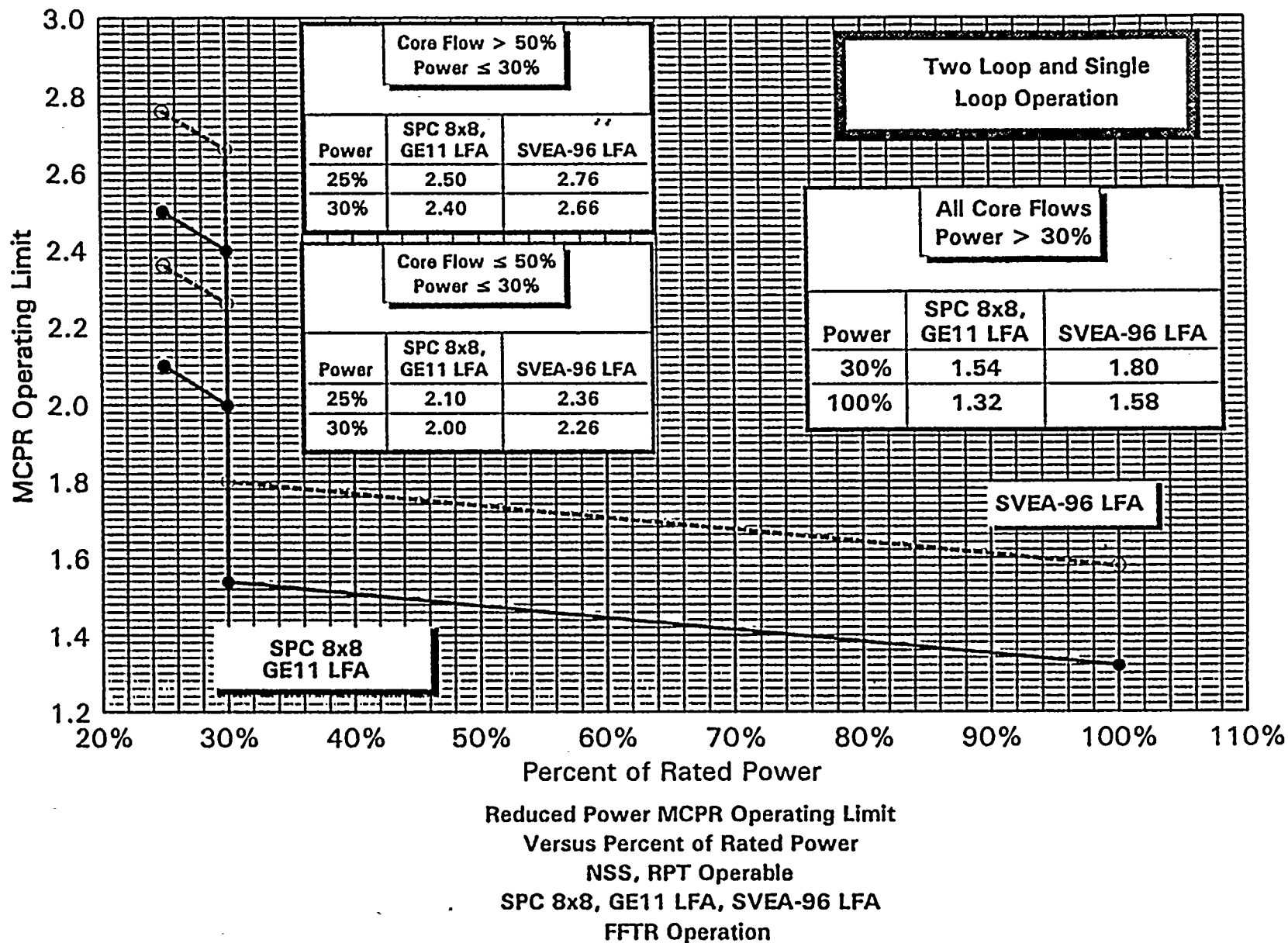


Figure 3.6

8/2/77

10/10/77

11/11/77

12/12/77

13/13/77

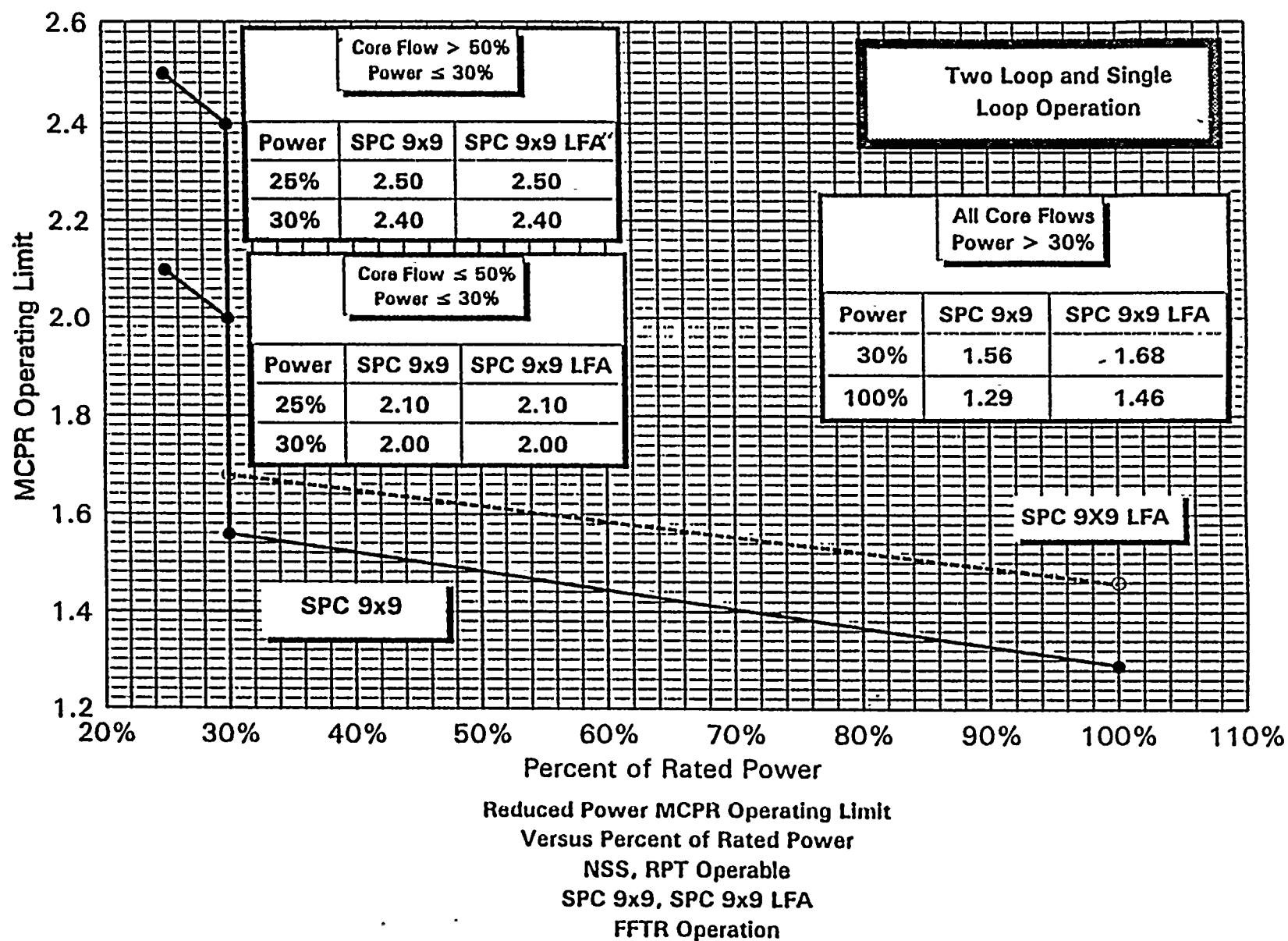


Figure 3.7

100

100

100

100

100

100

100

100

100

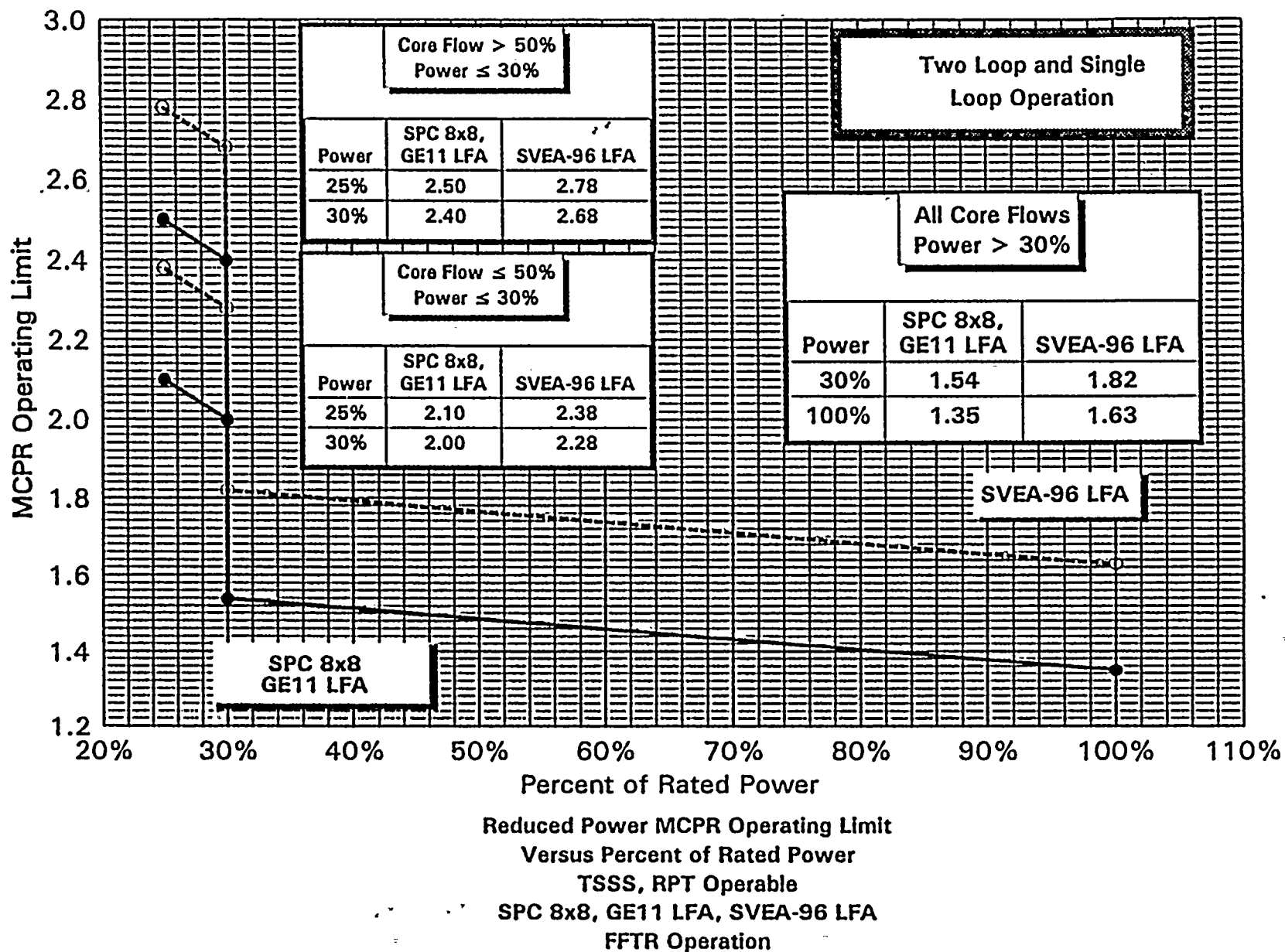


Figure 3.8

100

100

100

100

100

100

