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 FULLER, R.E. Washington Public Power Supply System  
 BAKER, J.W. Washington Public Power Supply System  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 91-036-00: on 911226, a missed ASME Section XI Inservice Test for FPC relief valve determined to violate TS requirements. Cause not identified. Perform required ASME surveillance testing on FPC-RV-117B.W/920127 ltr.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

January 27, 1992  
G02-92-021

Docket No. 50-397

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 91-036

Dear Sir:

Transmitted herewith is Licensee Event Report No. 91-036 for the WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the items of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

J. W. Baker (M/D 927M)  
WNP-2 Plant Manager

Enclosure:  
Licensee Event Report No. 91-036

cc: Mr. John B. Martin, NRC - Region V  
Mr. C. Sorensen, NRC Resident Inspector (M/D 901A)  
INPO Records Center - Atlanta, GA  
Ms. Dottie Sherman, ANI  
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LICENSEE EVENT REPORT (LER)	
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FACILITY NAME (1)  
Washington Nuclear Plant - Unit 2

DOCKET NUMBER (2)								PAGE (3)			
0	5	0	0	0	3	9	7	1	OF	3	

**TITLE (4)**

TITLE (4)

MISSED ASME SECTION XI SURVEILLANCE

EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																	
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES						DOCKET NUMBERS(S)											
1	2	2	6	9	1	9	1	--	0	3	6	--	0	0	0	1	2	7	9	2							0	5	0	0	0				

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10)	1	0	0	20.402(b)	20.405(C)	50.73(a)(2)(iv)	77.71(b)
				20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.73(c)
				20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract
				20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	below and in Text, NRC
				20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	Form 366A)
				20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER										
	AREA CODE										
R.E. Fuller, Compliance Engineer	5	0	9	3	7	7	-	4	1	4	8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

[illegible]

SUPPLEMENTAL REPORT EXPECTED (14)

☒ YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO

EXPECTED SUBMISSION DATE (15)	MONTH 02	DAY 27	YEAR 92
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EXPECTED SUBMISSION DATE (15)	MONTH 02	DAY 27	YEAR 92
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ABSTRACT (16)

On December 26, 1991, a missed ASME Section XI Inservice Test for Fuel Pool Cooling (FPC) relief valve, FPC-RV-117B, was determined to violate the requirements of Technical Specification Sections 4.0.2 and 4.0.5. This oversight was identified by a Maintenance Engineer on December 20, 1991.

The immediate corrective action was to declare FPC-RV-117B inoperable. There are no Technical Specification requirements for the Fuel Pool Cooling System, therefore no action statements were required to be entered.

The root cause analysis has not been finalized for this event. The results will be provided in a supplement to this report.

The corrective action was to perform the required ASME surveillance testing on FPC-RV-117B.

There is no safety significance associated with this event. There was no condition that challenged the operation of this relief valve. The test history of this valve indicates it would perform its safety function. This event posed no threat to the health or safety of the public or of Plant personnel.

<b>LICENSEE EVENT REPORT (LER)</b> <b>TEXT CONTINUATION</b>											
<b>FACILITY NAME (1)</b> Washington Nuclear Plant - Unit 2		<b>DOCKET NUMBER (2)</b> 0   5   0   0   0   3   9   7					<b>LER NUMBER (8)</b>			<b>PAGE (3)</b>	
							Year 91	Number 036	Rev. No. 00	2	OF 3
<b>TITLE (4) MISSED ASME SECTION XI SURVEILLANCE</b>											

### Plant Conditions

Power Level - 100%

Plant Mode - 1

### Event Description

On December 26, 1991, a missed ASME Section XI Inservice Test for Fuel Pool Cooling (FPC) relief valve, FPC-RV-117B, was determined to violate the requirements of Technical Specification Sections 4.0.2 and 4.0.5. This oversight was identified by a Maintenance Engineer on December 20, 1991.

Plant Technical Specification Surveillance Procedure PPM 7.4.0.5.20 is initiated annually to test designated safety related relief valves as required by the ASME Boiler and Pressure Vessel Code, Section XI. The procedure and program controls are intended to ensure that all valves are tested within the required five years. FPC-RV-117B needed to be tested during the 1991 refueling outage, R-6, to satisfy the ASME Code five year requirement. FPC-RV-117B was last tested April 8, 1986 and was later found to have not been tested during the R-6 outage.

### Immediate Corrective Action

FPC-RV-117B was declared inoperable December 26, 1991. There are no Technical Specification requirements for the Fuel Pool Cooling System, therefore no action statements were required to be entered. The test history of this valve indicates it would perform its safety function. Also, there are two trains of FPC and only one train was affected.

### Further Evaluation and Corrective Action

#### A. Further Evaluation

1. This event is reportable per 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the Plant's Technical Specifications. Technical Specification Section 4.0.5 requires in part that inservice testing of ASME Code Class 1, 2, and 3 valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50, Section 50.55a(g). The code requires that all valves must be tested within five years. FPC-RV-117B had been tested on April 8, 1986. The last date for testing the valve within the five year requirement was the last day of the refueling outage or June 11, 1991. After that date, the valve should have been declared inoperable until testing was performed to verify operability.
2. There were no other structures, components, or systems inoperable prior to the event which contributed to the event.
3. The root cause analysis has not been finalized for this event. The results will be provided in a supplement to this report.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION																
FACILITY NAME (1) Washington Nuclear Plant - Unit 2		DOCKET NUMBER (2) 0   5   0   0   0   3   9   7			LER NUMBER (8) <table border="1"> <tr> <td>Year</td> <td>Number</td> <td>Rev. No.</td> </tr> <tr> <td>91</td> <td>036</td> <td>00</td> </tr> </table>			Year	Number	Rev. No.	91	036	00	PAGE (3) 3 OF 3		
Year	Number	Rev. No.														
91	036	00														
TITLE (4) MISSED ASME SECTION XI SURVEILLANCE																

B. Further Corrective Action

1. The required ASME surveillance testing on FPC-RV-117B will be performed prior to January 31, 1992.

Safety Significance

There is no safety significance associated with this event. Should one of the FPC trains be unavailable, the second train can maintain pool water temperature below 155 degrees F. FSAR Section 9.1.3.3, the safety evaluation for the Fuel Pool Cooling System, states "One of the two 50% capacity FPC trains is adequate to prevent fuel pool boiling by a large margin." There was no condition that challenged the operation of this relief valve. In addition, previous testing of the relief valve indicated no history of failures. Therefore, the relief valve would likely have performed its safety function if it was required to do so. This event posed no threat to the health or safety of the public or of Plant personnel.

Similar Events

LER 91-13 documents a similar event where ASME required quarterly surveillances of selected valves of the Main Steam Leakage Control (MSLC) System were being missed. The ASME surveillance procedure for the valves had a precaution to test them only during cold shutdown which conflicted with the quarterly surveillance requirement. Justification was provided to allow them to be tested during cold shutdown.

EIIS Information

Text Reference

EIIS Reference

	<u>System</u>	<u>Component</u>
Relief Valve for Fuel Pool Cooling and Cleanup System	DA	RV
Main Steam System	SB	V