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 PARRISH, J.V. Washington Public Power Supply System  
 RECIP. NAME RECIPIENT AFFILIATION  
 Document Control Branch (Document Control Desk)

SUBJECT: Forwards GE Rept R-R8-111 re flaw in reactor bldg piping.  
 Review of program output for fatigue analysis & IGSCC  
 fracture mechanics showed current crack size well within  
 limit of util evaluation. Indication will be monitored.

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May 21, 1993  
G02-93-119

Docket No. 50-397

U.S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

Subject: **WNP-2, OPERATING LICENSE NPF-21  
REPORT ON FLAW IN REACTOR RECIRCULATION PIPING**

- References:
- 1) Letter, G02-92-123, dated May 14, 1992, GC Sorensen (SS) to NRC, "Report on Flaw in Reactor Recirculation Piping (TAC No. 80358)"
  - 2) Letter dated June 25, 1992, RR Assa (NRC) to GC Sorensen (SS), "Review of Updated Report on Reactor Recirculation Piping Flaw at Washington Public Power Supply System (WPPSS) Nuclear Reactor Number 2 (TAC No. M83721)"

The indication in ISI weld 20RRC(6)-8, as discussed in Reference 1, was reexamined, for the second successive time May 12, 1993, to determine any size change. No significant changes in the indication depth nor signal characteristics were noted. The flaw depth was found to be 0.175 inch (0.17 inch at R-7). The slight difference in depth is attributed to the minor differences in calibration and data point locations along the indication length. The length, now reported as 3.6 inches is based on the EPRI NDE practice for IGSCC detection of length based on reference gain. The length previously reported (4.5 inches) was based on 12db over reference to determine initial length of flaw based on CE-2 signal disappearing into baseline. Both lengths were reported in the R7 NDE data report 1RRU-166 enclosed with Reference 1. The length of the flaw remains unchanged. The indication does not exhibit the UT signals characteristic of IGSCC.

The initial fracture mechanics analysis was reviewed using the current indication depth of 0.175 and length of 3.62 inches. The analysis predicts a crack depth of 0.38 inches after 740 days from the initial examination in May of 1991, if the IGSCC mechanism is active. The analysis also predicts that the depth would exceed the maximum allowable depth of 0.62 inches after 4 additional years. The current flaw depth is 0.175 inches. This depth is approximately 10.8%

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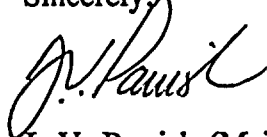
**REPORT ON FLAW IN REACTOR RECIRCULATION PIPING**

of the predicted growth as determined by analysis. Review of the program output for both the fatigue analysis and the IGSCC fracture mechanics analysis showed the current crack size is well within the limits of our evaluation that was previously performed. Therefore, based on the analysis and the examination results, the indication is determined to be acceptable for continued operation without repair until the next examination.

The Supply System will continue to monitor this indication. At refueling outage R-9, Spring 1994, the indication will be resized. If it still does not show any signs of significant growth, it will be reanalyzed to justify changing the examination frequency. This weld is still classified as an IGSCC, Category "F" weld.

The results of this reexamination and evaluation are submitted for your review and approval for plant restart scheduled for Sunday June 13, 1993, (Reference 2).

Sincerely,



J. V. Parrish (Mail Drop 1023)  
Assistant Managing Director, Operations

DPR/bk

Enclosure: NDE Report No. R-R8-111

cc: JB Martin - NRC RV  
NS Reynolds - Winston & Strawn  
JW Clifford - NRC  
DL Williams - BPA/399  
NRC Site Inspector - 901A