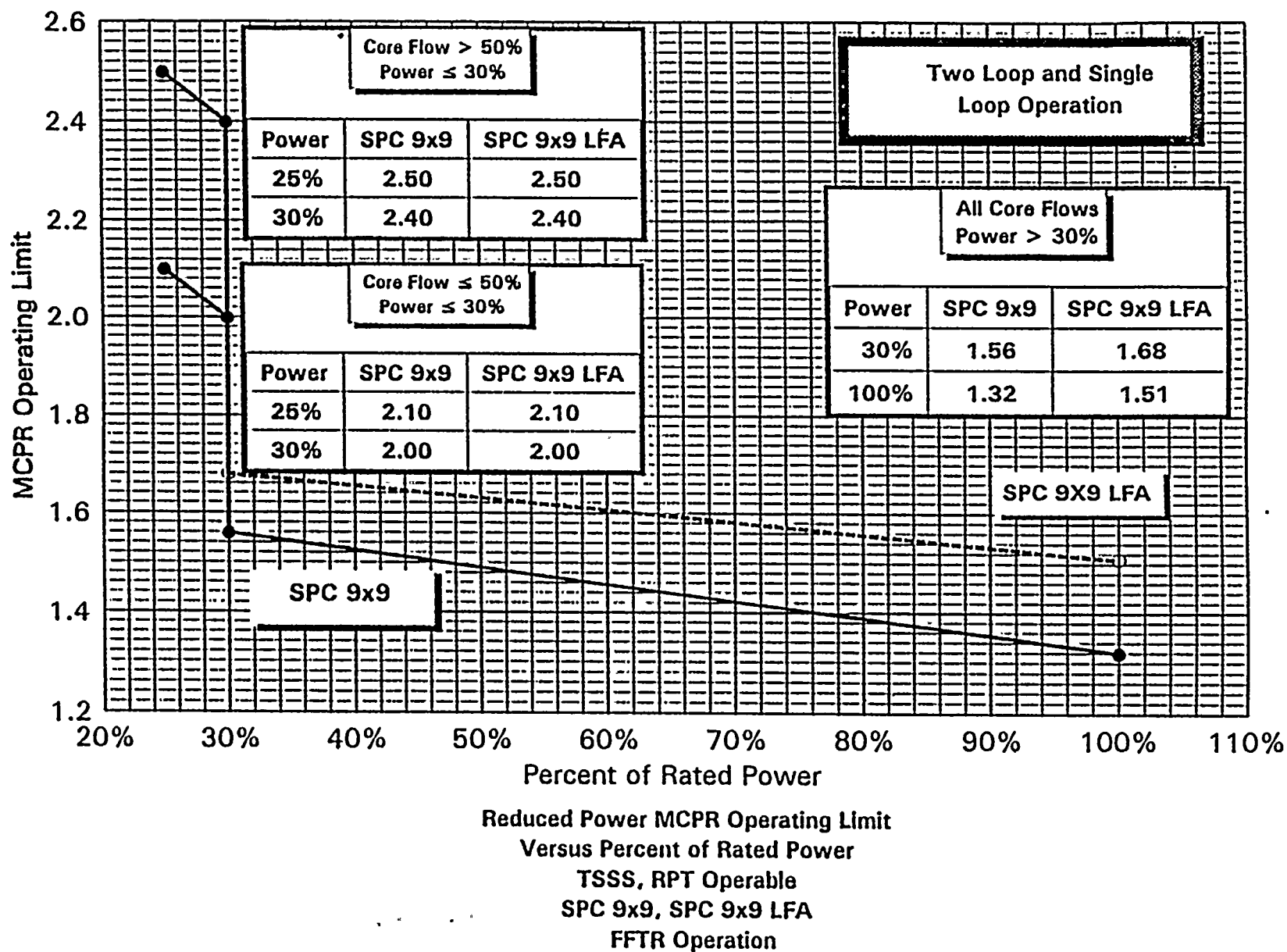


Figure 3.9

1

2

3

4

5

6

7

8

9

10

11

12

13

14

15

16

17

18

19

20

21

22

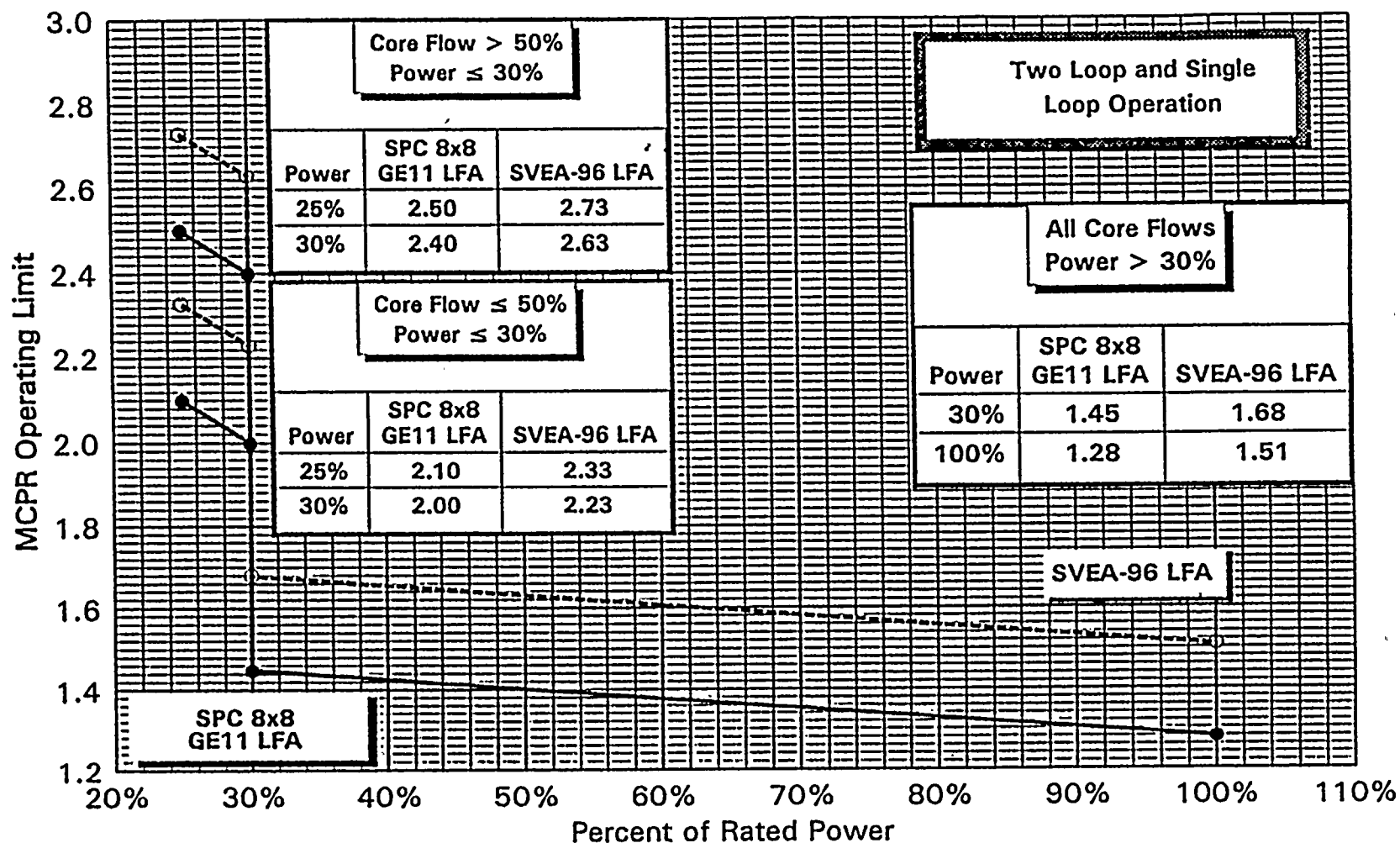
23

24

25

26

27



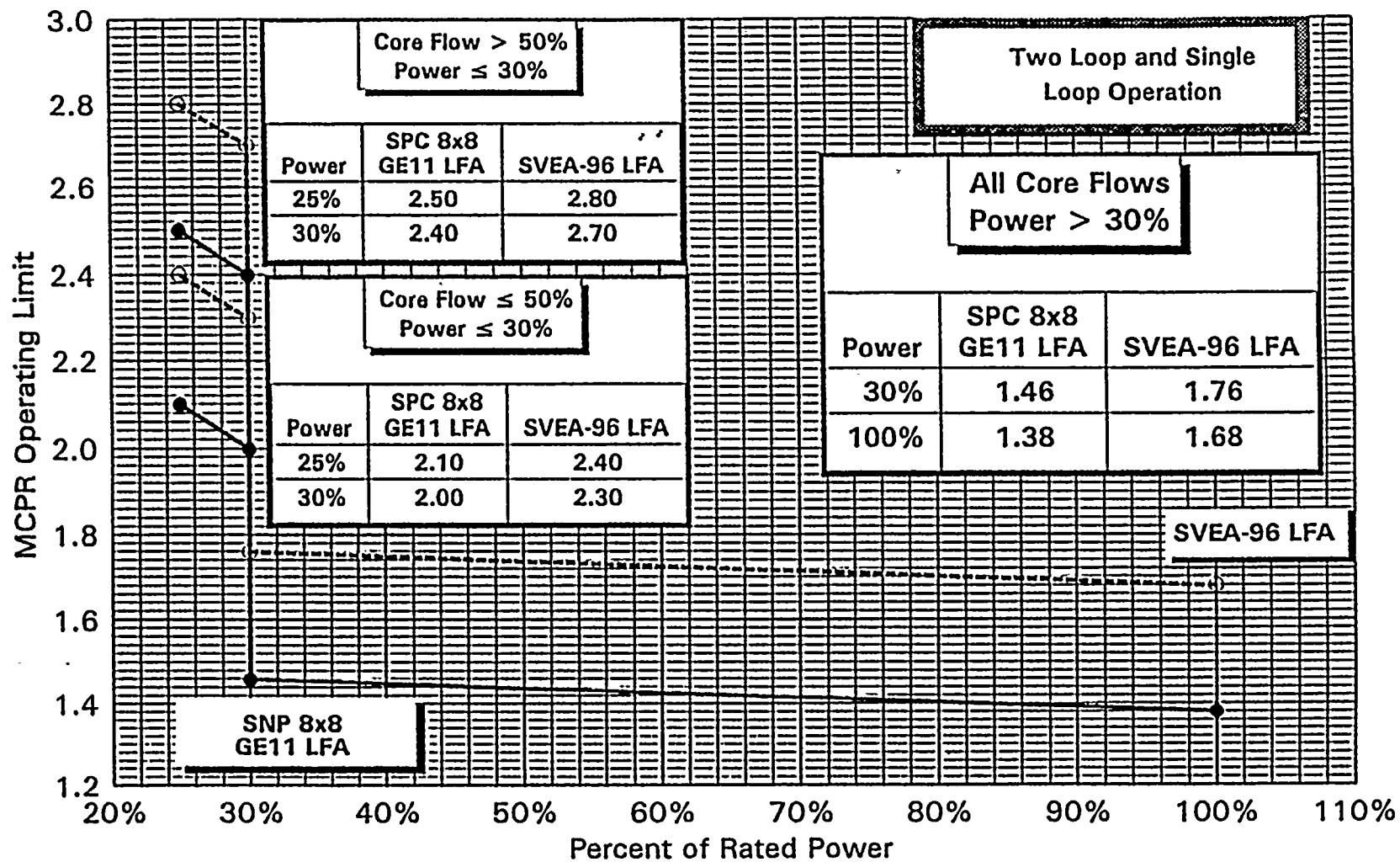
Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Inoperable
SPC 8x8, GE11 LFA, SVEA-96 LFA
Cycle Exposures ≤ 4500 MWd/MT

Figure 3.10a

1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

3





Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Inoperable
SPC 8x8, GE11 LFA, SVEA-96 LFA
Cycle Exposures > 4500 MWd/MT

Figure 3.10b

;

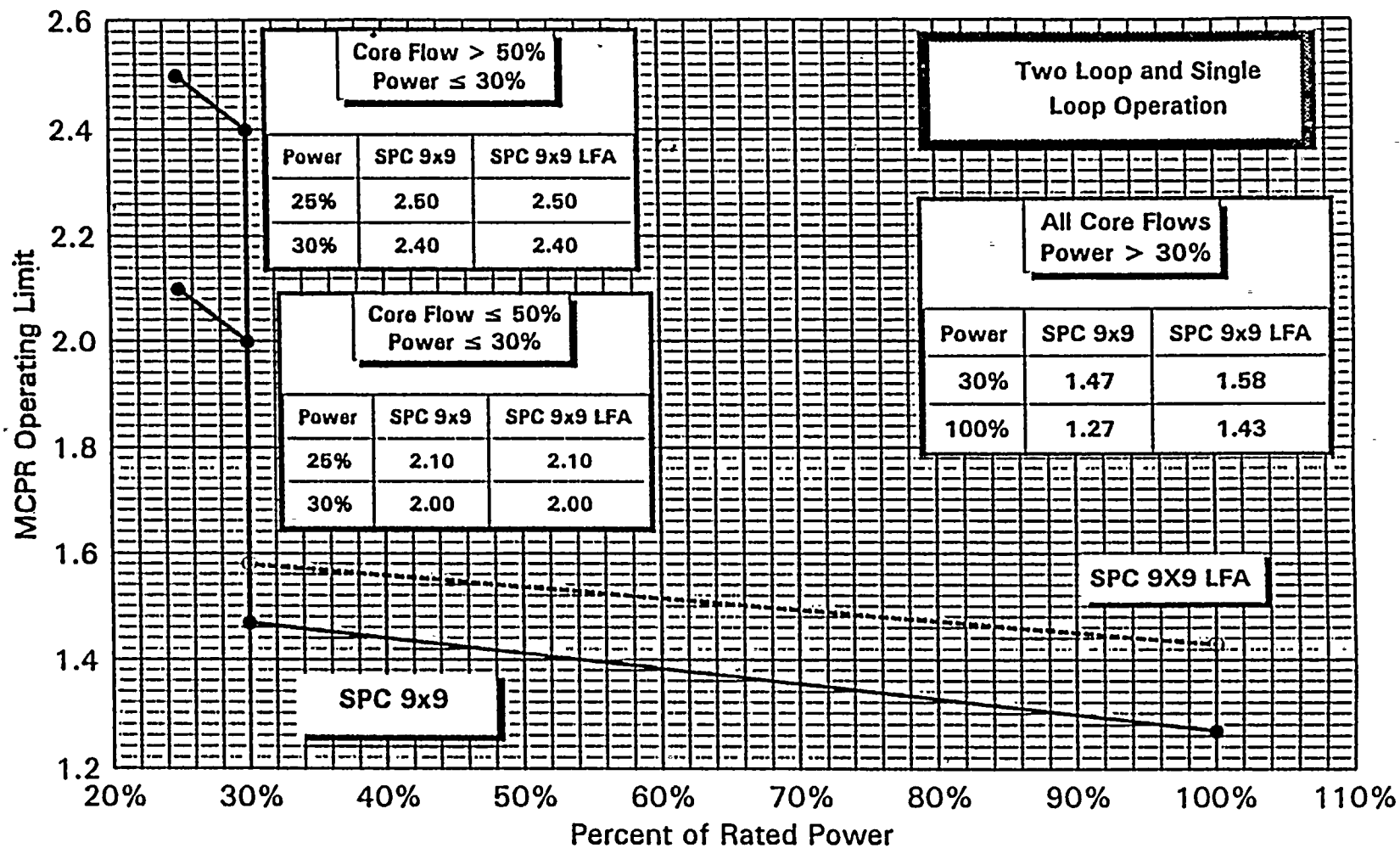
[illegible]

1
 2
 3
 4
 5
 6
 7
 8
 9
 10
 11
 12
 13
 14
 15
 16
 17
 18
 19
 20
 21
 22
 23
 24
 25
 26
 27
 28
 29
 30
 31
 32
 33
 34
 35
 36
 37
 38
 39
 40
 41
 42
 43
 44
 45
 46
 47
 48
 49
 50
 51
 52
 53
 54
 55
 56
 57
 58
 59
 60
 61
 62
 63
 64
 65
 66
 67
 68
 69
 70
 71
 72
 73
 74
 75
 76
 77
 78
 79
 80
 81
 82
 83
 84
 85
 86
 87
 88
 89
 90
 91
 92
 93
 94
 95
 96
 97
 98
 99
 100
 101
 102
 103
 104
 105
 106
 107
 108
 109
 110
 111
 112
 113
 114
 115
 116
 117
 118
 119
 120
 121
 122
 123
 124
 125
 126
 127
 128
 129
 130
 131
 132
 133
 134
 135
 136
 137
 138
 139
 140
 141
 142
 143
 144
 145
 146
 147
 148
 149
 150
 151
 152
 153
 154
 155
 156
 157
 158
 159
 160
 161
 162
 163
 164
 165
 166
 167
 168
 169
 170
 171
 172
 173
 174
 175
 176
 177
 178
 179
 180
 181
 182
 183
 184
 185
 186
 187
 188
 189
 190
 191
 192
 193
 194
 195
 196
 197
 198
 199
 200
 201
 202
 203
 204
 205
 206
 207
 208
 209
 210
 211
 212
 213
 214
 215
 216
 217
 218
 219
 220
 221
 222
 223
 224
 225
 226
 227
 228
 229
 230
 231
 232
 233
 234
 235
 236
 237
 238
 239
 240
 241
 242
 243
 244
 245
 246
 247
 248
 249
 250
 251
 252
 253
 254
 255
 256
 257
 258
 259
 260
 261
 262
 263
 264
 265
 266
 267
 268
 269
 270
 271
 272
 273
 274
 275
 276
 277
 278
 279
 280
 281
 282
 283
 284
 285
 286
 287
 288
 289
 290
 291
 292
 293
 294
 295
 296
 297
 298
 299
 300
 301
 302
 303
 304
 305
 306
 307
 308
 309
 310
 311
 312
 313
 314
 315
 316
 317
 318
 319
 320
 321
 322
 323
 324
 325
 326
 327
 328
 329
 330
 331
 332
 333
 334
 335
 336
 337
 338
 339
 340
 341
 342
 343
 344
 345
 346
 347
 348
 349
 350
 351
 352
 353
 354
 355
 356
 357
 358
 359
 360
 361
 362
 363
 364
 365
 366
 367
 368
 369
 370
 371
 372
 373
 374
 375
 376
 377
 378
 379
 380
 381
 382
 383
 384
 385
 386
 387
 388
 389
 390
 391
 392
 393
 394
 395
 396
 397
 398
 399
 400
 401
 402
 403
 404
 405
 406
 407
 408
 409
 410
 411
 412
 413
 414
 415
 416
 417
 418
 419
 420
 421
 422
 423
 424
 425
 426
 427
 428
 429
 430
 431
 432
 433
 434
 435
 436
 437
 438
 439
 440
 441
 442
 443
 444
 445
 446
 447
 448
 449
 450
 451
 452
 453
 454
 455
 456
 457
 458
 459
 460
 461
 462
 463
 464
 465
 466
 467
 468
 469
 470
 471
 472
 473
 474
 475
 476
 477
 478
 479
 480
 481
 482
 483
 484
 485
 486
 487
 488
 489
 490
 491
 492
 493
 494
 495
 496
 497
 498
 499
 500
 501
 502
 503
 504
 505
 506
 507
 508
 509
 510
 511
 512
 513
 514
 515
 516
 517
 518
 519
 520
 521
 522
 523
 524
 525

Figure 1. The effect of the concentration of the H_2O_2 solution on the amount of the released H_2 gas from the H_2 gas-generating system. The amount of the released H_2 gas was measured at 25 °C for 10 min. The concentration of the H_2O_2 solution was 0.01, 0.02, 0.05, 0.1, 0.2, 0.5, 1, 2, 5, 10, 20, 50, 100, 200, 500, 1000, and 2000 ppm. The amount of the released H_2 gas was measured at 25 °C for 10 min. The concentration of the H_2O_2 solution was 0.01, 0.02, 0.05, 0.1, 0.2, 0.5, 1, 2, 5, 10, 20, 50, 100, 200, 500, 1000, and 2000 ppm.

17

9



Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Inoperable
SPC 9x9, SPC 9x9 LFA
Cycle Exposures ≤ 4500 MWd/MT

Figure 3.11a

10



10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

10

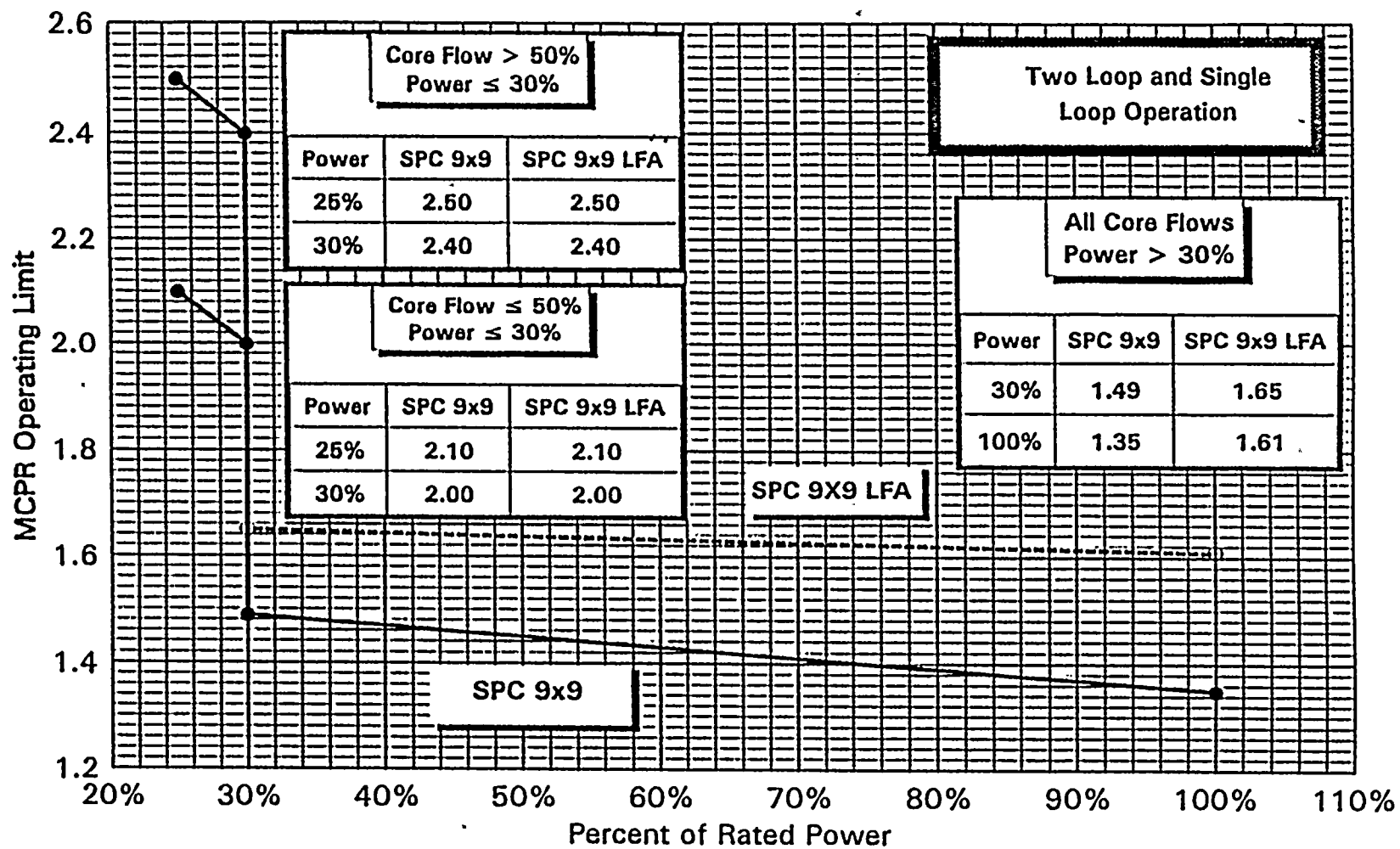
10

10

10

10





Reduced Power MCPR Operating Limit
Versus Percent of Rated Power
NSS, RPT Inoperable
SPC 9x9, SPC 9x9 LFA
Cycle Exposures > 4500 MWd/MT

Figure 3.11b

5
1
2
3
4

100-1000000



4.0 LINEAR HEAT GENERATION RATE (LHGR) LIMIT FOR USE IN TECHNICAL SPECIFICATION 3.2.4

The LHGR limit for use in Technical Specification 3.2.4 shall not exceed the values shown in Figures 4.1 through 4.5.

1000

1000

1000

1000

1000

1000

1000

1000

1000

1000



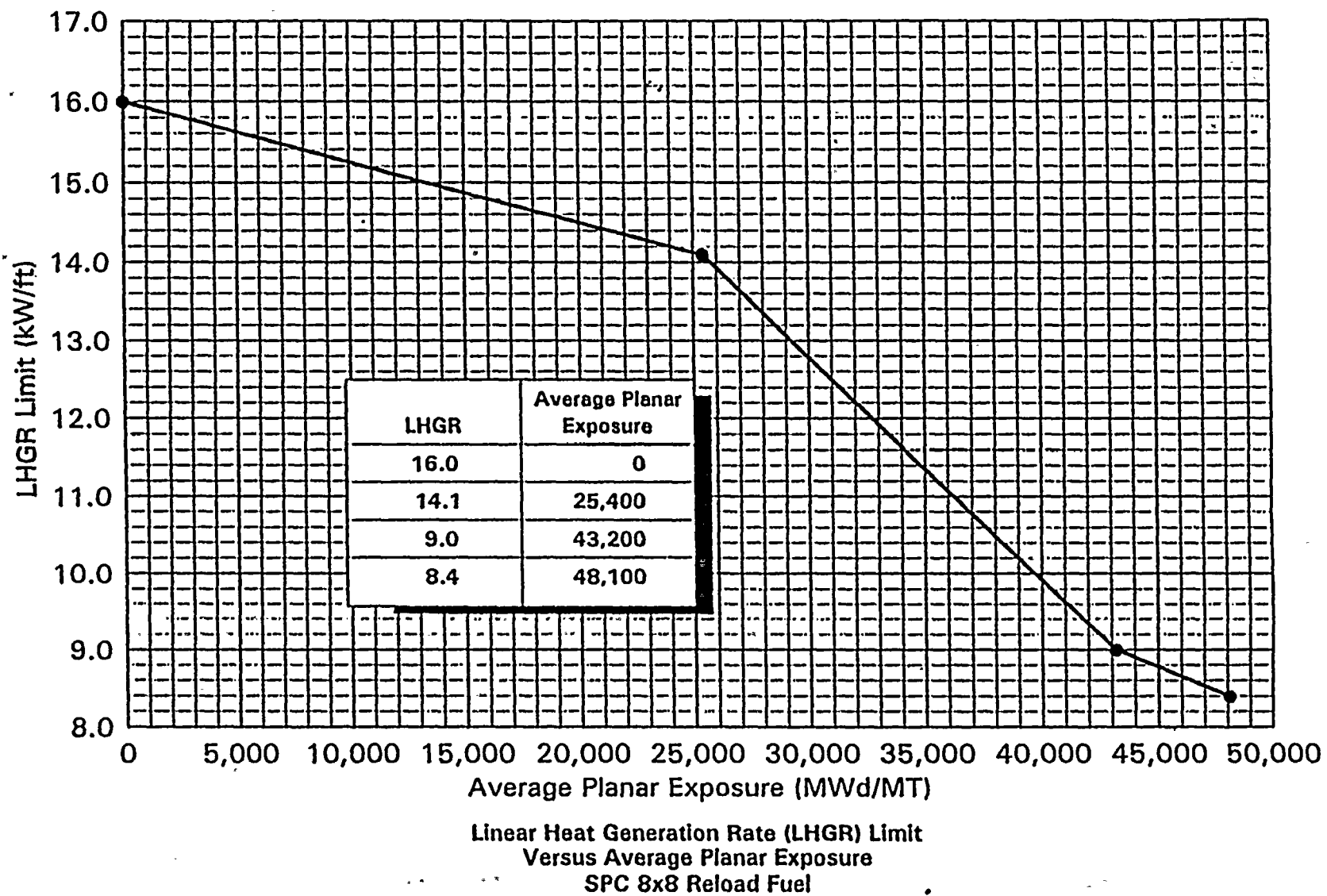


Figure 4.1

100

100

100

100

100

100

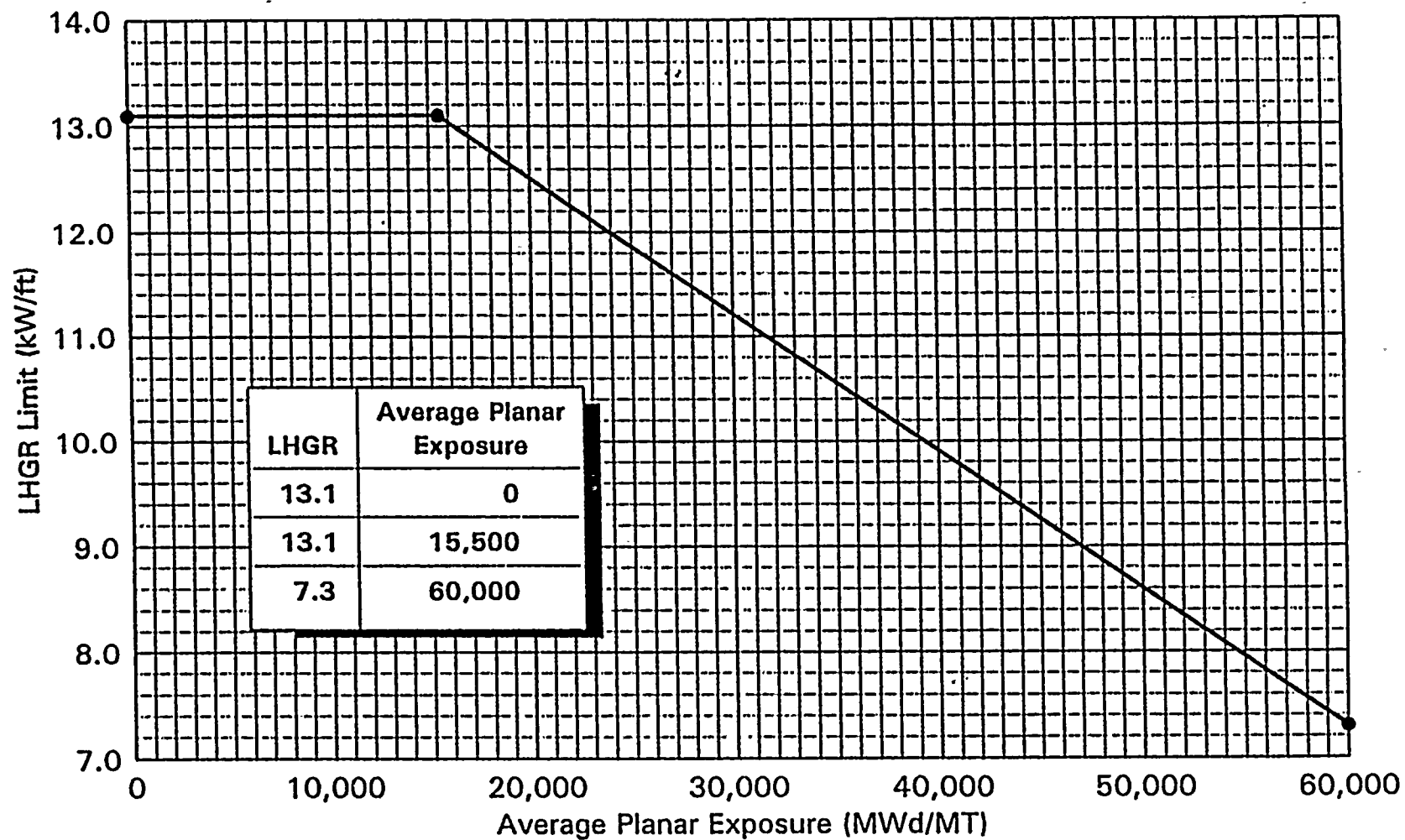
100

100

100

100





Linear Heat Generation Rate (LHGR) Limit
Versus Average Planar Exposure
SPC 9x9-9X Reload Fuel and SPC 9x9-9X LFA Fuel

Figure 4.2



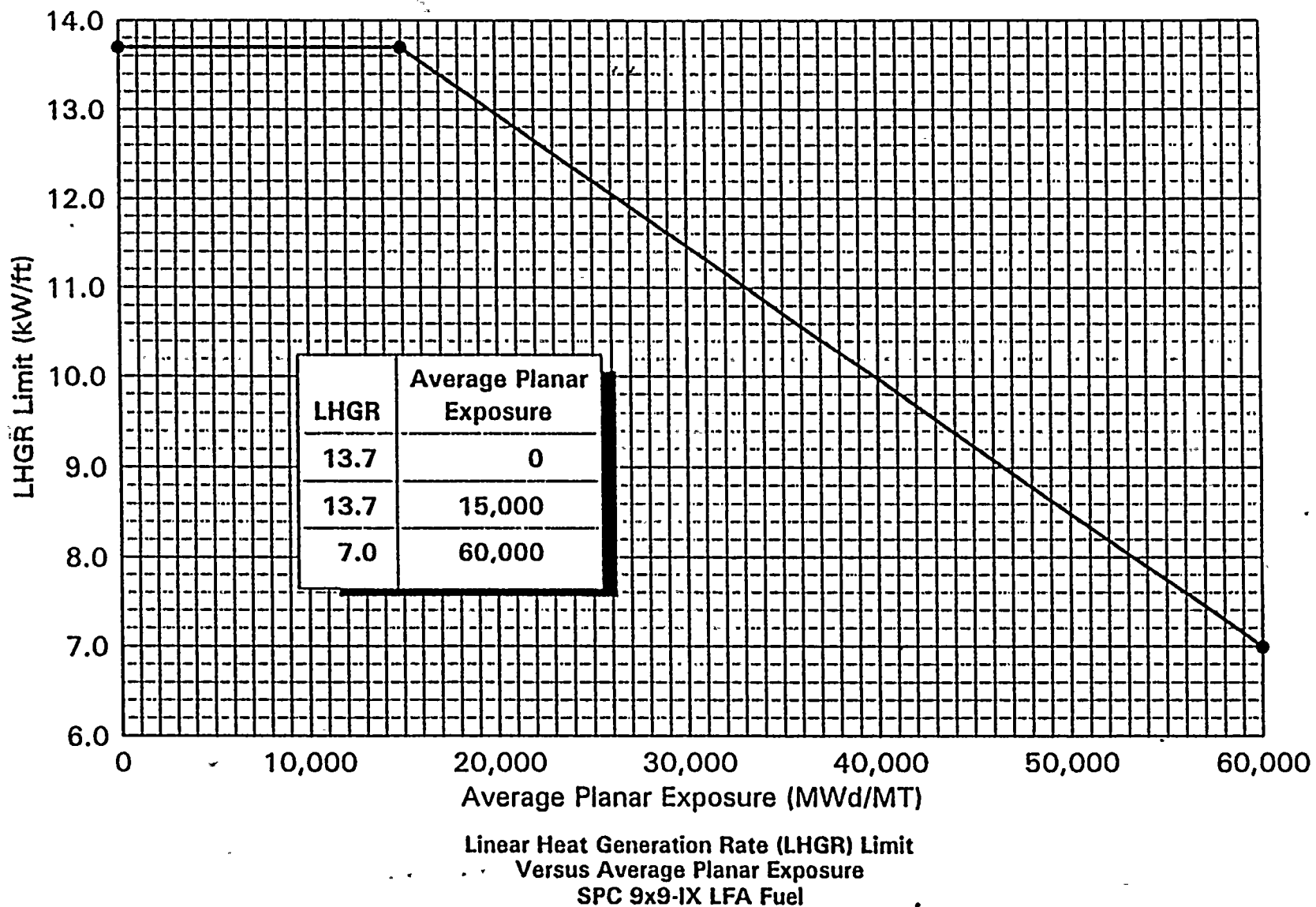


Figure 4.3

2
10
10

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

100 100

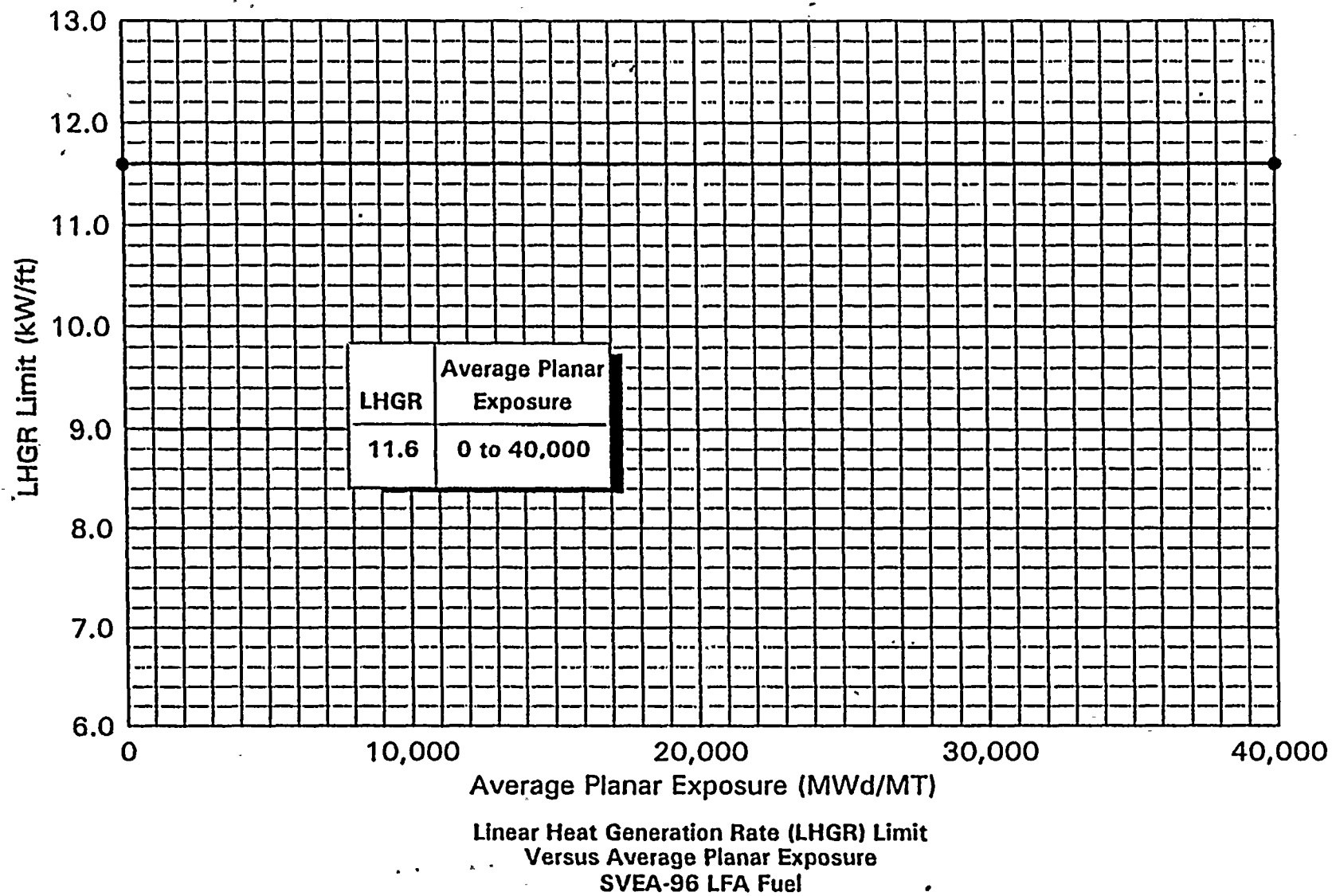


Figure 4.4

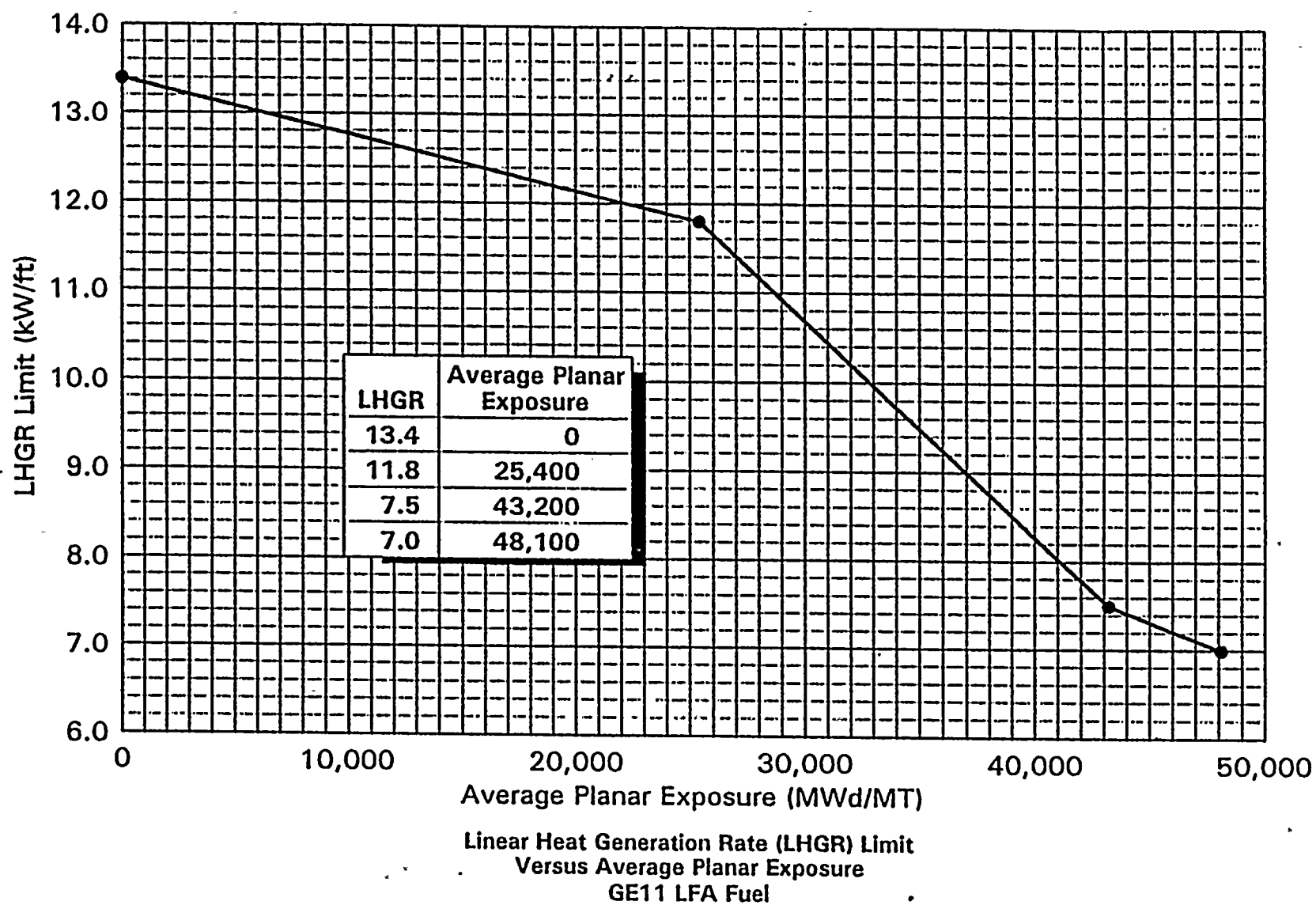


Figure 4.5

3



5.0 REFERENCES

5.1 Reports for Current Cycle

- 5.1.1 EMF-94-095, "WNP-2 Cycle 10 Plant Transient Analysis," Siemens Power Corporation, June 1994.
- 5.1.2 EMF-94-096, "WNP-2 Cycle 10 Reload Analysis," Siemens Power Corporation, June 1994.
- 5.1.3 SPCWP-94-041, "Licensing Results Supporting Section 3/4.2 of the WNP-2 Technical Specifications for Cycle 10," Letter from YU Fresk, Siemens Power Corporation, to RA Vopalensky, Supply System, April 1, 1994.
- 5.1.4 SPCWP-94-062, "STAIF Stability Results in Support of WNP-2 Cycle 10," Letter from YU Fresk, Siemens Power Corporation, to RA Vopalensky, Supply System, June 14, 1994.
- 5.1.5 SPCWP-94-042, "Licensing Results Supporting Section 2.1 of the WNP-2 Technical Specifications for Cycle 10," Letter from YU Fresk, Siemens Power Corporation, to RA Vopalensky, Supply System, April 1, 1994.
- 5.1.6 SPCWP-94-068, "SPC Comments on WNP-2 Cycle 10 Draft COLR," Letter from YU Fresk, Siemens Power Corporation, to RA Vopalensky, Supply System, June 23, 1994.
- 5.1.7 RDW:94-092, "WNP-2 Cycle 9 Core Operating Limits Report - GE11 Lead Use Assemblies," Letter from RD Williams, GE Nuclear Energy, to DL Whitcomb, Supply System, June 21, 1994.
- 5.1.8 ABBWP-94-040, "SVEA-96 Lead Fuel Assembly Treatment in WNP-2 Cycle 10 Core Operating Limits Report," Letter from CG Schon, ABB Combustion Engineering Nuclear Operations, to RA Vopalensky, Supply System, June 15, 1994.

5.2 Licensing Topical Reports in Technical Specification 6.9.3.2

- 5.2.1 ANF-1125(P)(A) and Supplements 1 and 2, "ANFB Critical Power Correlation," Advanced Nuclear Fuels Corporation, April 1990.
- 5.2.2 "NRC Approval of ANFB Additive Constants for 9x9-9X BWR Fuel," Letter from RC Jones, NRC, to RA Copeland, Advanced Nuclear Fuels Corporation, November 14, 1990.

- 5.2.3 ANF-524(P)(A), Revision 2 and Supplements 1 and 2, "Advanced Nuclear Fuels Critical Power Methodology for Boiling Water Reactors," Advanced Nuclear Fuels Corporation, November 1990.
- 5.2.4 ANF-913(P)(A), Volume 1, Revision 1 and Volume 1, Supplements 2, 3, and 4, "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analyses," Advanced Nuclear Fuels Corporation, August 1990.
- 5.2.5 ANF-CC-33(P)(A), Supplement 2, "HUXY: A Generalized Multirod Heatup Code with 10 CFR 50, Appendix K, Heatup Option," Advanced Nuclear Fuels Corporation, January 1991.
- 5.2.6 XN-NF-80-19(P)(A), Volume 1, Supplements 3 and 4, "Advanced Nuclear Fuels Methodology for Boiling Water Reactors: Benchmark Results for the CASMO-3G/MICROBURN-B Calculation Methodology," Advanced Nuclear Fuels Corporation, November 1990.
- 5.2.7 XN-NF-80-19(P)(A), Volume 4, Revision 1, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads," Exxon Nuclear Company, Inc., June 1986.
- 5.2.8 XN-NF-80-19(P)(A), Volume 3, Revision 2, "Exxon Nuclear Methodology for Boiling Water Reactors THERMEX: Thermal Limits Methodology Summary Description," Exxon Nuclear Company, Inc., January 1987.
- 5.2.9 XN-NF-85-67(P)(A), Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel," Exxon Nuclear Company, Inc., September 1986.
- 5.2.10 ANF-89-014(P)(A), Revision 1 and Supplements 1 and 2, "Generic Mechanical Design for Advanced Nuclear Fuels 9x9-IX and 9x9-9X Reload Fuel," Advanced Nuclear Fuels Corporation, October 1991.
- 5.2.11 XN-NF-81-22(P)(A), "Generic Statistical Uncertainty Analysis Methodology," Exxon Nuclear Company, Inc., November 1983.
- 5.2.12 NEDE-24011-P-A-6, "General Electric Standard Application for Reactor Fuel," GE Nuclear Energy, April 1983.
- 5.3 GE Nuclear Energy and ABB Combustion Engineering Nuclear Operations Lead Fuel Assembly Reports
 - 5.3.1 "GE11 Lead Fuel Assembly Report for Washington Public Power Supply System Nuclear Project No. 2, Reload 5, Cycle 6," GE Nuclear Energy, December 1989.

205



- 5.3.2 UK 90-126, "Supplemental Lead Fuel Assembly Licensing Report—SVEA-96 LFAs for WNP-2—Summary," ABB Atom, January 1990.
 - 5.3.3 ATOF-91-120, "Assembly Treatment in WNP-2 Cycle 7 Core Operating Limits Report," Letter from WR Harris, ABB Atom, to DL Whitcomb, Supply System, May 1, 1991.
 - 5.3.4 ABBWP-94-039, "WNP-2 SVEA-96 Lead Fuel Assembly Operating Limit MCPR," Letter from CG Schon, ABB Combustion Engineering Nuclear Operations, to RA Vopalensky, Supply System, June 15, 1994.
- 5.4 Reports for the Extended Load Line Limit Analysis (ELLLA)
- 5.4.1 "Reactor Vessel Internals Evaluation Task Report for WNP-2 Power Uprate Project," GE Nuclear Energy, April 1993 (DRAFT).
 - 5.4.2 "WNP-2 Power Uprate Containment Response Evaluation Input to Engineering Report," GE Nuclear Energy, January 19, 1993 (DRAFT).
 - 5.4.3 GE-NE-189-69-1092, "Effects of Adjustable Speed Drive on Reactor Internal Vibration at the WNP-2 Nuclear Power Plant," GE Nuclear Energy, October 1992.
 - 5.4.4 GE-NE-189-34-0392, "Jet Pump Sensing Line Vibration Test for Washington Nuclear Project 2," GE Nuclear Energy, March 1992.
 - 5.4.5 NEDE-24222, "Assessment of BWR Mitigation of ATWS, Vol. II (NUREG 0460, Alternate No. 3)," General Electric Company, December 1979.
 - 5.4.6 "Washington Nuclear Project Unit 2 System Evaluation Report for Power Uprate—Reactor Recirculation Control System," GE Nuclear Energy, February 1, 1993.
 - 5.4.7 GE Report 22A7104, Revision 0, "Dynamic Load Report—Fuel Vertical Support," GE Nuclear Energy, June 30, 1982.
 - 5.4.8 "Fuel Lift Non-Proprietary Letter," Letter from DM Kelly, GE Nuclear Energy, to WC Wolkenhauer, Supply System, February 15, 1993.
 - 5.4.9 93-PU-0054, "ELLLA Related Power Uprate Task Reports," Letter from DM Kelly, GE Nuclear Energy, to WC Wolkenhauer, Supply System, June 3, 1993.

11/14/74

